

Mr. William T. Cottle  
 President and Chief Executive Officer  
 STP Nuclear Operating Company  
 South Texas Project Electric  
 Generating Station  
 P. O. Box 289  
 Wadsworth, TX 77483

July 23, 1999

*Docket  
File*

SUBJECT: INSERVICE TESTING RELIEF REQUEST RR-56 (RR-52) - COMPONENT COOLING WATER AND SAFETY INJECTION SYSTEM CONTAINMENT ISOLATION CHECK VALVE CLOSURE TEST FREQUENCY, SOUTH TEXAS PROJECT, UNITS 1 AND 2 (STP) (TAC NOS. MA4211 AND MA4212)

Dear Mr. Cottle:

By letter dated November 16, 1998, STP Nuclear Operating Company (STPNOC) submitted relief request RR-56 for Unit 1 and RR-52 for Unit 2 for the first 10-year interval for the STP Inservice Testing (IST) Program. RR-56 (RR-52) requested relief from the ASME Section XI code requirement in order to extend testing of component cooling water and safety injection check valves for closure exercise to the same frequency as the 10 CFR Part 50, Appendix J, Option B frequency (i.e., from once each refueling outage to a periodic interval based on the safety significance and historical performance of each boundary and isolation valve to ensure the integrity of the overall containment system). By letter dated February 23, 1999, STPNOC amended this request providing additional information in support of the proposed test interval.

The Nuclear Regulatory Commission staff has reviewed proposed relief request RR-56 (RR-52) against the requirements of the 1983 Edition through Summer 1983 Addenda of Section XI of the American Society of Mechanical Engineers Boiler and Pressure Vessel Code. The staff concludes that relief request RR-56 (RR-52) is authorized for the life of STP pursuant to 10 CFR 50.55a(a)(3)(i) based on the alternative providing an acceptable level of quality and safety. The results of the staff's review are provided in the enclosed safety evaluation.

Sincerely,  
 ORIGINAL SIGNED BY  
 Robert A. Gramm, Chief, Section 1  
 Project Directorate IV & Decommissioning  
 Division of Licensing Project Management  
 Office of Nuclear Reactor Regulation

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 PDR

Docket Nos. 50-498 and 50-499

Enclosure: Safety Evaluation

cc w/encl: See next page

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