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EXISTING SPECIFICATION

ATTACHMENT A

NPF-38-216

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# ADMINISTRATIVE CONTROLS

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## HIGH RADIATION AREA (Continued)

- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate ievel in the area has been established and personnel have been made knowledgeable of them.
- c. A health physics qualified individual (i.e., qualified in radiation protection procedures) with a radiation dose rate monitoring device who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the facility Radiation Protection Superintendent-Nuclear in the RWP.

6.12.2 In addition to the requirements of Specification 6.12.1, areas accessible to personnei with radiation levels such that a major portion of the body could receive in one hour a dose greater than 1000 mrems\* but less than 500 rads\*\* shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Shift Superintendent on duty and/or health physics supervision/designee. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area. For individual areas accessible to personnel with radiation levels such that a major portion of the body could receive in 1 hour a dose in excess of 1000 mrems\* but less than 500 rads\*\* that are located within large areas, such as PWR containment, where no enclosure exists for purposes of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off, conspicuously posted and a flashing light shall be activated as a warning device. In lieu of the stay time specification of the RWP, direct or remote (such as use of closed circuit TV cameras) continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area.

### 6.13 PROCESS CONTROL PROGRAM (PCP)

6.13.1 The PCP shall be approved by the Commission prior to implementation.

6.13.2 Licensee-initiated changes to the PCP:

- a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.3p. This documentation shall contain:
  - Sufficient information to support the change together with the appropriate analyses or evaluation justifying the change(s) and

<sup>\*</sup>Measurement made at 30 centimeters from the radiation source or from any surface that the radiation penetrates.

<sup>\*\*</sup>Measurement made at 1 meter from the radiation source or from any surface that the radiation penetrates.

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## PROCESS CONTROL PROGRAM (Continued)

- A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.
- b. Shall become effective after review and acceptance by the PORC and the approval of the General Manager Plant Operations.

## 6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)

- 6.14.1 The ODCM shall be approved by the Commission prior to implementation.
- 6.14.2 Licensee-initiated changes to the ODCM:
  - a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.3p. This document shall contain:
    - Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
    - A determination that the change will maintain the level of radioactive effluent control required pursuant to 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose or setpoint calculations.
  - Shall become effective after review and acceptance by the PORC and the approval of the General Manager Plant Operations.
  - c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

## 6.15 CONTAINMENT LEAKAGE RATE TESTING PROGRAM

A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995.

The peak calculated containment internal pressure for the design basis loss of coolant accident, P<sub>a</sub>, is 44 psig.

The maximum allowable containment leakage rate,  $L_a$ , is 0.5% of containment air weight per day at  $P_a$ .

## CONTAINMENT LEAKAGE RATE TESTING PROGRAM (Continued)

Leakage rate acceptance criteria are:

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- a. Overall containment leakage rate acceptance criteria is ≤ 1.0 L. During the first unit startup following each test performed in accordance with this program, the overall containment leakage rate acceptance criteria are ≤ 0.60 L, for the Type B and Type C tests and ≤ 0.75 L, for Type A tests.
- b. Air lock acceptance criteria are:
  - Overall air lock leakage rate is ≤ 0.05 L, when tested at ≥ P.
  - Leakage rate for each door seal is ≤ 0.005 L, when pressurized to ≥ 10 psig.
- c. Secondary containment bypass leakage rate acceptance criteria is ≤ 0.06 L, when tested at ≥ P<sub>a</sub>.
- d. Containment purge valves with resilient seals acceptance criteria is ≤ 0.06 L, when tested at ≥ P<sub>a</sub>.

The provisions of Specification 4.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program.

The provisions of Specification 4.0.3 are applicable to the Containment Leakage Rate Testing Program. ۱

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PROPOSED MARKED-UP SPECIFICATION

ATTACHMENT B

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### HIGH RADIATION AREA (Continued)

- A radiation monitoring device which continuously integrates the radiation dose rate in b. the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.
- c. A health physics gualified individual (i.e., gualified in radiation protection procedures) with a radiation dose rate monitoring device who is responsible for providing positive control over the activities within the area and shall perform periodic radiation surveillance at the frequency specified by the facility Radiation Protection Superintendent-Nuclear in the RWP.

6.12.2 In addition to the requirements of Specification 6.12.1, areas accessible to personnel with radiation levels such that a major portion of the body could receive in one hour a dose greater than 1000 mrems\* but less than 500 rads\*\* shall be provided with locked doors to prevent unauthorized entry, and the keys shall be maintained under the administrative control of the Shift Superintendent on duty and/or health physics supervision/designee. Doors shall remain locked except during periods of access by personnel under an approved RWP which shall specify the dose rate levels in the immediate work area and the maximum allowable stay time for individuals in that area. For individual areas accessible to personnel with radiation levels such that a major portion of the body could receive in 1 hour a dose in excess of 1000 mrems\* but less than 500 rads\*\* that are located within large areas, such as PWR containment, where no enclosure exists for purposes of locking, and no enclosure can be reasonably constructed around the individual areas, then that area shall be roped off, conspicuously posted and a flashing light shall be activated as a warning device. In lieu of the stay time specification of the RWP, direct or remote (such as use of closed circuit TV cameras) continuous surveillance may be made by personnel qualified in radiation protection procedures to provide positive exposure control over the activities within the area.

### 6.13 PROCESS CONTROL PROGRAM (PCP)

6.13.1 The PCP shall be approved by the Commission prior to implementation.

6.13.2 Licensee-initiated changes to the PCP:

Shall be documented and records of reviews performed shall be retained as required a. by Specification 6.10.3p. This documentation shall contain:

> Sufficient information to support the change together with the appropriate analyses or evaluation justifying the change(s) and

"Measurement made at 30 centimeters from the radiation source or from any surface that the radiation penetrates.

\*\*Measurement made at 1 meter from the radiation source or from any surface that the radiation penetrates.

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the Quality Assurance Program Manual AMENDMENT NO. 68,116,146

## PROCESS CONTROL PROGRAM (Continued)

- A determination that the change will maintain the overall conformance of the solidified waste product to existing requirements of Federal, State, or other applicable regulations.
- b. Shall become effective after review and acceptance by the PORC and the approval of the General Manager Plant Operations.

## 6.14 OFFSITE DOSE CALCULATION MANUAL (ODCM)

- 6.14.1 The ODCM shall be approved by the Commission prior to implementation.
- 6.14.2 Licensee-initiated changes to the ODCM:
  - a. Shall be documented and records of reviews performed shall be retained as required by Specification 6.10.3p. This document shall contain:

1. the Quality I Assurance Program Manual 2.

- Sufficient information to support the change together with the appropriate analyses or evaluations justifying the change(s) and
- A determination that the change will maintain the level of radioactive effluent control required pursuant to 10 CFR 20.1302, 40 CFR Part 190, 10 CFR 50.36a, and Appendix I to 10 CFR Part 50 and not adversely impact the accuracy or reliability of effluent, dose or setpoint calculations.
- Shall become effective after review and acceptance by the PORC and the approval of the General Manager Plant Operations.
- c. Shall be submitted to the Commission in the form of a complete, legible copy of the entire ODCM as a part of or concurrent with the Annual Radioactive Effluent Release Report for the period of the report in which any change to the ODCM was made. Each change shall be identified by markings in the margin of the affected pages, clearly indicating the area of the page that was changed, and shall indicate the date (e.g., month/year) the change was implemented.

## 6.15 CONTAINMENT LEAKAGE RATE TESTING PROGRAM

A program shall be established to implement the leakage rate testing of the containment as required by 10 CFR 50.54(o) and 10 CFR 50, Appendix J, Option B, as modified by approved exemptions. This program shall be in accordance with the guidelines contained in Regulatory Guide 1.163, "Performance-Based Containment Leak-Test Program," dated September 1995.

The peak calculated containment internal pressure for the design basis loss of coolant accident, P<sub>a</sub>, is 44 psig.

The maximum allowable containment leakage rate, L<sub>a</sub>, is 0.5% of containment air weight per day at P<sub>a</sub>.

6-24

### CONTAINMENT LEAKAGE RATE TESTING PROGRAM (Continued)

Leakage rate acceptance criteria are:

44.

- a. Overall containment leakage rate acceptance criteria is ≤ 1.0 L<sub>o</sub>. During the first unit startup following each test performed in accordance with this program, the overall containment leakage rate acceptance criteria are ≤ 0.60 L<sub>o</sub> for the Type B and Type C tests and ≤ 0.75 L<sub>o</sub> for Type A tests.
- b. Air lock acceptance criteria are:
  - Overall air lock leakage rate is ≤ 0.05 L, when tested at ≥ P.
  - Leakage rate for each door seal is ≤ 0.005 L, when pressurized to ≥ 10 psig.
- c. Secondary containment bypass leakage rate acceptance criteria is ≤ 0.06 L, when tested at ≥ P<sub>a</sub>.
- d. Containment purge valves with resilient seals acceptance criteria is ≤ 0.06 L, when tested at ≥ P<sub>a</sub>.

The provisions of Specification 4.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program.

The provisions of Specification 4.0.3 are applicable to the Containment Leakage Rate Testing Program.



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### 6.16 (RESERVED FOR CONFIGURATION RISK MANAGEMENT PROGRAM)

# INSERT 2 ON NEXT PAGE (6-26)

#### 6.17 TECHNICAL SPECIFICATIONS BASES CONTROL PROGRAM

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This program provides a means for processing changes to the Bases of these Technical Specifications.

- a. Changes to the Bases of the Technical Specifications shall be made under appropriate administrative controls and reviews.
- Licensees may make changes to Bases without prior NRC approval provided the changes do not involve either of the following:
  - 1. A change in the Technical Specifications incorporated in the license; or
  - A change to the UFSAR or Bases that involves an unreviewed safety question as defined in 10 CFR 50.59.
- c. The Bases Control Program shall contain provisions to ensure that the Bases are maintained consistent with the UFSAR.
- Proposed changes to the Technical Specifications incorporated in the license or proposed changes to the UFSAR or Bases that involve an unreviewed safety question as defined in 10CFR 50.59 shall be reviewed and approved by the NRC prior to implementation.
  Implemented changes to the Bases not requiring prior NRC approval shall be provided to the NRC on a frequency consistent with 10 CFR 50.71(e).

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# ATTACHMENT C

PROPOSED SPECIFICATION

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6.17 TECHNICAL SPECIFICATIONS BASES CONTROL PROGRAM	6-26

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## HIGH RADIATION AREA (Continued)

- b. A radiation monitoring device which continuously integrates the radiation dose rate in the area and alarms when a preset integrated dose is received. Entry into such areas with this monitoring device may be made after the dose rate level in the area has been established and personnel have been made knowledgeable of them.
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<sup>\*</sup>Measurement made at 30 centimeters from the radiation source or from any surface that the radiation penetrates.

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## PROCESS CONTROL PROGRAM (Continued)

- 1. Sufficient information to support the change together with the appropriate analyses or evaluation justifying the change(s) and
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## CONTAINMENT LEAKAGE RATE TESTING PROGRAM (Continued)

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- b. Air lock acceptance criteria are:
  - 1. Overall air lock leakage rate is ≤ 0.05 L<sub>a</sub> when tested at ≥ P<sub>a</sub>.
  - 2. Leakage rate for each door seal is  $\leq 0.005 L_a$  when pressurized to  $\geq 10$  psig.
- c. Secondary containment bypass leakage rate acceptance criteria is  $\leq 0.06 L_a$  when tested at  $\geq P_a$ .
- d. Containment purge valves with resilient seals acceptance criteria is  $\leq 0.06 L_a$  when tested at  $\geq P_a$ .

The provisions of Specification 4.0.2 do not apply to the test frequencies specified in the Containment Leakage Rate Testing Program.

The provisions of Specification 4.0.3 are applicable to the Containment Leakage Rate Testing Program.

6.16 (RESERVED FOR CONFIGURATION RISK MANAGEMENT PROGRAM)

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- c. The Bases Control Program shall contain provisions to ensure that the Sases are maintained consistent with the UFSAR.
- d. Proposed changes to the Technical Specifications incorporated in the license or proposed changes to the UFSAR or Bases that involve an unreviewed safety question as defined in 10 CFR 50.59 shall be reviewed and a proved by the NRC prior to implementation. Implemented changes to the Bases not requiring prior NRC approval shall be provided to the NRC on a frequency consistent with 10 CFR 50.71(e).

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## ATTACHMENT D

## COMMITMENT IDENTIFICATION/VOLUNTARY ENHANCEMENT FORM

## Attachment D to W3F1-99-0111 COMMITMENT IDENTIFICATION/VOLUNTARY ENHANCEMENT FORM

Subject: Technical Specification Change Request NPF-38-216, Administrative Controls Section 6.0 Technical Specification Changes Date: July 13, 1999

COMMITMENT(S)	ONE- TIME ACTION	CONTINUING COMPLIANCE	SCHEDULED COMPLETION DATE (IF REQUIRED)	ASSOCIATED CR OR ER
Establish Waterford 3 administrative controls to process TS Bases changes in accordance with requirements delineated in TSCR NPF-38-216.	N/A	YES	Commitment 25661 Prior to TS Change Implementation	TSCR NPF-38-216

\*Check one only

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VOLUNTARY ENHANCEMENT(S)	ASSOCIATED CR OR ER