

PUBLIC SERVICE COMPANY OF COLORADO  
FORT ST. VRAIN NUCLEAR GENERATING STATION

MONTHLY OPERATIONS REPORT

NO. 151

August, 1986

FORM 286 22 0218

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This report contains the highlights of the Fort St. Vrain, Unit No. 1, activities operated under the provisions of the Nuclear Regulatory Commission Operating License DPR-34. This report is for the month of August, 1986.

1.0 NARRATIVE SUMMARY OF OPERATING EXPERIENCE AND MAJOR SAFETY RELATED MAINTENANCE

The reactor remained shutdown during the entire month of August for Environmental Qualification modifications.

On August 5, 1986, a meeting was held between Public Service Company and Region IV to discuss the Systematic Assessment of Licensee Performance (SALP) Report for the period March 1, 1985 through May 6, 1986.

On August 5-6, 1986, the annual Fort St. Vrain Radiological Emergency Response Plan exercise was conducted.

Three items were identified requiring reports to the Nuclear Regulatory Commission. These are: LER 86-023, Radiation Monitors Found Out of Calibration; LER 86-024, PCRV Rupture Disc Found Out of Tolerance; and LER 86-025, Reactor Building Sump Release Not Continuously Sampled.

Hot Service Facility decontamination has been completed and preparations for reflector block shipping are in progress.

Security improvements appear to be on schedule with the exception of one area which is expected to require approximately three months in addition to the original schedule.

On August 24, 1986, primary coolant circulation was switched from Loop I to Loop II. Environmental Qualification modifications are now proceeding on Loop I.

2.0 SINGLE RELEASES OF RADIOACTIVITY OR RADIATION EXPOSURE IN EXCESS OF 10% OF THE ALLOWABLE ANNUAL VALUE

None

3.0 INDICATION OF FAILED FUEL RESULTING FROM IRRADIATED FUEL EXAMINATIONS

None

4.0 MONTHLY OPERATING DATA REPORT

Attached

OPERATING DATA REPORT

DOCKET NO. 50-267

DATE September 9, 1986

COMPLETED BY F. J. Novachek

TELEPHONE (303) 620-1007

OPERATING STATUS

1. Unit Name: Fort St. Vrain Unit No. 1
2. Reporting Period: 860801 through 860831
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 342
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:  
None

NOTES

9. Power Level To Which Restricted, If Any (Net MWe): 0.0
10. Reasons for Restrictions, If Any: Per commitment to the Nuclear Regulatory Commission remain shutdown pending completion of Environmental Qualification modifications.

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	<u>744.0</u>	<u>5,831.0</u>	<u>62,856.0</u>
12. Number of Hours Reactor Was Critical	<u>0.0</u>	<u>3,456.6</u>	<u>30,537.8</u>
13. Reactor Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
14. Hours Generator On-Line	<u>0.0</u>	<u>1,087.1</u>	<u>19,555.1</u>
15. Unit Reserve Shutdown Hours	<u>0.0</u>	<u>0.0</u>	<u>0.0</u>
16. Gross Thermal Energy Generated (MWE)	<u>0.0</u>	<u>370,316.6</u>	<u>10,255,399.8</u>
17. Gross Electrical Energy Generated (MWE)	<u>0.0</u>	<u>85,402.0</u>	<u>3,333,996.0</u>
18. Net Electrical Energy Generated (MWE)	<u>-2,248.0</u>	<u>58,926.0</u>	<u>2,955,199.0</u>
19. Unit Service Factor	<u>0.0</u>	<u>18.6</u>	<u>31.1</u>
20. Unit Availability Factor	<u>0.0</u>	<u>18.6</u>	<u>31.1</u>
21. Unit Capacity Factor (Using MDC Net)	<u>0.0</u>	<u>3.1</u>	<u>14.2</u>
22. Unit Capacity Factor (Using DER Net)	<u>0.0</u>	<u>3.1</u>	<u>14.2</u>
23. Unit Forced Outage Rate	<u>100.0</u>	<u>81.4</u>	<u>60.4</u>

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Environmental Qualification modifications, 860901, 1160 hours

25. If Shut Down at End of Report Period, Estimated Date of Startup: December 31, 1986

26. Units In Test Status (Prior to Commercial Operation):

	Forecast	Achieved
INITIAL CRITICALITY	<u>N/A</u>	<u>N/A</u>
INITIAL ELECTRICITY	<u>N/A</u>	<u>N/A</u>
COMMERCIAL OPERATION	<u>N/A</u>	<u>N/A</u>

# AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-267  
Unit Fort St. Vrain Unit No. 1  
Date September 9, 1986  
Completed By F. J. Novachek  
Telephone (303) 620-1007

Month August

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

1	<u>0.0</u>
2	<u>0.0</u>
3	<u>0.0</u>
4	<u>0.0</u>
5	<u>0.0</u>
6	<u>0.0</u>
7	<u>0.0</u>
8	<u>0.0</u>
9	<u>0.0</u>
10	<u>0.0</u>
11	<u>0.0</u>
12	<u>0.0</u>
13	<u>0.0</u>
14	<u>0.0</u>
15	<u>0.0</u>
16	<u>0.0</u>

DAY AVERAGE DAILY POWER LEVEL  
(MWe-Net)

17	<u>0.0</u>
18	<u>0.0</u>
19	<u>0.0</u>
20	<u>0.0</u>
21	<u>0.0</u>
22	<u>0.0</u>
23	<u>0.0</u>
24	<u>0.0</u>
25	<u>0.0</u>
26	<u>0.0</u>
27	<u>0.0</u>
28	<u>0.0</u>
29	<u>0.0</u>
30	<u>0.0</u>
31	<u>0.0</u>

\* Generator on line but no net generation.

TELEPHONE (303) 620-1007

REPORT MONTH August, 1986

NO.	DATE	TYPE	DURATION	REASON	METHOD OF SHUTTING DOWN REACTOR	LER #	SYSTEM CODE	COMPONENT CODE	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURRENCE
86-03	860801	F	744 hr	D	1	N/A	ZZ	ZZZZZZ	Environmental Qualification modifications

# REFUELING INFORMATION

1. Name of Facility	Fort St. Vrain Unit No. 1
2. Scheduled date for next refueling shutdown.	March 1, 1988
3. Scheduled date for restart following refueling.	June 1, 1988
4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?	No
If answer is yes, what, in general, will these be?	-----
If answer is no, has the reload fuel design and core configuration been reviewed by your Plant Safety Review Committee to determine whether any unreviewed safety questions are associated with the core reload (Reference 10 CFR Section 50.59)?	No
If no such review has taken place, when is it scheduled?	1986
5. Scheduled date(s) for submitting proposed licensing action and supporting information.	-----
6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.	-----
7. The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.	a) 1482 HTGR fuel elements b) 0 spent fuel elements



REFUELING INFORMATION (CONTINUED)

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.	Capacity is limited in size to about one-third of core (approximately 500 HTGR elements). No change is planned.
9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity.	1992 under Agreements AT(04-3)-633 and DE-SC07-79ID01370 between Public Service Company of Colorado, and General Atomic Company, and DOE.*

- \* The 1992 estimated date is based on the understanding that spent fuel discharged during the term of the Agreements will be stored by DOE at the Idaho Chemical Processing Plant. The storage capacity has evidently been sized to accommodate eight fuel segments. It is estimated that the eighth fuel segment will be discharged in 1992.



Public Service

Public Service  
Company of Colorado

16805 WCR 19 1/2, Platteville, Colorado 80651

September 15, 1986  
Fort St. Vrain  
Unit No. 1  
P-86551

Office of Inspection and Enforcement  
ATTN: Document Control Desk  
U. S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Docket No. 50-267

SUBJECT: MONTHLY OPERATIONS  
REPORT - AUGUST, 1986

REFERENCE: Facility Operating  
License No. DPR-34

Dear Sir:

Enclosed, please find the Monthly Operations Report for the month of August, 1986.

If you have any questions, please contact Mr. M. H. Holmes at (303) 480-6960.

Sincerely,

J. W. Gahm  
Manager, Nuclear Production  
Fort St. Vrain Nuclear  
Generating Station

Enclosure

cc: Mr. R. D. Martin,  
Regional Administrator, Region IV

JWG:djm

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