



THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

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MURRAY R. EDELMAN
SR. VICE PRESIDENT
NUCLEAR

May 1, 1987
PY-CEI/NRR-0644 L

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D. C. 20555

Perry Nuclear Power Plant
Docket No. 50-440
Supplement Information
to Letter PY-CEI/NRR-0631 L

Gentlemen:

By a previous letter dated April 7, 1987 (PY-CEI/NRR-0631 L), CEI committed to inform the NRC staff of the results of post modification testing of reactor vessel level instrumentation reference leg modification implemented to correct reactor vessel level anomalies during Reactor Core Isolation Cooling (RCIC) injection.

During an outage from March 25 to April 11, 1987 modifications were made to the A, C, and D reference legs. The A reference leg tap was modified with the installation of an instrument tap insert which prevented the film flow of water from blocking the tap. The D reference leg slope was adjusted and more insulation added to the steam portion of the reference leg. A combination of the two modifications was done to the C reference leg/tap.

On April 12 and 13 special RCIC testing was performed as committed to in our previous letter dated March 31, 1987 (PY-CEI/NRR-0624 L). A review of the test data indicated the following:

- 1) The A, B, and C reference legs instrumentation was unaffected over the entire range of testing.
- 2) The D reference leg instrumentation was effected at every test pressure.

Based upon the test exceptance criteria provided in Amendment 4 to NPF-58, these test results allowed continued operation until May 31, 1987. CEI additionally concluded that the instrument tap insert modification was successful, and that the slope modification was not.

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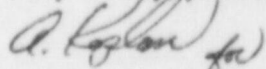
Consequently, the tap insert modification was implemented on both the B and D reference leg taps during an outage from April 13 to April 26, 1987. Post modification testing was performed on April 27 and 28 at 250 psig and 950 psig, with no indication of level anomalies. The test data was reviewed by the CEI Nuclear Engineering Department on April 29 and 30, and the determination made that no level anomalies were encountered on any reference leg instrumentation.

Additionally a hot quick start of the RCIC system to rated system flow was performed on April 30, 1987 as part of the Startup Test program with the reactor operating at approximately 950 psig and 8% rated thermal power. Again, no level anomalies were experienced.

Based on the modification and testing performed, CEI has concluded that the level anomalies experienced during previous RCIC injection has been corrected. Due to this, the RCIC system will be restored to a fully OPERATIONAL condition including automatic opening of the RCIC injection valve. Since the footnote added to Technical Specification 3/4.7.3 in Amendment 4 is no longer applicable, CEI will request removing it in a future amendment.

If you have any questions, please feel free to call.

Very truly yours,



Murray R. Edelman
Senior Vice President
Nuclear Group

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