UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

July 6, 1999

Mr. Charles H. Cruse
Vice President, Nuclear Energy
Baltimore Gas & Electric Company
Calvert Cliffs Nuclear Power Plant
1650 Calvert Cliffs Parkway
Lusby, MD 20657

SUBJECT: CLOSURE OF TAC NOS. MA0532 AND MA0533 - RESPONSE TO THE REQUESTS FOR ADDITIONAL INFORMATION TO GENERIC LETTER 92-01, REVISION 1, SUPPLEMENT 1, "REACTOR VESSEL STRUCTURAL INTEGRITY," FOR CALVERT CLIFFS NUCLEAR POWER PLANT, UNIT NOS. 1 AND 2

Dear Mr. Cruse:

On May 19, 1995, the U.S. Nuclear Regulatory Commission (NRC) issued Generic Letter 92-01, Revision 1, Supplement 1 (GL 92-01, Rev. 1, Supp. 1), "Reactor Vessel Structural Integrity," to holders of nuclear operating licenses. In issuing the GL, the staff required addressees of the GL to:

(1) identify, collect and report any new data pertinent to the analysis of structural integrity of the reactor pressure vessels (RPVs) at their nuclear plants, and

(2) to assess the impact of that data on their RPV integrity analyses relative to the requirements of Sections 50.60 and 50.61 to Part 50 of Title 10 of the *Code of Federal Regulations* (10 CFR 50.60 and 10 CFR 50.61), and to the requirements of Appendices G and H to Part 50 of Title 10 of the *Code of Federal Regulations* (Appendices G and H to 10 CFR Part 50).

On July 24, 1995, you submitted your initial response to GL 92-01, Rev. 1, Supp. 1. and provided the requested information relative to the structural integrity assessments for Calvert Cliffs Nuclear Power Plants. The staff evaluated your response to GL 92-01, Rev. 1, Supp. 1, and provided its conclusion relative to your response on August 1, 1996. However, since the time of the staff's closure letter, the Combustion Engineering (CE) Owners Group and the Babcock and Wilcox (B&W) Owners Group have each submitted additional data regarding the alloying chemistries of beltline welds in CE and B&W fabricated vessels. The additional alloying data were submitted in Topical Reports CE NPSD-1039, Revision 2, CE NPSD-1119, Revision 1, for CE fabricated RPV welds, and BAW-2325, Revision 1, for B&W fabricated RPV welds. In addition, Chicago Bridge and Iron (CB&I) BWR data were submitted in Topical Report BWRVIP-46. As a result of the efforts by CE and B&W, the staff determined that additional information was necessary relative to the structural integrity assessments for your plants. On April 3, 1998, the staff issued a request for additional information (RAI) in regard to the alloying chemistries of beltline welds, your assessment of surveillance data for your facility, pressure-temperature (P-T) limits, and pressurized thermal shock (PTS) assessments for the Calvert Cliffs Nuclear Power Plants. In general, with respect to the contents of the RAI, the staff requested that you reassess the alloying chemistries for the beltline welds and RPV surveillance welds relative to the chemistries provided in the applicable topical report, and

provide the impact of any changes to the best-estimate chemistries for your beltline RPV welds on the structural integrity assessments for your facility relative to the requirements of 10 CFR 50.60, 10 CFR 50.61, and Appendices G and H to 10 CFR Part 50, as applicable to the licensing bases for your plants.

You provided your response to the staff's RAI for Calvert Cliffs Nuclear Plants on July 1, 1998. As a result of the staff's review of your responses to GL 92-01, Revision 1, GL 92-01, Rev. 1, Supp. 1, and the Supp. 1 RAI, the staff has revised the information in the Reactor Vessel Integrity Database (RVID) and is releasing it as RVID Version 2.

The new database diskettes are posted on the world-wide-web at a location which is linked to the NRC home page (http://www.nrc.gov/NRR/RVID/index.html). We recommend that you review this information. If the staff does not receive comments by September 1, 1999, we will assume that the data entered into the RVID are acceptable for your plant. No additional information is necessary with regard to the structural integrity assessments. Future submittals on P-T limits, PTS, or upper shelf energy (USE) should reference the most current information.

This closes the staff's efforts in regard to TAC Nos. MA0532 and MA0533. The staff appreciates your efforts in regard to this matter.

Sincerely,

original signed by:
Alexander W. Dromerick , Sr. Project Manager, Section 1
Project Directorate I
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket Nos. 50-317 and 50-318

cc: See next page

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Sincerely,

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