



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20565-0001

July 8, 1999

Mr. Garry L. Randolph
Vice President and Chief Nuclear Officer
Union Electric Company
Post Office Box 620
Fulton, MO 65251

SUBJECT: GENERIC LETTER 92-01, REVISION 1, SUPPLEMENT 1, "REACTOR VESSEL STRUCTURAL INTEGRITY," FOR CALLAWAY PLANT, UNIT 1 (TAC NO. MA0531)

Dear Mr. Randolph:

On May 19, 1995, the U.S. Nuclear Regulatory Commission (NRC) issued Generic Letter 92-01, Revision 1, Supplement 1 (GL 92-01, Rev. 1, Supp. 1), "Reactor Vessel Structural Integrity," to holders of nuclear operating licenses. In issuing the GL the staff required addressees of the GL to:

- (1) identify, collect and report any new data pertinent to the analysis of structural integrity of the reactor pressure vessels (RPVs) at their nuclear plants, and
- (2) to assess the impact of that data on their RPV integrity analyses relative to the requirements of Sections 50.60 and 50.61, and Appendices G and H to 10 CFR Part 50.

You submitted your initial response to the GL on August 11, 1995, and provided the requested information relative to the structural integrity assessment for Callaway Plant, Unit 1 (Callaway). The staff evaluated your response to the GL and provided its conclusions in a letter on August 2, 1996. However, since the time of the staff's closure letter, the Combustion Engineering (CE) Owners Group and the Babcock and Wilcox (B&W) Owners Group have each submitted additional data regarding the alloying chemistries of beltline welds in CE and B&W fabricated vessels. The additional alloying data were submitted in Topical Reports CE NPSD-1039, Revision 2, CE NPSD-1119, Revision 1, for CE fabricated RPV welds, and BAW-2325, Revision 1, for B&W fabricated RPV welds. In addition, Chicago Bridge and Iron (CB&I) BWR data were submitted in Topical Report BWRVIP-46. As a result of the efforts by CE and B&W, the staff determined that additional information was necessary relative to the structural integrity assessments for your plant.

On April 9, 1998, the staff issued a request for additional information (RAI) in regard to the alloying chemistries of beltline welds, your assessment of surveillance data for your facility, pressure-temperature (P-T) limits, and pressurized thermal shock (PTS) assessment (only applicable to PWRs) for Callaway. In general, with respect to the contents of the RAI, the staff requested that you reassess the alloying chemistries for the beltline welds and RPV surveillance welds relative to the chemistries provided in the applicable topical report, and provide the impact of any changes to the best-estimate chemistries for your beltline RPV welds on the structural integrity assessments for Callaway relative to the requirements of Sections 50.60 and 50.61, and Appendices G and H to 10 CFR Part 50, as applicable to the licensing basis for Callaway.

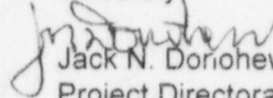
July 8, 1999

You provided your response to the staff's RAI on July 10, 1998. As a result of the staff's review of your responses to GL 92-01, Revision 1, GL 92-01, Rev. 1, Supp. 1, and the Supp. 1 RAI, the staff has revised the information in the Reactor Vessel Integrity Database (RVID) and is releasing it as RVID Version 2.

The new database diskettes are posted on the world-wide-web at a location which is linked to the NRC home page (<http://www.nrc.gov/NRR/RVID/index.html>). We recommend that you review this information. If the staff does not receive comments by September 1, 1999, we will assume that the data entered into the RVID are acceptable for your plant. No additional information is necessary with regard to the structural integrity assessments. Future submittals on P-T limits, PTS (only applicable to PWRs), or upper shelf energy (USE) should reference the most current information.

You have adequately responded to the GL for Callaway and this letter closes the staff's efforts in regard to TAC No. MA0531. The staff appreciates your efforts in regard to this matter.

Sincerely,



Jack N. Donohew, Project Manager, Section 2
Project Directorate IV & Decommissioning
Division of Licensing Project Management
Office of Nuclear Reactor Regulation

Docket No. 50-483

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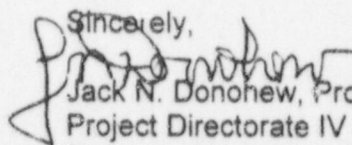
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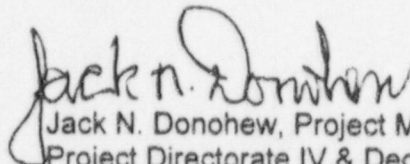
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