UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

PORTLAND GENERAL ELECTRIC COMPANY

THE CITY OF EUGENE, OREGON

PACIFIC POWER AND LIGHT COMPANY

DOCKET NO. 50-344

TROJAN NUCLEAR PLANT

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 130
License No. NPF-1

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment by Portland General Electric Company, et al., (the licensee) dated October 29, 1985, as revised January 9, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

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ATTACHMENT TO LICENSE AMENDMENT NO. 130

TO FACILITY OPERATING LICENSE NO. NPF-1

DOCKET NO. 50-344

Revise Appendix A as follows:

Remove Pages	Insert Pages
TS 3/4 7-8	TS 3/4 7-8
B 3/4 7-2	B 3/4 7-2

2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. NPF-1 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 130, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications, except where otherwise stated in specific license conditions.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

George W. Knighton

George W. Knighton, Director
Project Directorate V
Division of Reactor Projects-III/IV/V and
Special Projects

Attachment:
Changes to the Technical
Specifications

Date of Issuance: April 22, 1987

PLANT SYSTEMS

CONDENSATE STORAGE TANK

LIMITING CONDITION FOR OPERATION

3.7.1.4 The condensate storage tank shall be OPERABLE with a minimum contained volume of 239,000 gallons of water.

APPLICABILITY: MODES 1, 2 and 3.

ACTION:

With the condensate storage tank inoperable, within 4 hours either:

a. Restore the condensate storage tank to OPERABLE status or be in HOT SHUTDOWN within the next 12 hours, or

b. Demonstrate the OPERABILITY of the Service Water System as a backup supply to the auxiliary feedwater pumps and restore the condensate storage tank to OPERABLE status within 7 days or be in HOT SHUTDOWN within the next 12 hours.

SURVEILLANCE REQUIREMENTS

4.7.1.4.1 The condensate storage tank shall be demonstrated OPERABLE at least once per 12 hours by verifying the water level is within its limit when the tank is the supply source for the auxiliary feedwater pumps.

4.7.1.4.2 The Service Water System shall be demonstrated OPERABLE at least once per 12 hours by verifying that at least one Service Water System train is operating and that the Service Water System-feedwater isolation valves are either open or OPERABLE whenever the Service Water System is the supply source for the auxiliary feedwater pumps.

U = Maximum number of inoperable safety valves per operating steam line

109 = Power Range Neutron Flux-High Trip Setpoint for 4 loop operation

75 = Maximum percent of RATED THERMAL POWER permissible by P-8 Setpoint for 3 loop operation

X = Total relieving capacity of all safety valves per steam line in lbs/hour (4.12×10^6 lbs/hr)

Y = Maximum relieving capacity of any one safety valve in lbs/hour (9.27×10^5 lbs/hr)

3/4.7.1.2 AUXILIARY FEEDWATER SYSTEM

The OPERABILITY of the auxiliary feedwater systems ensures that the Reactor Coolant System can be cooled down to less than 350°F from normal operating conditions in the event of a total loss of off-site power.

Either of the two pumps has the required capacity to provide sufficient feedwater flow to remove reactor decay heat and reduce the RCS temperature to 350°F where the Residual Heat Removal System may be placed into operation for continued cooldown.

The electric auxiliary feedwater pump is not safety grade and is used during startup and shutdown to preclude unnecessary wear on the safety-related pumps. The electric auxiliary feedwater pump has approximately the same rated capacity as either of the safety-related pumps.

With no auxiliary feedwater pumps OPERABLE, power is reduced to 35 percent, because at this power level the feedwater regulating valves should be in AUTO and the plant in a stable low power mode which does not require auxiliary feedwater.

3/4.7.1.3 (deleted)

3/4.7.1.4 CONDENSATE STORAGE TANK

The OPERABILITY of the condensate storage tank with the minimum water volume ensures that sufficient water is available to maintain the RCS at HOT STANDBY conditions for 2 hours with steam discharge at atmosphere concurrent with total loss of off-site power. The minimum contained volume accounts for: (1) water volume required to be delivered to the steam generators (196,000 gallons), (2) unusable volume in the bottom of the tank (27,700 gallons), and (3) instrument error (14,400 gallons), rounded off to 239,000 gallons.