

U.S. NUCLEAR REGULATORY COMMISSION  
REGION I

Report No. 50-247/86-136

Docket No. 50-247

License No. DPR-26

Licensee: Consolidated Edison Company of New York, Inc.  
Broadway and Bleakley Avenue  
Buchanan, New York 10511

Facility Name: Indian Point Nuclear Generating Station, Unit 2

Meeting At: NRC Region I, King of Prussia, Pennsylvania

Meeting Conducted: December 9, 1986

Approved by: Jon R. Johnson  
J. Johnson, Chief, Operational  
Programs Section, OB, DRS

1/21/87  
date

Meeting Summary:

A meeting was held at the licensee's request (attendees are listed in Attachment A) on December 9, 1986 to discuss their plans for an Integrated Assessment of the Indian Point Unit 2 Auxiliary Feedwater (AFW) System. The subjects discussed are described in the attached letter from Consolidated Edison Company to the NRC dated November 13, 1986 (Attachment B). The licensee provided the results of their initial assessment and concluded that the present AFW system design was: 1) in accordance with Chapters 10 and 14 of their Final Safety Analysis Report, and 2) that this design was acceptable to perform the intended function. The licensee is presently having this analysis confirmed and several calculations independently checked.

The licensee has contacted manufacturers' representatives to evaluate the reliability of installed equipment (such as the overspeed trip linkage) and to determine whether it should be upgraded. Modifications are being considered such as providing direct indication in the control room of overspeed trip mechanism status. Evaluations are also being made of the discharge capacity and orifice size of the relief valve in the turbine steam supply line.

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The licensee stated that a system functional analysis is also in progress. This study includes an evaluation of the minimum flow requirements in order to determine whether one pump can satisfy the entire system needs of removing decay heat.

The licensee stated that the results of these long term actions of system reliability and safety function evaluations would be used to maintain system availability and update the Final Safety Analysis Report and the Technical Specifications if warranted.

The meeting ended with a commitment from the licensee to provide the NRC with their plans for a preventive maintenance program for verifying the proper condition of electrical connectors in the station.

ATTACHMENT A

DECEMBER 9, 1986 MEETING WITH  
CONSOLIDATED EDISON COMPANY (IP2) AND NRC CONCERNING  
THE AUXILIARY FEEDWATER SYSTEM

<u>Name</u>	<u>Organization</u>	<u>Title</u>
J. Johnson	NRC/RI	Chief, Operational Programs
S. Ebnetter	NRC/RI	Director, Division of Reactor Safety
W. Johnston	NRC/RI	Deputy Director, Division of Reactor Safety
P. Eselgroth	NRC/RI	Chief, Division of Reactor Projects, Branch 2
M. C. Lee	Consolidated Edison	General Manager, Technical Support
P. Destefano	Consolidated Edison	Systems Engineer
T. Wong	Consolidated Edison	Electrical Engineering
J. Mooney	Consolidated Edison	Systems Engineer
D. J. Holody	NRC/RI	Enforcement Specialist
J. R. Ellwanger	Consolidated Edison	Senior Licensing Engineer
L. Liberatori	Consolidated Edison	Manager, Safety Assessment
R. J. Giardina	NRC/NRR/DPLA/PSB	Mechanical Engineer
R. M. Gallo	NRC/RI	Chief, Projects Section 2A
L. Norrholm	NRC/RI	Chief, Reactor Projects Section 2B
M. Slosson	NRC	Licensing Project Manager
L. Rossbach	NRC/RI	Senior Resident Inspector
F. Paulitz	NRC/RI	Reactor Engineer
N. Blumberg	NRC/RI	Lead Reactor Engineer
H. Sager	Consolidated Edison	Manager, NPQA Engineering
P. Kelley	NRC/RI	Resident Inspector
P. Malik	Consolidated Edison	Principal Engineer



Consolidated Edison Company of New York, Inc.  
Indian Point Station  
Broadway & Bleakley Avenue  
Buchanan, NY 10511  
Telephone (914) 737-8116

November 13, 1986

U.S. Nuclear Regulatory Commission  
Attn: Mr. William F. Kane  
Director, Division of Reactor Projects  
Region 1  
631 Park Avenue  
King of Prussia, PA 19406

SUBJECT: Integrated Assessment of Indian Point Unit 2  
Auxiliary Feedwater System

Dear Mr. Kane:

As a result of the events associated with the Indian Point Unit 2 Auxiliary Feedwater System during the two reactor trips during the week of October 20, 1986, we have instituted a program with the objective of defining any changes that may be desirable in order to improve the system's function. The basic elements of the program will be as outlined by Consolidated Edison to the Region during a telephone discussion on Sunday, October 26, 1986 prior to resuming plant operation. It will consist of an evaluation of five principal areas:

1. System functional requirements
2. System design and modifications
3. Demonstrate adequacy of present equipment performance
4. Maintenance and operational history
5. Adequacy of surveillance

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It is currently envisioned that this assessment will be partly accomplished within Con Edison and also with an outside consultant experienced in such matters who can provide an independent view point. Initially, a short-term evaluation will be performed in-house, the status of which will be made known to the Region in a presentation the week of December 1, 1986. This preliminary review will focus on the functional criteria and the system design basis of the AFW system. In the long term, with the assistance of the outside consultant, a program will be developed to evaluate the status of the installed equipment and our methods for maintaining its continued qualified availability. The program will also evaluate the potential for system enhancement. A schedule for accomplishing this program together with a plan will form part of the presentation to the Region the week of December 1, 1986.

If you would like to discuss our course of action as outlined above or if you have any questions, please contact us.

Very truly yours,

*Murray Selman*

cc: Senior Resident Inspector  
P.O. Box 38  
Buchanan, NY 10511

Marylee Slosson  
Projects Manager  
Division of PWR Licensing - A  
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