UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

November 18, 1986

Docket Nos: 50-373/374,
50-387/388,
50-397/410,
and 50-341/354

LICENSEES: Commonwealth Edison Company
Pennsylvania Power and Light Company
Washington Public Power Supply System
Niagara Mohawk Power Corporation
Detroit Edison Company
Public Service Electric & Gas Company

FACILITIES: La Salle County Station, Units 1 and 2
Susquehanna Steam Electric Station, Units 1 and 2
WPPSS Nuclear Project No. 2
Nine Mile Point Nuclear Station, Unit 2
Fermi-2
Hope Creek Generating Station

SUBJECT: BWR PROJECT DIRECTORATE NO. 3 COUNTERPART MEETING
SEPTEMBER 9, 1986

On September 9, 1986, the NRC staff met with utility representatives with projects assigned to BWR Project Directorate No. 3 (BWD-3) to discuss relevant licensing issues. The purpose of this meeting was to improve the lines of communication between the NRC and the licensees.

Presentations on licensing issues were presented by both NRC staff and utility representatives. The topics of major interest included: the Sholly process, intrepretation of 10 CFR Part 50.59, the Technical Specification Improvement Program, and discretionary enforcement. Interest was also expressed regarding Mr. Sorensen's presentation on utility performance indicators.

The NRC staff was pleased with the willingness of the utilties to attend and participate in this type of a meeting and encouraged the utilities to partitate more aggressively in the development of agenda for future meetings. The NRC staff expressed the hope that this meeting was beneficial to all participants and that these types of meetings may continue.

Enclosure 1 contains the agenda; Enclosure 2 contains the meeting handouts; and Enclosure 3 contains the list of Attendees.

At the close of the meeting, the utility representatives were asked to provide comments either formally or through their project managers.

Elinor G. Adensam

Elinor G. Adensam, Director
BWR Project Directorate No. 3
Division of BWR Licensing

Enclosure:
As stated

cc: See next page

cc:
Mr. Harry H. Voigt, Esq.
LeBoeuf, Lamb, Leiby & MacRae
1333 New Hampshire Avenue, N. W.
Washington, D. C. 20036

John Flynn, Esq.
Senior Attorney
The Detroit Edison Company
2000 Second Avenue
Detroit, Michigan 48226

Mr. Dennis R. Hahn, Chief
Nuclear Facilities and Environmental
Monitoring Section Office
Division of Radiological Health
P. O. Box 30035
Lansing, Michigan 48909

Mr. Robert Woolley
Acting Supervisor-Licensing
The Detroit Edison Company
Fermi Unit 2
6400 No. Dixie Highway
Newport, Michigan 48166

Mr. Walt Rogers
U. S. Nuclear Regulatory Commission
Resident Inspector's Office
6450 W. Dixie Highway
Newport, Michigan 48166

Monroe County Office of Civil
Preparedness
963 South Raisinville
Monroe, Michigan 48161

Ronald C. Callen
Adv. Planning Review Section
Michigan Public Service Commission
6545 Mercantile Way
P. O. Box 30221
Lansing, Michigan 48909

Regional Administrator, Region III
U. S. Nuclear Regulatory Commission
799 Roosevelt Road
Glen Ellyn, Illinois 60137

cc:
Philip P. Steptoe, Esquire
Suite 4200
One First National Plaza
Chicago, Illinois 60603

John W. McCaffrey
Chief, Public Utilities Division
160 North La Salle Street, Room 900
Chicago, Illinois 60601

Assistant Attorney General
188 West Randolph Street
Suite 2315
Chicago, Illinois 60601

Resident Inspector/LaSalle, NPS
U.S. Nuclear Regulatory Commission
Rural Route No. 1
P.O. Box 224
Marseilles, Illinois 61341

Chairman
La Salle County Board of Supervisors
La Salle County Courthouse
Ottawa, Illinois 61350

Attorney General
500 South 2nd Street
Springfield, Illinois 62701

Chairman
Illinois Commerce Commission
Leland Building
527 East Capitol Avenue
Springfield, Illinois 62706

Mr. Gary N. Wright, Manager
Nuclear Facility Safety
Illinois Department of Nuclear Safety
1035 Outer Park Drive, 5th Floor
Springfield, Illinois 62704

Regional Administrator, Region III
U. S. Nuclear Regulatory Commission
799 Rossevelt Road
Glen Ellyn, Illinois 60137

cc:
Mr. Troy B. Conner, Jr., Esq.
Conner & Wetterhahn
Suite 1050
1747 Pennsylvania Avenue, N.W.
Washington, D.C. 20006

Richard Goldsmith
Syracuse University
College of Law
E. I. White Hall Campus
Syracuse, New York 12223

Ezra I. Bialik
Assistant Attorney General
Environmental Protection Bureau
New York State Department of Law
2 World Trade Center
New York, New York 10047

Resident Inspector
Nine Mile Point Nuclear Power Station
P. O. Box 99
Lycoming, New York 13093

Mr. John W. Keib, Esq.
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Mr. James Linville
U. S. Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

Norman Rademacher, Licensing
Niagara Mohawk Power Corporation
300 Erie Boulevard West
Syracuse, New York 13202

Don Hill
Niagara Mohawk Power Corporation
Suite 550
4520 East West Highway
Bethesda, Maryland 20814

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
king of Prussia, Pennsylvania 19406

Mr. Paul D. Eddy
New York State Public Serice
Commission
Nine Mile Point Nuclear Station -
Unit II
P.O. Box 63
Lycoming, New York 13093

Mr. Richard M. Kessel
Chair and Executive Director
State Consumer Protection Board
99 Washington Avenue
Albany, New York 12210

cc:
Jay Silberg, Esq.
Shaw, Pittman, Potts & Trowbridge
2300 N Street N.W.
Washington, D.C. 20037

Bryan A. Snapp, Esq.
Assistant Corporate Counsel
Pennsylvania Power & Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Mr. William E. Barberich
Manager-Nuclear Licensing
Pennsylvania Power & Light Company
2 North Ninth Street
Allentown, Pennsylvania 18101

Mr. R. Jacobs
Resident Inspector
P.O. Box 52
Shickshinny, Pennsylvania 18655

Mr. R. J. Benich
Services Project Manager
General Electric Company
1000 First Avenue
King of Prussia, Pennsylvania 19406

Mr. Thomas M. Gerusky, Director
Bureau of Radiation Protection
Resources
Commonwealth of Pennsylvania
P. O. Box 2063
Harrisburg, Pennsylvania 17120

Robert W. Alder, Esquire
Office of Attorney General
P.O. Box 2357
Harrisburg, Pennsylvania 17120

Mr. William Matson
Allegheny Elec. Coorperative, Inc.
212 Locust Street
P.O. Bxo 1266
Harrisburg, Pennsylvania 17108-1266

Mr. W. H. Hirst, Manager
Joint Generation
Projects Department
Atlantic Electric
P.O. Box 1500
1199 Black Horse Pike
Pleasantville, New Jersey 08232

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Mr. G. C. Sorensen, Manager
Washington Public Power Supply System

WPPSS Nuclear Project No. 2
(WNP-2)

cc:
Nicholas S. Reynolds, Esq.
Bishop, Liberman, Cook,
Purcell & Reynolds
1200 Seventeenth Street, N.W.
Washington, D.C. 20036

Regional Administrator, Region V
U.S. Nuclear Regulatory Commission
1450 Maria Lane, Suite 210
Walnut Creek, California 94596

Mr. G. E. Doupe, Esquire
Washington Public Power Supply System
P. O. Box 968
3000 George Washington Way
Richland, Washington 99532

Mr. Curtis Eschels, Chairman
Energy Facility Site Evaluation Council
Mail Stop PY-11
Olympia, Washington 98504

P. L. Powell, Licensing Manager
Washington Public Power Supply System
P. O. Box 968, MD 956B
Richland, Washington 99352

Mr. W. G. Conn
Burns and Roe, Incorporated
c/o Washington Public Power Supply
System
P. O. Box 968, MD 994E
Richland, Washington 99352

P. B. Glasscock, Director
Licensing and Assurance
Washington Public Power Supply System
P. O. Box 968, MD 280
Richland, Washington 99352

Mr. C. M. Powers
WNP-2 Plant Manager
Washington Public Power Supply System
P. O. Box MD 927M
Richland, Washington 99352

November 18, 1986

MEETING SUMMARY DISTRIBUTION

Docket No(s): 50-373/374/387/388/397/410/341/354
NRC PDR
Local PDR
BWD #3 r/f
J. Partlow
E. Adensam
Attorney, OGC
E. Jordan
B. Grimes
ACRS (10)
Project Manager Listed Below
E. Hylton

NRC PARTICIPANTS

E. Adensam
R. Bernero
E. Hylton
D. Wagner
J. Bradfute
R. C. Barr
M. Haughey
A. Bournia
M. Campagnone
M. D. Lynch

FOIA-86-855
A/4

bcc: Applicant & Service List

ENCLOSURE 1

COUNTERPART MEETING FOR

BWD-3 HELD ON SEPTEMBER 9, 1986

NAME	AFFILIATION
E. G. Adensam	NRR/DBL
R. Bernero*	NRR/DBL
E. Hylton	NRR/DBL
D. Wagner	NRR/DBL
B. Preston	PSE&G Co.
R. Beckwith	PSE&G Co.
L. A. Reiter	PSE&G Co.
John O. Bradfute	NRR/DBL
G. C. Sorensen	Washington Public Power Supply System
Pat Powell	Washington Public Power Supply System
Larry Aeschliman	Washington Public Power Supply System
R. C. Barr	NRC/WNP-2 Resident
Mary Haughey	NRR/DBL
A. F. Zallnick	NMPC
P. E. Francisco	NMPC
Tom Hammerick	Commonwealth Edison
Mike Tubbak	Commonwealth Edison
Anthony Bournia	NRR/DBL
C. M. Allen	Commonwealth Edison
Thomas Randazzo	Detroit Edison
Rob Woolley	Detroit Edison
Mari-Josette Campagnone	NRR/DBL
C. T. Coddington	Pennsylvania Power & Light Co.
R. R. Sgarro	Pennsylvania Power & Light Co.
W. E. Barberich	Pennsylvania Power & Light Co.
M. David Lynch	NRR/DBL
Steve Frost	Detroit Edison - Fermi-2
Paul Christofakis	Detroit Edison - Fermi-2
Paul Eddy	New York State Public Service Comm.
Gerald Klingler	IE/DI/ORPB
Steve Washington	WNP-2
Dennis Vandeputte	SWEC
Greg Brown	Stone & Webster
Fred Stetson	NUS Corp.
George S. Daves	PSE&G - Hope Creek

ENCLOSURE 2

PROPOSED AGENDA

8:00 a.m. INTRODUCTION

- Elinor G. Adensam, BWR, PD #3
- Robert M. Bernero, Director, DRL

8:30 a.m. NRC/UTILITY LIVING SCHEDULES

- Tracking System in use by NRC (Bournia)
- Systems used by Utilities (Coddington/PP&L, Wooley/Detroit Edison)
- Benefits of a Utility/NRC Integrated Living Schedule (Adensam)

9:30 a.m. UTILITY PERFORMANCE

- How do Utilities track their own performance (~~Powell~~ Sorensen/WPPSS)

9:45 a.m. COFFEE BREAK

10:00 a.m. LEGAL CONSIDERATIONS

- Sholly Process/NSHC (Scinto)
- Exemptions and the new Rule (Cameron)
- Emergency TS changes (Campagnone/Scinto)
- Interpretation of 10 CFR 50.59 (Scinto)

12:00 LUNCH

1:30 p.m. TECHNICAL SPECIFICATION IMPROVEMENTS

- Introduction/Background (Butcher)
- NRC's role (Butcher)
- Utility Participation (Sgarro/PP&L)

2:45 p.m. COFFEE BREAK

3:~~00~~ 15 p.m. SEVERE ACCIDENT POLICY/DEGRADED CORE PROGRAMS

- Presentation/Discussion (Bernero)

3:~~45~~ 00 p.m. What is DISCRETIONARY ENFORCEMENT

- Presentation (Vollmer)

4:00 p.m. OPEN DISCUSSION

4:30 p.m. Closing Remarks (Adensam - Bernero)

5:00 p.m. Close

8:00 A.M. INTRODUCTION

ELINOR G. ADENSAM

ROBERT M. BERNERO

8:30 A.M. NRC/UTILITY LIVING SCHEDULES

ANTHONY BOURNIA (NRC)

CORNELIUS CODDINGTON (PP&L)

ROBERT WOOLLEY (DETROIT EDISON)

ELINOR ADENSAM (NRC)

TRACKING SYSTEM IN USE BY NRC

NRC FORM 187 (11-84)

U.S. NUCLEAR REGULATORY COMMISSION

TECHNICAL ASSIGNMENT CONTROL

OFFICE OF NUCLEAR REACTOR REGULATION

TAC NUMBER * 62246

[X] NEW ASSIGNMENT
[ ] NEW INFORMATION

SECTION I. REQUEST DATA

PREPARED BY (Name): A. Bournia

MAIL STOP: P-904-C

(AD) DATE PREPARED * MONTH DAY YEAR: 09 02 86

(AE/AF) FACILITY NAME AND ASSIGNMENT TITLE * (Limit to 120 characters): La Salle Unit 1 - Static-O-Ring switch analysis for Unit 1 startup

(BA) DOCKET NUMBER *: 50- 373

(AZ) PLANNED ACCOMPLISHMENT NUMBER: 1 1 1 1

(AH) REQUESTING TARGET DATE MONTH DAY YEAR: 09 10 86

(AG) REQUEST CONTACT *: A. Bournia

(AX) REQUEST ER'S INITIALS *: ABB

(AB) REQUESTING ORGANIZATION *: BWD-3

(AL) MULTIPLANT ACTION NUMBER:

(E) REVIEW BY REGION: [ ] 1 [ ] 2 [ ] 3 [ ] 4 [ ] 5

ACCESSION NUMBER:

[X] OR COPY OF INCOMING ATTACHED

SECTION II. SYSTEMS CONTROL DATA

A. OPERATING REACTOR ACTIONS

PRIORITY *	[X] 1	[ ] 2	[ ] 3	[ ] N/A
(FB) FEE RECOVERABLE *		[X] YES	[ ] NO	
(FP) APPLICATION FEE RECEIVED *		[ ] YES	[X] NO	

(AR) APPLICATION DATE * MONTH DAY YEAR: 08 29 86

B. TOPICAL REPORT REVIEWS

VENDOR'S NAME:

REPORT IDENTIFICATION SYMBOL

(DA) P-PROPRIETARY:

(AR) REPORT DATE MONTH DAY YEAR	(AS) ADDITIONAL INFORMATION REQUEST DATE MONTH DAY YEAR	(AT) SUBMIT DATE MONTH DAY YEAR	(AU) DATE LETTER TO VENDOR MONTH DAY YEAR

(EA) NP-NONPROPRIETARY VERSION:

(FA) NONPROPRIETARY REPORT:

(AV) STATUS: [ ] ACCEPTED [ ] NOT ACCEPTED [ ] WITHDRAWN

(AJ/AK) REQUESTING REMARKS (Limit to 120 characters):

SECTION III. REVIEW DATA

REVIEWER'S SURNAME	(CA) REVIEWER'S INITIALS *	(CB) ESTIMATED HOURS	COMPLETION DATE (CC) ESTIMATED MONTH DAY YEAR	COMPLETION DATE (CD) ACTUAL MONTH DAY YEAR
A. Bournia	ABB			
J. Knox	JYK			
T. Collins	TEC			

* REQUIRED ENTRIES

Routing Slip

TRANSMITTAL OF DIVISION OF BWR LICENSING WORK REQUEST
SPECIAL HANDLING - PROCESS WITHOUT DELAY

TAC#-Plant Name-Title 62246 - La Salle 1 - SOR switch analysis for Unit 1 startup

Description of review requested: By letter dated August 29, 1986 (attached), the licensee submitted additional information concerning SOR switche in order for the staff to perform analysis review for Unit 1 startup

Request target date Sept 10, 1986

Basis for request date: The unit startup is scheduled for Sept. 15, 1986

Regional Resource Determination:

	CIRCLE ONE
A. Will the review benefit from unique regional knowledge?	YES / (NO)
B. Will the review benefit from regional proximity to the site?	YES / (NO)

C. If the answer to A and/or B is "yes", explain why the review is not being sent to the Regions.

SEQUENCE	NAME	DATE
1. Project Manager	A. Bournia	
2. Project Director	E. Adensam	

☒ This action is requested to be added to the review branch's current commitments

☐ This action is requested to be completed in lieu of TAC# _____ for _____ (Plant Name)

3. Review Branch Chief W. Hodges _____

4. Regional Div. Director/Asst. Director-BWR Licensing G. Lainas _____

☐ This action is accepted for completion with a target date of _____

☐ This action is accepted for completion with a target date of _____ in lieu of completing TAC# _____ in this fiscal year

7. Return to Review Branch Chief for assignment of reviewer and retention of work package

_____ (Reviewer's Name) _____ (RAMS Initials)

8. Return routing slip to originator

FROM	MAIL STOP	PHONE
A. Bournia	P-904-C	28698

DIVISION OF LICENSING
LICENSING ACTIONS REPORT
(HEADQUARTERS)

FACILITY: LASALLE 1

PLANT LOCATION: 11 MI SE OF OTTAWA, ILL
DOCKET NUMBER: 050-00373
ARCH/ENGINEER: S&L
IE INSPECTOR: R. HEISHMAN

LICENSED POWER: 3323 MWT
DESIGN POWER: 1078 MWE
NSSS VENDOR: GE

PROJECT MANAGER: A. BOURNIA
BRANCH CHIEF: E. ADENSAM
LIC. ASSISTANT: E. HYLTON

TAC NUMBER	MULTI PLANT ACTION	INIT DATE	TYPE	NOTICE EXPIRES	PRI	REV ORGN	REVIEWER	RAI DATE	RAI RESPONSE DATE	IER DATE	SE INPUT DATE	LIC ACT DATE	COMMENTS	SCTOAD IE
I. ACTIVE AND COMPLETED ACTIONS IN FY														
**** ACTIVE ACTIONS ****														
ACTION ITEM: LASALLE 1 - ENVIRONMENTAL QUALIFICATION OF SAFETY RELATED ELECTRICAL EQUIPMENT (80-CLI-21)														
M42536	B-60	08/20/80	G	N/A	1	BWD3	BOURNIA				11/30/85	09/15/86		01
													LICENSEE IMPLEMENTATION DATE & STATUS CODE	09/00/86L
ACTION ITEM: LA SALLE 1 - INSTRUMENTS FOR DETECTION ON INADEQUATE CORE COOLING (II.F.2)														
M49460	F-26	12/10/82	G	N/A	3	BWD3	BOURNIA					06/18/87		01
						BWRS	HUANG				06/14/85X			
													LICENSEE IMPLEMENTATION DATE & STATUS CODE	00/00/87L
ACTION ITEM: LASALLE 1 - INSTRUMENTATION TO FOLLOW THE COURSE OF AN ACCIDENT (RG 1.97)														
M51102	A-17	04/04/83	G	N/A	3	PAEI	ROSA	03/31/85			07/18/86X	01/24/87		01
						BWD3	BOURNIA				01/10/86			
													LICENSEE IMPLEMENTATION DATE & STATUS CODE	00/00/87L
ACTION ITEM: LASALLE 1 - I.D.1.1 - DETAILED CONTROL ROOM DESIGN REVIEW PROGRAM PLAN F-08														
M51172	F-71	04/04/83	G		3	HFIB	FROELICH				07/11/83X	10/10/86		02
						PAEI	ECKENRODE				07/07/83P			
						BWEI	RAMIREZ				07/04/86X			
						BWD3	BOURNIA							
													LICENSEE IMPLEMENTATION DATE & STATUS CODE	00/00/89L
ACTION ITEM: LASALLE 1 - ITEM 2.1 - EQUIPMENT CLASSIFICATION AND VENDOR INTERFACE - RTS COMPONENTS														
M52850	B-77	11/01/83	G	N/A	3	BWD3	BOURNIA		11/25/83C			04/10/87		02
						PAEI	LASHER				*03/31/87*			
													LICENSEE IMPLEMENTATION DATE & STATUS CODE	00/00/87L

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ONRR STAFF INPUT REPORT OFFICE: NRR DIVISION: DBL A/D: WEEK ENDING 09/06/86

EMPLOYEE - (ABB) - A BOURNIA BRANCH: BWD3 SECTION: RUN DATE 08/29/86

TACS # / DOCKT# / PROJ #	RPT NO / DATE OF REQUEST	PA #	TARGET DATE	ESTIMATE DATE	STATUS/ COMPL. DATE	HOURS REG	HOURS NON REG	ESTM M/HR	TITLE/DESC
M618892	07/09/86	1111	08/15/86		ACTIVE	___.-	___.-		LA SALLE 2 - REPORTING REQUIREMENTS FOR PRIMARY COOLANT HIGH REACTIVITY
M61890	07/09/86	1111	08/15/86		ACTIVE	___.-	___.-		LA SALLE 2 - CABLE SEPARATION CONCERNS
M61906	07/10/86	1111	08/15/86		ACTIVE	___.-	___.-		LA SALLE 1 - REVIEW OF BULK-TO-LOCAL POOL TEMPERATURE DIFFERENCE
M62093	08/07/86	1111			ACTIVE	___.-	___.-		LA SALLE 1 - REVIEW OF LICENSE REVISION TO PCP
M62094	08/07/86	1111			ACTIVE	___.-	___.-		LA SALLE 2 - REVIEW OF LICENSE REVISION TO PCP
M62095	08/07/86	1111			ACTIVE	___.-	___.-		LA SALLE 1 - REVIEW OF LICENSE REVISIION TO ODCM
M62096	08/07/86	1111			ACTIVE	___.-	___.-		LA SALLE 2 - REVIEW OF LICENSE REVISION TO ODCM
M621362	08/13/86	1163			ACTIVE	___.-	___.-		LA SALLE 1 - ASSISTANCE TO R-III FOR REVIEW OF TYPE "A" TEST VALVE LINEUP (TIA 86-41B)
05000373		1125				___.-	___.-		LASALLE 1 PROJECT MANAGER ADMINISTRATION
05000374		1125				___.-	___.-		LASALLE 2 PROJECT MANAGER ADMINISTRATION
***** NEW ACTIVITIES *****									
_____		____ _____				___.-	___.-		NEW WORK ITEM ADD TO WEEKLY REPORT? YES__
_____		____ _____				___.-	___.-		NEW WORK ITEM ADD TO WEEKLY REPORT? YES__
_____		____ _____				___.-	___.-		NEW WORK ITEM ADD TO WEEKLY REPORT? YES__
_____		____ _____				___.-	___.-		NEW WORK ITEM ADD TO WEEKLY REPORT? YES__
***** GENERAL ACTIVITY *****									
		174				___.-	___.-		CORRESPONDENCE AND F.O.I.A.
		181				___.-	___.-		SUPERVISION AND MANAGEMENT
		19				___.-	___.-		ABSENCE (EXCLUDING LWOP)
		185				___.-	___.-		TRAINING AND PROFESSIONAL MEETINGS

R-1208775 REGULATORY INFORMATION TRACKING SYSTEM PAGE 887

ONRR STAFF INPUT REPORT OFFICE: NRR DIVISION: DBL A/D: WEEK ENDING 09/06/86

EMPLOYEE - (ABB) - A BOURNIA BRANCH: BWD3 SECTION: RUN DATE 08/29/86

TACS # / DOCKT # / PROJ #	RPT NO / DATE OF REQUEST	PA #	TARGET DATE	ESTIMATE DATE	STATUS/ COMPL. DATE	HOURS REG	HOURS NON REG	ESTM M/MR	TITLE/DESC

I CERTIFY THAT, TO THE BEST OF MY KNOWLEDGE, THE TIME ALLOCATIONS REPORTED ON THIS FORM ARE ACCURATE.

EMPLOYEE'S SIGNATURE & DATE: _____

IMMEDIATE SUPERVISOR'S SIGNATURE & DATE: _____

SUSQUEHANNA COMMITMENT TRACKING AND

SCHEDULING

- LICENSING COMPLIANCE LIST
- NUCLEAR DEPARTMENT FIVE YEAR PLAN
- YEARLY MAJOR CAPITAL WORK PROJECTS PLAN

C.T. Coddington
September 9, 1986

NUCLEAR DEPARTMENT FIVE YEAR PLAN

- PURPOSE - IDENTIFY MAJOR WORK PROJECTS
  - FORECAST OUTAGE WORK PROJECTS
  - ESTABLISH PROJECT OBJECTIVES (MILESTONES)
  - PROVIDE A BASELINE FOR LONG TERM PLANNING
  - FORECAST REQUIREMENTS FOR FINANCIAL RESOURCES

- PROJECT CATEGORY - REACTOR SAFETY
  - PERSONNEL SAFETY
  - REGULATORY
  - PLANT BETTERMENT
  - ALARA
  - OTHER

- PROVIDES MANAGEMENT LEVEL SCHEDULE

LICENSING COMPLIANCE LIST

- OBJECT - PROVIDE A CONCISE METHOD TO ENSURE THAT OPEN REGULATORY ITEMS ARE CLOSED OUT

- COMPUTER LISTING

- LISTING CONSISTS OF:

  LER'S AND SPECIAL REPORTS
  NRC INSPECTION REPORT ITEMS
  LICENSE CONDITIONS
  MISC. LETTERS FROM NRC
  REQUIRED NRC REPORTS
  SAFETY EVALUATION REPORT OPEN ITEMS
  TECHNICAL SPECIFICATION CHANGE REQUESTS

- UPDATED AND ISSUED MONTHLY

YEARLY MAJOR CAPITAL WORK PROJECTS PLAN

- PROVIDES LISTING OF ITEMS WHICH HAVE BEEN APPROVED FOR WORK DURING THE YEAR

- PROVIDES DETAIL SCHEDULES OF ACTIVITIES

SSES FIVE YEAR PLAN
PROPOSED 1987 MAJOR PROJECTS

Title	Unit	Activity	Priority
1986 Project Carryovers			
1. Turbine-Generator Rotor Replacement	2	E&I	76
2. ATWS	1/2	E&I	66
3. Loss of AC Instrument Indication	1/2	E&I	54
4. Radwaste Solidification Evaluation	C	E	54
5. Drywell Cooling Modifications	1	I	50
6. Spare Single Phase Transformer	2	E&I	48
7. Control Room Design Review	1/2	E	46
8. Feedwater LLRT Mods	1	E&I	46
9. Appendix "R" Mods	1	E&I	44
10. Turbine-Generator Maintenance Items	1	I	38
11. Radwaste Phase "C"	C	I	36
12. Electric Power Interruption Mods	1/2	E&I	36
13. Nuisance Alarms	1/2	E&I	36
14. SPING Enhancements	1/2	I	36
15. Cross Around Piping Painting	1	I	34
16. Spare Penetrations	1/2	E&I	30
17. ECCS & RCIC Keep Fill Modification	1	E	28
18. RCU Charging Water Check Valve Leakage	1	E	26
19. ESW Butterfly Valve Changeout	C	E	22
20. Hydrogen Water Chemistry	1/2	E&I	20
21. Access Improvements	1/2	E&I	18
22. Simulator Enhancements	C	E&I	18
23. Cowanesque Reservoir Modification	C	I	N/A
New Projects:			
1. MSIV Valve Modification (Study)	1/2	E	42
2. Feedwater Heater Cooldown Line	1/2	E	38
3. PASS System Upgrade	1/2	E	38
4. SRV Position Indication Redundancy	1/2	E	36
5. Condenser Waterbox Vent Valve Mods	1/2	E	36
6. Diesel Generator Reliability Enhancements (Study)	C	E	36
7. Drywell Cooling Isolation Valve Power Supplies	2	E	36
8. Compressed Air Systems Cooling Supply	1/2	E&I	35
9. Fire Protection Modifications	1	E	34
10. Excore NMS Amplifier Relocation/Logic Mod	2	E	32
11. Circ Water Chemical Treatment System	1	E	30
12. Condensate Demin URC Flow Rate Mod	2	E&I	28
13. ESW Piping Changeout (Seal Water Coolers)	1/2	E	28
14. Feedwater Sample Probe Changeout	1	E	28
15. RHR Shutdown Cooling Valve Operability	1	E	28
16. Turbine Bldg. LP Hanger Mods	1	E	25
17. Turbine Bldg. SP Hanger Mods	2	E&I	25
18. Dryer/Separator Pool Water Seal	1/2	E&I	22
19. Auxiliary Boiler Reliability Enhancements (Study)	C	E	18
20. Recirc Pump Upper Cavity Vent Valve	1	E	18
21. Fault Recorders	1	E	11

TOTAL PROJECTS - 44

SSES UNIT 2 LICENSING COMPLIANCE LIST
(ALL ITEMS)

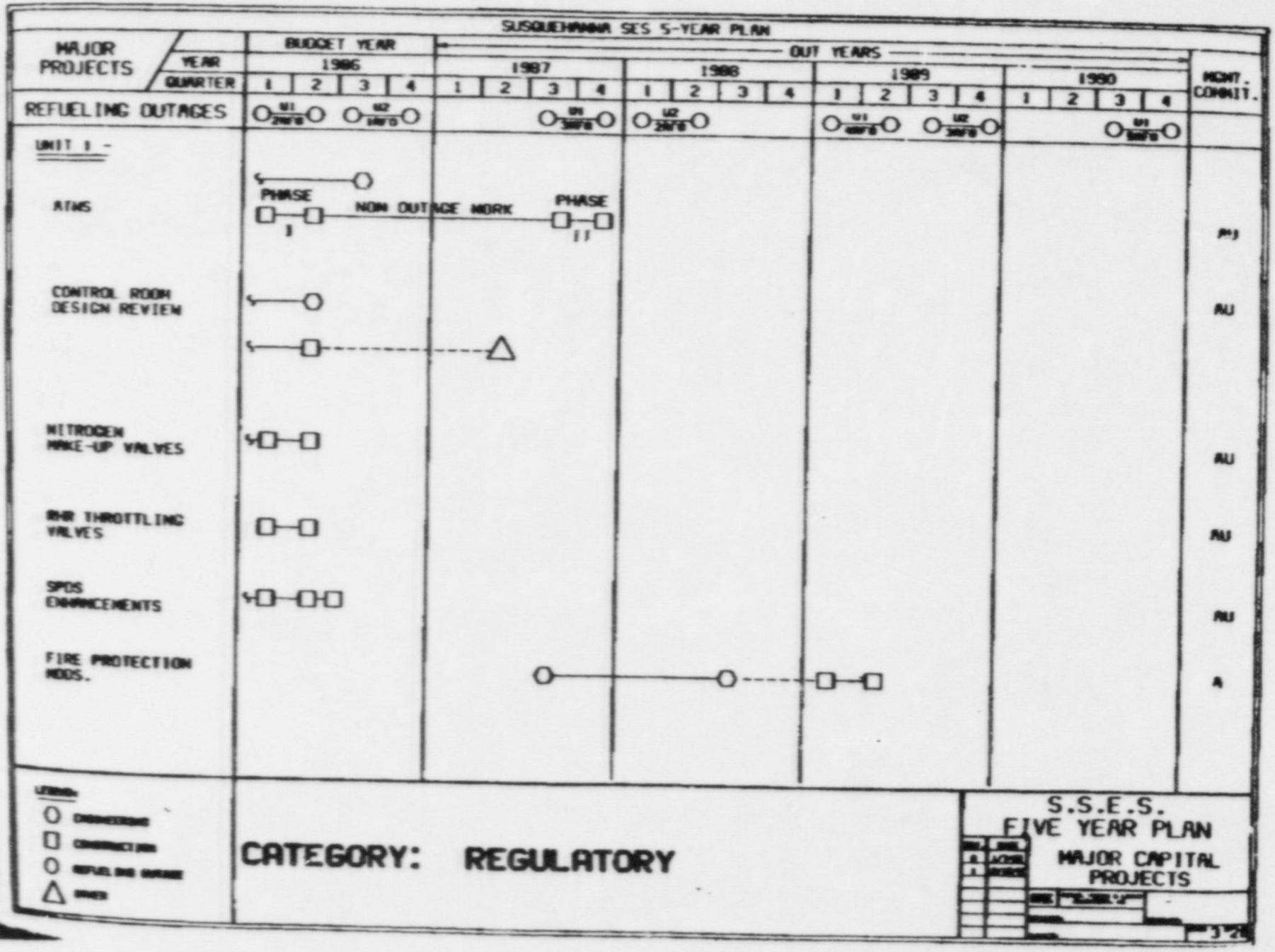
07/25/86

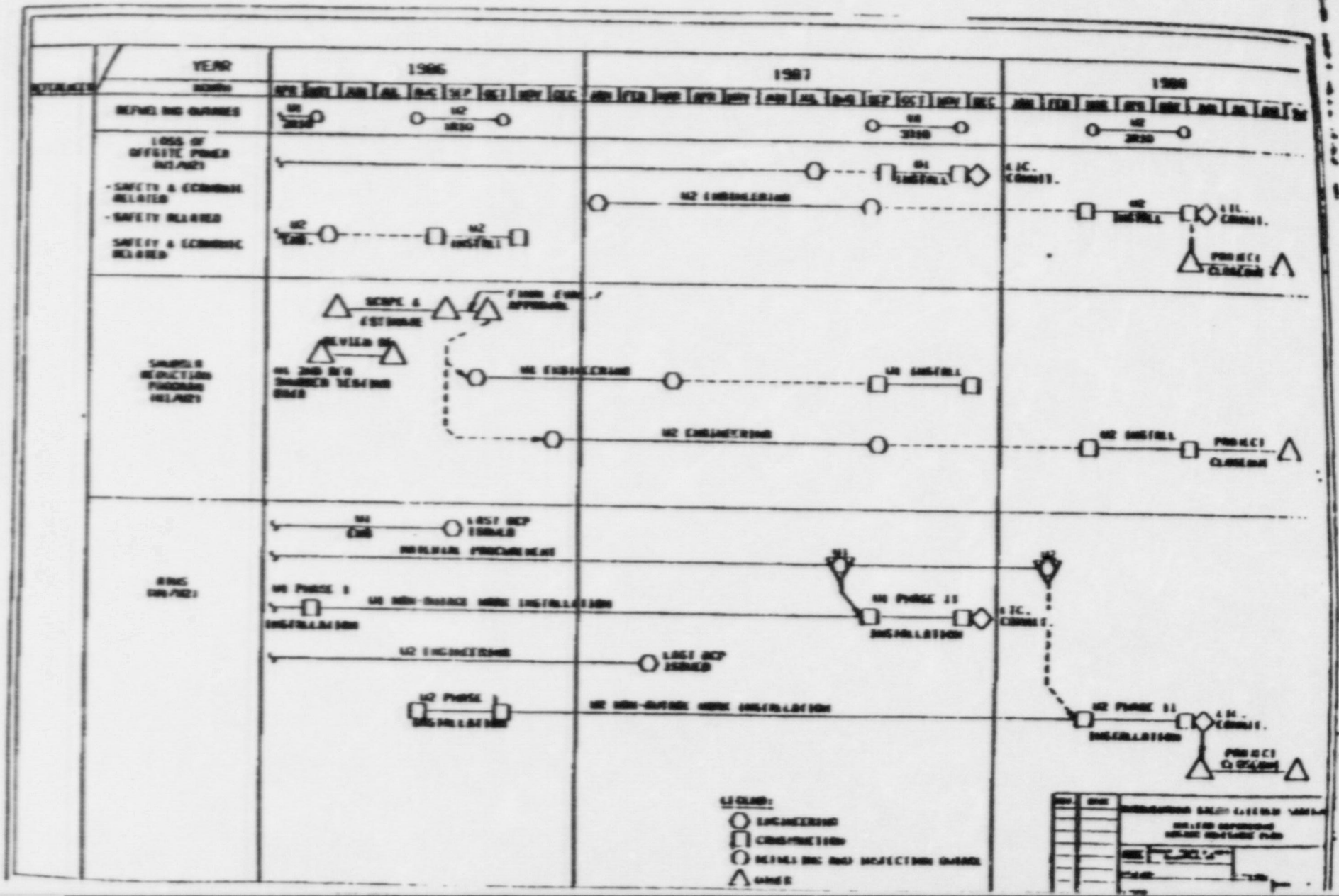
ITEM NUMBER	RESP ORG/IND	TARGET AND REVD DATES	STATUS	DESCRIPTION	A	B	C	D
UNIT 2 **** TECH SPEC CHANGE REQUESTS								
OUTAGE TS		R-1ST REF		LICENSING. THESE CHANGES ARE REQUIRED IN SUPPORT OF THE UNIT 2 1ST REFUELING AND INSPECTION OUTAGE.		X		
	NPE/JGR	T-05/23/86	COMPLETE	SUBMIT RELOAD PACKAGE TO LICENSING. (LICENSING TO SUBMIT BY 6/9/86)				
TS-107		R-1ST REF	CLOSED	REVISE THE OVERCURRENT PROTECTION AND ASSOCIATED TESTING REQUIREMENTS FOR 480V CONDUCTORS IMBEDDED IN CONTAINMENT PENETRATION ASSEMBLIES (DCP 85-621) REQUIRED PRIOR TO STARTUP FOLLOWING THE UNIT 2 FIRST REFUELING AND INSPECTION OUTAGE.	X	X		
	LIC	T-10/18/85	COMPLETE	SUBMIT PROPOSED AMENDMENT TO NRC. SEE PLA-2544, PROPOSED AMENDMENT 25				
	NRC	T-12/30/85	COMPLETE	NOTICE PROPOSED AMENDMENT IN FEDERAL REGISTER.				
	NRC	T-04/01/86	COMPLETE	ISSUE LICENSE AMENDMENT 074.				
TS-112		R-NONE	X-REF	SEE UNIT 1 TECH SPEC CHANGE #112	X	X		
TS-115		R-NONE	X-REF	SEE UNIT 1 TECH SPEC CHANGE #115.	X	X		
TS-117		R-1ST REF	OPEN	ALLOW ISOLATION OF ZONE I OR ZONE II FROM ZONE III IN CONDITION a. DESIRED PRIOR TO SHUTDOWN FOR UNIT 2 1ST REFUEL & INSPECTION OUTAGE. REQUIRED TO INSTALL DCP 86-3130.	X	X		
	NPE/JGR	T-02/12/86	COMPLETE	COMPLETE CALCULATIONS SUPPORTING SAFETY IMPACT ASSESSMENT. EWR NMIS 86-0575				
	SUP	T-05/07/86	COMPLETE	OBTAIN PORC APPROVAL				
	LIC	T-05/12/86	COMPLETE	OBTAIN SRC APPROVAL				
	LIC	T-05/14/86	COMPLETE	SUBMIT PROPOSED AMENDMENT TO NRC. SEE PLA-2643, PROPOSED AMENDMENT 037 (UNIT 2) AND 04 (UNIT 1).				
	NRC	T-07/02/86	ACTION	NOTICE PROPOSED AMENDMENT IN FEDERAL REGISTER.				
	NRC	T-08/02/86	ACTION	ISSUE LICENSE AMENDMENT.				
TS-116		R-NONE	X-REF	SEE UNIT 1 TECH SPEC CHANGE #116.	X	X		
TS-122		R-NONE	X-REF	SEE UNIT 1 TECH SPEC CHANGE #122.	X	A		
TS-123		R-NONE	X-REF	SEE UNIT 1 TECH SPEC CHANGE #123	X	X		

NUCLEAR DEPARTMENT
FIVE-YEAR MILESTONE PLAN

MAJOR CAPITAL PROJECTS

CATEGORY	TITLE
Reactor Safety	
	Powerplex Unit 1
	Drywell Cooling-Isol.Vlv Pwr. Supplies
Personnel Safety:	
Regulatory:	
	ATWS Unit 1/2
	Control Room Design Review Unit 1/2
	Nitrogen Makeup Valves Unit 1
	RWR Throttling Valves Unit 1/2
	SPDS Enhancements Unit 1/2
	Fire Protection Mods. Unit 1/2
	Loss of AC Instr. Indication Unit 1/2
	Appendix "R" Mods Unit 1/2
	ESW Piping Changeout Motor Oil Coolers Unit 1/2
	PASS System Upgrade Unit 1/2
	Cowanesque Reservoir Mod. Unit C
	Hazardous Waste Facility Unit C
	Sewage Treatment Plant Unit C
Plant Betterment:	
	Alternate HPCI Room Cooling Unit 1
	Degraded Grid Voltage Unit 1/2
	Drywell Cooling Mods Unit 1/2
	o Phase II Unit 1/2
	Feedwater LLRT Mods Unit 1/2
	Turbine Generator Maint Items Unit 1/2
	Turbine Generator Rotor Replacement Unit 1/2
	Access Improvements Unit 1/2
	ECCS & RCIC Keepfill Mod Unit 1/2
	Electric Power Interruption Mods Unit 1/2
	HCU Charging Water Check Vlv. Leakage Unit 1/2
	Hydrogen Water Chemistry Unit 1/2
	SPING Enhancements Unit 1/2
	Circ. Water Chemical Treatment System Unit 1/2
	Compressed Air Systems Cooling Supply Unit 1/2


SUSQUEHANNA SES 5-YEAR PLAN
MAJOR PROJECTS
YEAR
QUARTER
BUDGET YEAR
1986
OUT YEARS
1987
1988
1989
1990
1 2 3 4
MGMT. COMMIT.
REFUELING OUTAGES
UNIT 1 -
ATWS
PHASE
I
NON OUTAGE WORK
PHASE
II
MU
CONTROL ROOM DESIGN REVIEW
AU
NITROGEN MAKE-UP VALVES
AU
RHR THROTTLING VALVES
AU
SPDS ENHANCEMENTS
AU
FIRE PROTECTION MODS.
A
CATEGORY: REGULATORY
S.S.E.S.
FIVE YEAR PLAN
MAJOR CAPITAL PROJECTS


YEAR
1986
1987
1988
LEGEND:

COMMITMENT TRACKING AND SCHEDULING AT FERMI 2

- REGULATORY ACTION AND COMMITMENT TRACKING SYSTEM (RACTS)
- FERMI 2 INTEGRATED MASTER PLAN

R. L. WOOLLEY
SEPTEMBER 9, 1986

OBJECTIVES

RACTS HAS TWO PRIMARY OBJECTIVES

- ENSURE REGULATORY ACTION ITEMS AND COMMITMENTS ARE SATISFIED
- ONCE SATISFIED, ACTION ITEMS AND COMMITMENTS REMAIN SATISFIED

PROCESS

- ACTION ITEMS AND COMMITMENTS IN INCOMING AND OUTGOING CORRESPONDENCE IDENTIFIED
- ITEMS ENTERED IN COMPUTERIZED DATABASE AND ASSIGNED
- RESPONSE VERIFIED BY QA PRIOR TO CLOSURE

CONTINUING COMPLIANCE

- CROSS REFERENCE REPORT DISTRIBUTED ON A MONTHLY BASIS
- REPORT LISTS ALL IMPLEMENTING DOCUMENTS ALPHA NUMERICALLY AND CROSS REFERENCES THEM TO THE SEQUENCE NUMBER OF THE COMMITMENTS WHICH THEY IMPLEMENT

INTEGRATED MASTER PLAN

- PROJECT ? BASED
- RECENT FERMI ? TOOL
- USED FOR 08/04/86 RESTART
- DEVELOPING 18 MONTH LOOK AHEAD
- DATED RACTS COMMITMENTS ARE SEMI-AUTOMATICALLY INCLUDED
- WEEKLY REVIEW

"LIVING" SCHEDULES

- OBJECTIVES
- ADMINISTRATIVE IMPACT
- PROCEDURE
- GAINS

ELINOR G. ADENSAM
SEPTEMBER 9, 1986

OBJECTIVES

- FOCUS ON REAL SAFETY ISSUES
- COORDINATION OF EFFORTS
  - LICENSEE
  - REGION
  - NRR
- PROPER ALLOCATION OF RESOURCES

ADMINISTRATIVE IMPACT

- NOTICE GENERATION
- NOTICE PERIOD
- STAFF RESOURCES

PROCEDURE

- DEVELOP LIST & PRIORITIES
  - LICENSEE
  - REGION (RESIDENT)
  - NRR (PM)
- MEET TO DISCUSS & AGREE ON PRIORITIES
- PROPOSE COMPROMISES WHERE PRIORITIES NOT AGREED UPON
- USE MEETINGS TO DISCUSS OTHER LICENSING ACTIVITIES
- SCHEDULE PERIODIC UPDATE MEETINGS TO ADJUST SCHEDULES/ PRIORITIES AS NECESSARY

GAINS

- STAFF/LICENSEE EFFORTS ARE FOCUSSED ON SAFETY FIRST
- SOME DEGREE OF ASSURANCE OF AVAILABLE STAFF RESOURCES
- REDUCED POTENTIAL FOR "EMERGENCIES"
- LICENSEE KNOWS A "NICE-TO-HAVE" MAY BE DELAYED
- LICENSEES HAVE A FORUM TO JUSTIFY THEIR REQUESTS & PRIORITIES
- STAFF CAN ANTICIPATE THEIR WORKLOAD

9:30 A.M. UTILITY PERFORMANCE

G. C. SORENSEN (WPPSS)

PERFORMANCE INDICATOR REPORT
OPERATIONAL AREAS

OPERATIONS

MAINTENANCE

TECHNICAL

HP/CHEMISTRY

ADMINISTRATION

TRAINING

QUALITY ASSURANCE

SUPPORT SERVICES

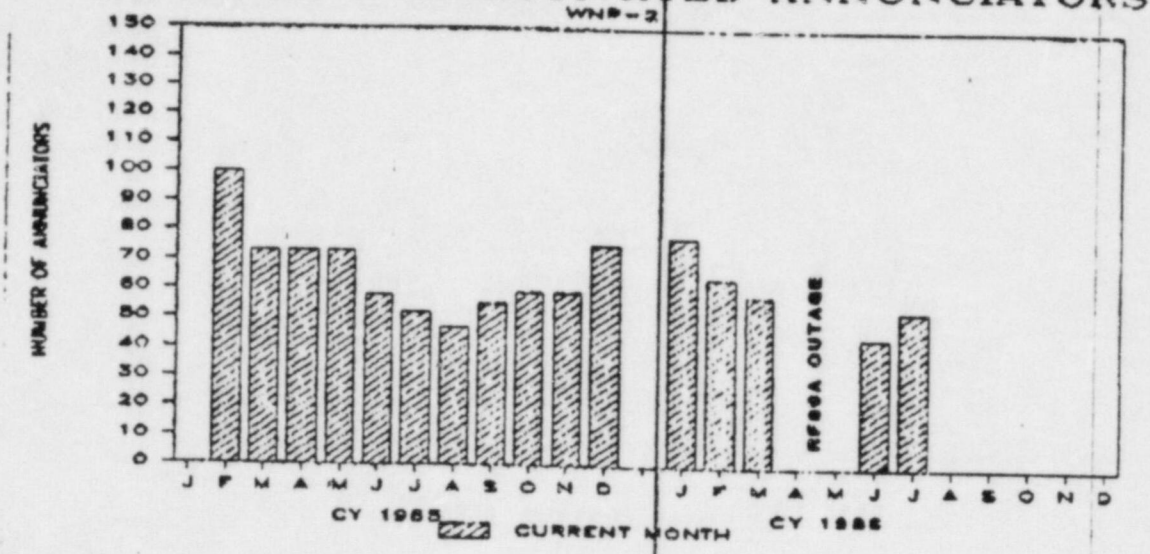
MATERIAL MANAGEMENT

G. C. SORENSEN
SEPTEMBER 9, 1986

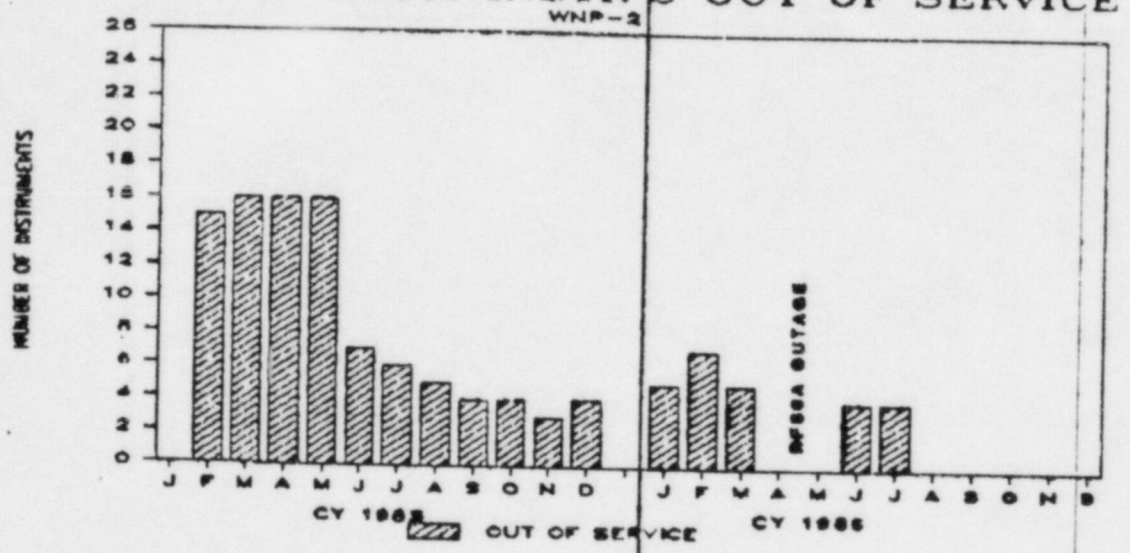
CONTROL ROOM INSTRUMENT OPERABILITY

This first graph below depicts the number of control room annunciators activated during plant operation while at power for a duration exceeding 24 consecutive hours. The second graph depicts the number of control room instruments that are not performing their design function, regardless of the reason. This includes instruments on control room back panels.

CONTINUOUSLY ACTIVATED ANNUNCIATORS



CONTROL ROOM INSTR'S OUT OF SERVICE

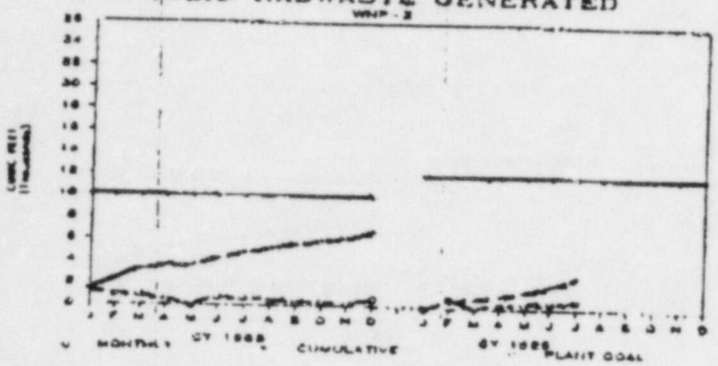


SOLID RADWASTE

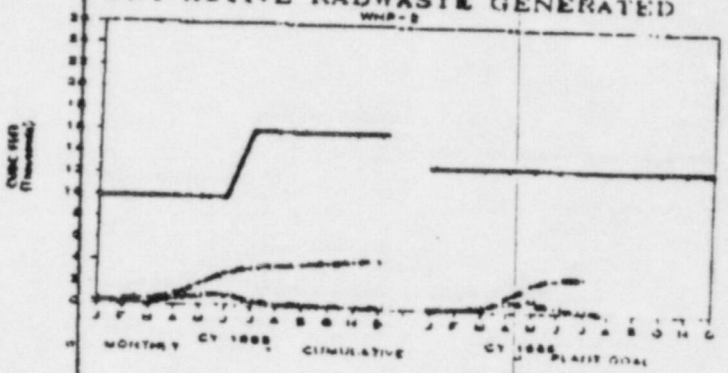
Solid radwaste includes two products: (1) solidified radioactive waste which is the accumulation of radioactive products (sludge, resins, filter cake) removed by liquid and gaseous processing systems; and (2) contaminated solid materials (disposable gloves, smears, trash) generated as a result of maintenance activities. These graphs depict the monthly and cumulative cubic feet of solidified radioactive waste and dry active waste generated at WNP-2 and subsequently shipped off site.

For FY 87, WNP-2's goal is to limit the volume of solid radwaste shipped from the plant to 25,000 cubic feet.

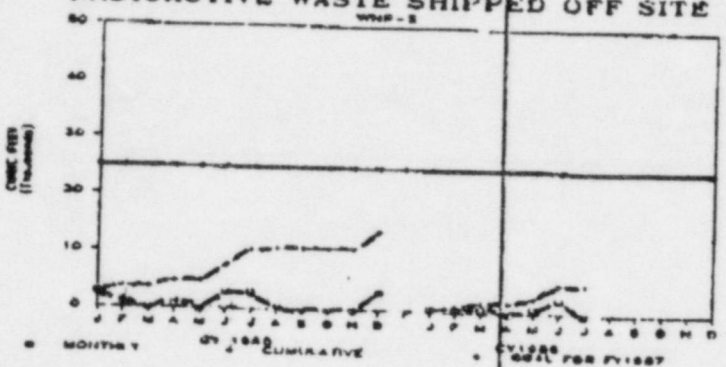
SOLID RADWASTE GENERATED



DRY ACTIVE RADWASTE GENERATED



RADIOACTIVE WASTE SHIPPED OFF SITE



This Month: Total radioactive waste (i.e., solidified and dry active) generated during July was 959 cubic feet.

QUALITY PROGRAM TREND REPORT

NONCONFORMANCE REPORTS (NCRs)

PLANT DEFICIENCY REPORTS (PDRs)

QUALITY SURVEILLANCE REPORTS

AUDIT REPORTS

NRC CITATIONS

LICENSEE EVENT REPORTS (LERs)

TECHNICAL SPECIFICATIONS VIOLATIONS

WNP-2 TREND ANALYSIS REPORT FOR THE FIRST QUARTER OF 1986

The results of this analysis are:

ITEM NO.	ITEM	EVENT 2Q85	FREQUENCY 3Q85	BY 4Q85	QUARTER 1Q86
1	Level switch set point drift	4	5	4	4
2	Valve leaks	24	7	4	8 (1)
3	Valve functional failure	11	6	3	7 (2)
4	Security not notified of employee termination.	4	2	3	3
5	Calibration control problems	0	2	1	7 (3)
6	Misc. problems with records	24	14	13	12 (4)
7	Items incorrectly stored	0	1	1	4
8	Tech Spec surveil not performed in time	4	7	3	4

Notes:

(1) Five (5) of these were containment isolation valve tests

(2) Three (3) of these were MSLC

(3) Six (6) of these were one surveillance

(4) Not considered excessive

Each of the items listed above was analyzed for safety significance by applying the following significance factors:

	Significant Test Criteria	Significance Factor (Multiplier)
a.	Frequency in current quarter greater than the average frequency for the previous three quarters	1.20
b.	Frequency in current quarter greater than twice the average frequency for the previous three quarters	1.50
c.	Event resulted in a reactor scram	1.50
d.	Event involved a challenge to a safety system	1.50
e.	Event was reportable to the NRC in more than 4 hours	1.50
f.	Event was reportable to the NRC within 4 hours	1.65
g.	Event was reportable to the NRC within 1 hour	1.80

AUDIT DEFICIENCIES

Fig 19 REPORTS AND DEFICIENCIES

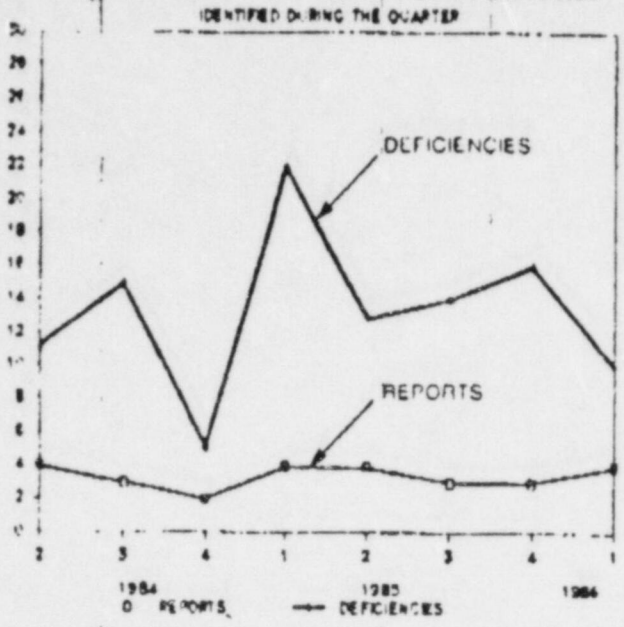
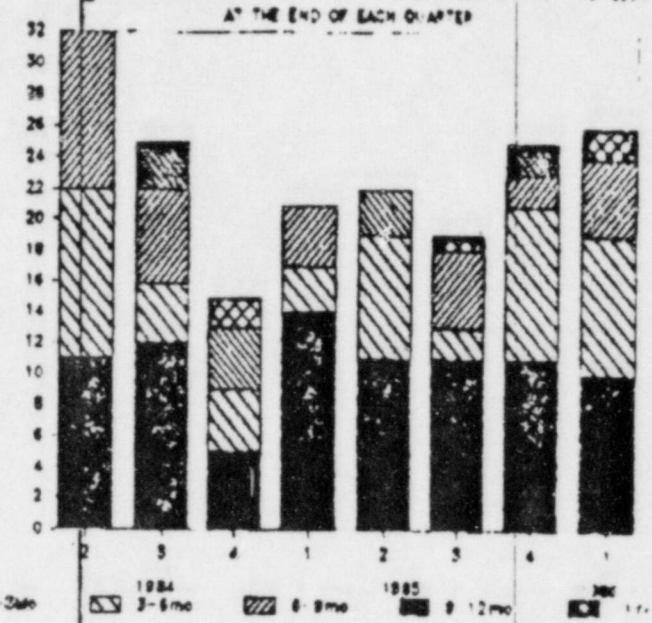
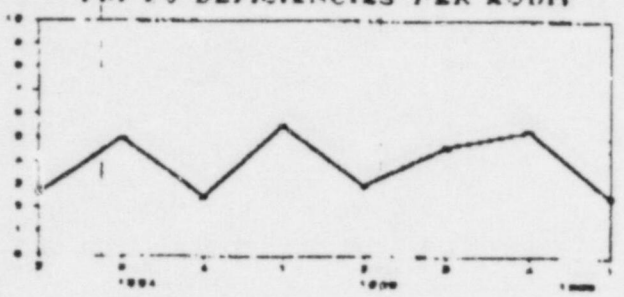


Fig 21 DEFICIENCIES REMAINING OPEN



NO PO DEFICIENCIES PER AUDIT



NO RP OPEN TO CLOSE RATIO

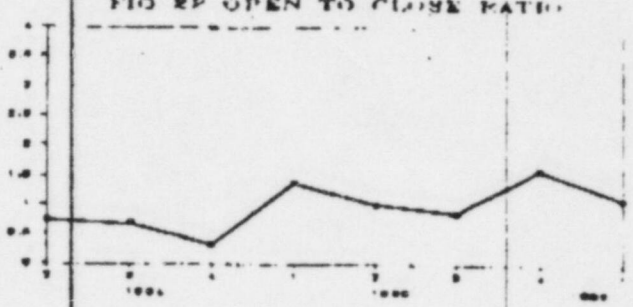


Fig 23 AV NO OF DAYS TO CLOSE

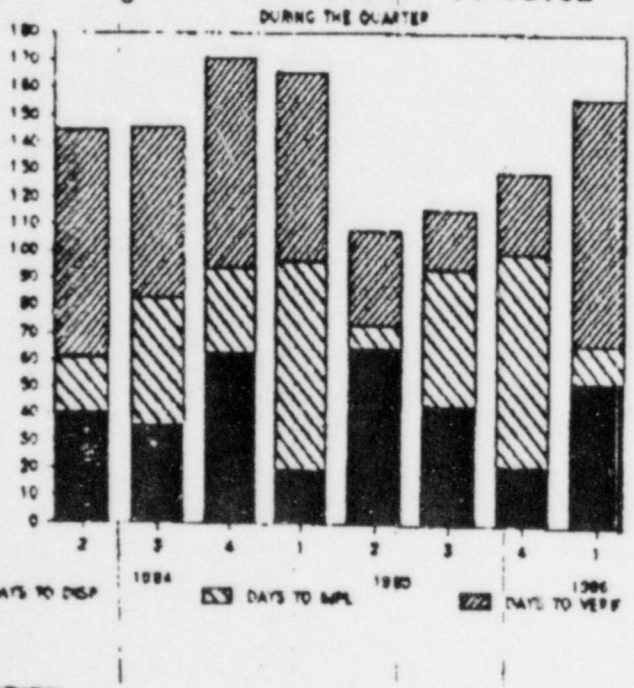
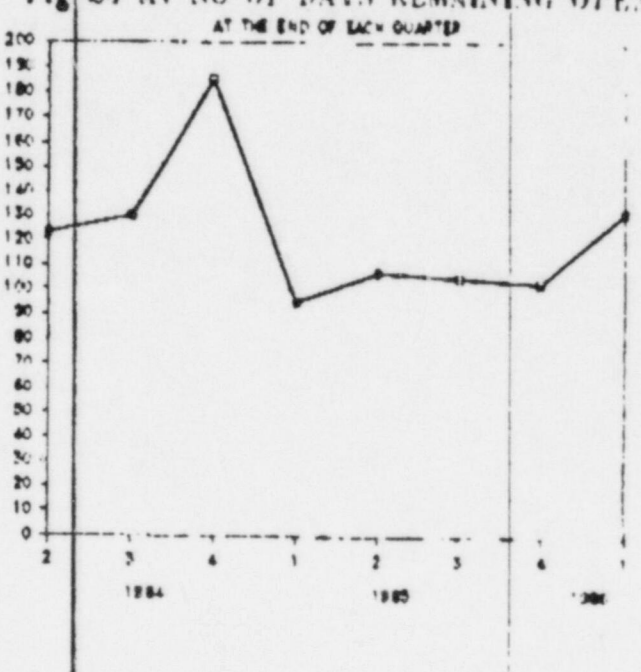


Fig 24 AV NO OF DAYS REMAINING OPEN



10:00 A.M. LEGAL CONSIDERATIONS

J. SCINTO (NRC)

C. CAMERON (NRC)

M. CAMPAGNONE (NRC)

SPECIFIC EXEMPTION
(10 CFR 50.12)

- MUST BE AUTHORIZED BY LAW; NO UNDUE RISK TO PUBLIC HEALTH & SAFETY; CONSISTENT WITH COMMON DEFENSE AND SECURITY

- SPECIAL CIRCUMSTANCES MUST BE PRESENT:
  - CONFLICTS WITH OTHER RULES
  - UNDERLYING PURPOSE NOT SERVED
  - HARDSHIP
  - BENEFIT TO PUBLIC HEALTH AND SAFETY
  - TEMPORARY RELIEF & GOOD FAITH EFFORT
  - MATERIAL CIRCUMSTANCES NOT PREVIOUSLY CONSIDERED

§50.12 SPECIFIC EXEMPTIONS

(a) The Commission may, upon application by any interested person or upon its own initiative, grant exemptions from the requirements of the regulations of this part, which are -

(1) Authorized by law, will not present an undue risk to the public health and safety, and are consistent with the common defense and security.

(2) The Commission will not consider granting an exemption unless special circumstances are present. Special circumstances are present whenever -

(i) Application of the regulation in the particular circumstances conflicts with other rules or requirements of the Commission; or

(ii) Application of the regulation in the particular circumstances would not serve the underlying purpose of the rule or is not necessary to achieve the underlying purpose of the rule; or

(iii) Compliance would result in undue hardship or other costs that are significantly in excess of those contemplated when the regulation was adopted, or that are significantly in excess of those incurred by others similarly situated; or

(iv) The exemption would result in benefit to the public health and safety that compensates for any decrease in safety that may result from the grant of the exemption; or

(v) The exemption would provide only temporary relief from the applicable regulation and the licensee or applicant has made good faith efforts to comply with the regulation; or

(vi) There is present any other material circumstance not considered when the regulation was adopted for which it would be in the public interest to grant an exemption. If such condition is relied on exclusively for satisfying paragraph (a)(2) of this section, the exemption may not be granted until the Executive Director for Operations has consulted with the Commission.

- OPERATING PROCEDURES FOR EMERGENCY LICENCE AUTHORIZATION

- APPLICABILITY
  - PLANT SHUTDOWN
  - DERATE
  - EXTENDED OUTAGE
  - TIME NOT AVAILABLE TO NOTICE (15 DAYS)

- AUTHORIZATION
  - DIVISION DIRECTOR
  - AUTHORITY MAY NOT BE DELEGATED TO ANY ORGANIZATIONAL LEVELS BELOW DIVISION DIRECTOR

MARI-JOSETTE CAMPAGNONE
SEPTEMBER 9, 1986

LICENSEE PROCEDURE

When the licensee determines that the time required to restore components or systems to an operable condition is greater that the period specified in Technical Specification Limiting Conditions of Operation or when a Technical Specification Surveillance Requirement otherwise cannot be satisfied, a formal submittal shall be made to the NRC. This submittal shall contain:

(1) A safety evaluation with a no significant hazards consideration determination;

(2) Revised Technical Specification pages;

(3) A discussion of proposed interim compensatory measures to be imposed;

(4) A discussion of circumstances surrounding the situation, and a determination of why the need for prompt action could not have been avoided;

(5) The scheduled date for returning inoperable components or systems to an operable condition, or the scheduled date for accomplishing required surveillance;

(6) A statement that a best effort has been made to notify State personnel; and

(7) Information for the NRC to prepare an environmental assessment or the basis for NRC to determine that the amendment involves a categorical exclusing under 10 CFR Part 51.

This submittal shall be made promptly to the Director, Office of Nuclear Reactor Regulation, with copies to the Regional Administrator, Office of Inspection and Enforcement, and to the Resident Inspector.

The purpose of prompt reporting is to allow the NRC to review the circumstances of the request for an expedited NRC review and to render a timely decision on whether to authorize continued reactor operations or reactor startup. The prompt submittal shall be made in all cases where NRC action in less than 15 days.

PM PROCEDURE

- The PM shall determine that a complete submittal has been received. If a complete submittal has not been provided, the PM, with Project Director concurrence, shall advise the licensee of the pending denial or, given sufficient time, request a resubmittal or additional information.

- The PM shall review the conclusion that plant shutdown, derate, or extended plant outage will result. If a plant shutdown, derate, or extended outage will not result, the normal procedures for TS changes shall be followed and the PM, with the Project Director concurrence,shall advise the licensee that such action is being taken.

- The PM shall prepare and sign a handwritten safety evaluation (SE) (The handwritten safety evaluation must include the NRC basis for a final no significant hazards consideration determination and a statement supporting the need for an emergency license authorization in light of the circumstances causing the licensee's request. This supporting documentation must state why the requested expedited action could not have been avoided, what appropriate compensatory measures will be taken, when the authorization expires, and that the State has been consulted and what, if any, comments were made by the state.), environmental assessment (EA) if applicable, and final no significant hazards consideration. Technical Specification pages shall accompany the SE, and, if appropriate, handwritten changes to the pages as submitted by the licensee shall be made.

In performing the review and preparing the documentation the PM shall: (1) obtain handwritten SER input from the ORAB and/or the cognizant DRL review branches, if appropriate, (2) consult with the resident inspector and/or appropriate regional personnel, and (3) request that all participating parties attempt to obtain their respective management's concurrence for input provided.

- The PM make a "best effort" via telephone to advise the state of the pending NRC action and to obtain state comments on the proposed NRC determination. The SE shall discuss this consultation and any state comments. The PM shall document the final no significant hazards finding (10 CRR 50.92) in the SE along with the environmental impact findings and conclusions. Finally, the PM shall complete and sign the emergency license authorization check off list. (See Attached)

- After completion of the SE, the PM shall obtain the concurrence of his Project Director and the cognizant Division Director. If the cognizant Division Director is not available concurrence may be obtained from one of the other NRR Division Directors. If no DDS are available, concurrence shall be referred to higher management. (Concurrences may be obtained verbally during non-duty hours) After obtaining the necessary concurrences, the PM shall contact the Region Branch Chief and the resident inspector and advise of the outcome.

The DD shall then contact the facility's licensing management or plant manager and verbally communicate the emergency license authorization. the PM shall ensure that the necessary information to characterize accurately the full extent and conditions of the licensee's request and the NRC authorization, is documented and understood by the licensee at the time of verbal authorization. This includes a handwritten SER/EA, a final NSHC, and the licensee's submittal including affected Technical Specification pages. (If concurrence is not obtained, the DD shall orally advise the licensee of the pending denial and, if time allows specify the criteria which must be satisfied in order to receive NRC authorizaton.) If approval is granted, the PM shall telecopy revised Technical Specification pages to the licensee and to the resident inspector.

Within two working days from DD oral authorization the PM shall insure that a follow-up license amendment, including a NSHC and post notice, is forwarded which provides the bases for NRC approval. The PM shall ensure that documentation is forwarded to the LPDR.

Enclosure 1

EMERGENCY LICENSE AUTHORIZATION

CHECK LIST

1. Complete submittal (Section III, Item 1) ☐

2. Prepare and sign handwritten SER, EA, final NSHC and Technical Specifications (Section III, Item 5) ☐

   a. ORAB or technical branch input ☐
   b. Resident or regional personnel input ☐

3. "Best effort" to obtain state comments (Section III, Item 6) ☐

4. Assistant Director concurrence (Section III, Item 7) ☐

5. Assistant Director oral authorization to licensee (Section III, Item 8) ☐

6. Telecopy Technical Specifications (Section III, Item 8) ☐

7. Forward final two day license amendment with post notice and FNSHC (Section III, Item 9) (Prepare DLOP 228, Attachment 4) ☐

Project Manager _____

Branch Chief _____

ORAB Branch Chief / Tech. Review Branch Chief* _____

ORAB AD / Tech. Review Branch AD* _____

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* To the extent practicable.

1:30 P.M. TECHNICAL SPECIFICATION IMPROVEMENTS

E. BUTCHER (NRC)

R. SGARRO (PP&L)

NRR TECHNICAL SPECIFICATIONS IMPROVEMENT PROGRAM

PHASE I

Problem Identification and Recommendations, TSIP Report

PHASE II

Implementation, TSCB

TECHNICAL SPECIFICATIONS COORDINATION BRANCH (TSCB)

- FUNCTIONS

  - Implementation of TSIP Recommendations
  - General Oversight of All NRR Technical Specifications Activities
  - Generic(i.e., Non-Vendor Specific) Interpretations

- ORGANIZATION - PROJECT TEAMS

W	- Tom Dunning/Dave Langford
GE	- Kulin Desai
CE	- Sam Bryan/Millard Wohl
B&W	- Sam Bryan/Millard Wohl

NRC TECHNICAL SPECIFICATIONS

IMPROVEMENT PROGRAM PLAN OUTLINE

1.0 INTRODUCTION (PROGRAM OBJECTIVES)

- POLICY STATEMENT ON TS SELECTION CRITERIA AND NEW STS.
- SHORT TERM IMPROVEMENTS TO EXISTING STS.

2.0 DEVELOPMENT OF NEW STS (BASED ON POLICY STATEMENT).

3.0 SHORT TERM IMPROVEMENTS TO EXISTING STS.

4.0 OTHER STS IMPROVEMENT ACTIVITIES.

5.0 COORDINATION AND POLICY STATEMENT SCHEDULE.

EXECUTIVE SUMMARY

NRC TECHNICAL SPECIFICATIONS

IMPROVEMENT PROGRAM PLAN

MARCH 28, 1986

Prepared by:

Technical Specifications Coordination Branch

Division of Human Factors Technology, NRR

TABLE OF CONTENTS

EXECUTIVE SUMMARY

TECHNICAL SPECIFICATIONS IMPROVEMENT

PROGRAM PLAN

EXECUTIVE SUMMARY
TECHNICAL SPECIFICATIONS IMPROVEMENT PROGRAM PLAN

1.0 INTRODUCTION

In the past several years the nuclear industry and the NRC staff have been studying the question of whether improvement to the current system of establishing Technical Specification (TS) requirements for nuclear power plants is needed. The two most recent studies of this issue were performed by an NRC task group known as the Technical Specifications Improvement Project (TSIP) and a Subcommittee of the Atomic Industrial Forum's Committee on Reactor Licensing and Safety.[1] The overall conclusion of these studies was that many improvements in the scope and content of Technical Specifications are needed, and that a joint NRC and Industry program should be initiated to implement these improvements. Both of these groups made specific recommendations which are summarized as follows:

1) The NRC should adopt the criteria for defining the scope of TS proposed in the AIF and TSIP reports. Those criteria should then be used by the NRC and each of the Industry Owners Groups to completely rewrite/streamline the existing Standard Technical Specifications (STS). This process would result in many requirements being transferred from control by Technical Specifications to control by other mechanisms (e.g., the FSAR, Operating Procedures, QA Plan) which would not require a license amendment or prior NRC approval when changes are needed. The new STS would also include greater emphasis on human factors principles to add clarity and understanding to the overall text and Bases Section.

2) A parallel program of short term improvements in both the scope and substance of the existing TS should be initiated in addition to developing a new set of STS as identified in 1) above.

The purpose of this Executive Summary of the Program Plan is to outline the specific set of activities to be performed by the industry and the NRC aimed at the practical implementation of these recommendations.

This document is structured so as to link specific activities under the program with the two major objectives embodied in the TSIP and AIF recommendations summarized above. As such, Section 2.0 below is devoted to the development of a new set of STS while Section 3.0 is devoted to implementing shorter term improvements to the existing STS. Section 4.0 describes other general activities necessary to support the overall program. And finally, Section 5.0 summarizes the schedule of activities for the issuance of the Commission Policy Statement on Technical Specifications.

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[1] "Recommendations for Improving Technical Specifications," NRC Technical Specification Improvement Project, September 30, 1985.
"Technical Specification Improvements," AIF Subcommittee on Technical Specifications Improvements, October 1, 1985.

2.0 DEVELOPMENT OF NEW STS

The first priority of this Program Plan shall be the development of a Commission Policy Statement to establish a specific set of objective criteria for determining which regulatory requirements and operating restrictions should be included in TS.

2.1 Trial Use of AIF and TSIP Criteria

Before the staff can recommend that the Commission issue a Policy Statement based on the TSIP and AIF criteria, these criteria must be validated (i.e., shown to be technically adequate and practical to implement). The validation process will be through a trial use of the criteria on actual operating reactor TS.

	Activities	Schedule Goals
1.	AIF and NRC separately applied the criteria to Wolf Creek and Limerick TS. Limiting Conditions for Operation and associated Surveillance Requirements, were evaluated against the criteria.	Completed - 02/18/86
2.	AIF and NRC met to discuss the results of the trial application of the criteria. Areas of agreement and disagreement were discussed and differences resolved where possible. Remaining defects in the criteria or changes needed to improve clarity were summarized.	Meetings Completed Wolf Creek-01/28/86 Limerick-02/26/86 Report Issued-03/21/86
3.	NRC RRAB will perform an evaluation of the risk significance of the systems or components with LCOs that would be removed from the TS and currently require a power reduction or shutdown. If the criteria result in LCO's with major risk significance being removed from the TS, then changes to the criteria will be proposed.	Started-03/10/86 Finish-04/30/86
4.	The results of 2 and 3 above will be used for modifying or clarifying the criteria, as needed. The final criteria developed through this process will be included in the Policy Statement discussed in Section 2.2 below.	Start-In Parallel with 2 and 3 above. Finish-04/30/86

2.2 Develop Commission Policy Statement

The second step in developing the new STS is to issue a Policy Statement which defines the scope, purpose, and content for Technical Specifications. The core of this Policy Statement will be the TS selection criteria validated by the process outlined in Section 2.1 above.

	Activities	Schedule Goals
1.	TSCB, with the support of ELD and other NRC staff will draft a Policy Paper recommending that a Notice of Proposed Policy Statement be issued for public comment stating the Commission's intent to establish a specific set of objective criteria for determining which regulatory requirements and operating restrictions should be included in TS. The Policy Paper will include a discussion of all the issues listed in Appendix A which were identified in the Commission's Staff Requirements memorandum dated 02/21/86. Withdrawal of the earlier proposed rule change for 10 CFR 50.36 would be included in this Notice.	Started-03/24/86 Finish-04/30/86 First Draft Issued
2.	The first draft Policy Paper will be circulated for review and comment to each of the NRC Program Offices, Regional Offices and NRR Divisions. After comments from all groups have been considered and appropriate changes made, a second draft will be issued for ACRS review.	Start-05/01/86 Finish-07/03/86 Second Draft Issued.
3.	The second draft Policy Paper will be presented to the ACRS. Any changes necessary will be made and a final draft paper prepared and forwarded to CRGR.	Start-07/07/86 Finish-08/01/86 ACRS Review Complete and Policy Paper Forwarded to CRGR.
4.	The final draft paper will be presented to CRGR for review and approval. Any required changes will be made and the Policy Paper will be forwarded to the Commission.	Start-08/04/86 Finish-08/29/86 CRGR Review Complete and Policy Paper Forwarded to Commission.
5.	The staff will, at the Commission's option, make a presentation to the Commission on the Notice of Proposed Policy Statement and make changes directed by the Commission prior to publishing the Notice for public comment.	Start-09/02/86 Finish-10/17/86 Notice of Proposed Policy Statement Issued.

6. Public comments received in response to the Notice will be reviewed and addressed in a second Policy Paper for the Commission proposing a final Policy Statement on Technical Specifications Improvement. Following Commission approval the Policy Statement will be issued.	Start-11/13/86 Comments received. Finish-01/30/87 Policy Statement Issued.

2.3 Improvements to TS Text and Bases Sections

In addition to culling out the less important requirements in the existing STS by applying the selection criteria discussed in Sections 2.1 and 2.2 above, a major objective of the TS Improvement Program is to, through the application of human factors principles, add clarity to the TS. These types of changes represent one of the primary safety benefits to be achieved from the program. NRC and Industry activities will include the development of a Standard Format and Content Guide for TS text and Bases. This work will be completed and available for use in preparing the new STS discussed below.

2.4 Industry Preparation/Submittal of New STS

The primary instrument to be used for achieving the desired improvement in TS will be a new set of STS based on selection criteria to be defined in a Commission Policy Statement. It is expected that the Industry, through the individual owners groups, will take the lead in preparing the new STS and submitting them in a Topical Report. The details of this process and a schedule for submittals have not yet been worked out with the Industry, however, the objective is for Industry to develop and issue the new STS and any subsequent revisions. The NRC role would be limited to review and approval.

2.5 NRC Review/Approval of New STS

A schedule goal of six months from the date of submittal has been established for completion of the staff's review. The bases for the staff's review will be the guidelines established in the Commission Policy Statement and the guidance developed under subsection 2.3 above.

2.6 Plant Specific Implementation of New STS

Plant specific implementation of the new STS is not considered a part of the Program Plan. TSCB will, however, in conjunction with Industry, develop guidelines for the contents of the individual licensee amendment submittals necessary to convert to the new STS. A likely requirement for the submittal package will be some document or method to identify how each requirement removed from the TS would be controlled after the license is amended. The effective date of the amendment would be specified to allow time for any required changes in the licensees procedures and administrative controls.

3.0 SHORT TERM IMPROVEMENTS TO EXISTING STS

There is mutual agreement between NRC and Industry that many short term improvements in the current STS should be made in parallel with the longer term plan to develop new STS as discussed in Section 2.0 above. These improvements are needed to resolve recurring problems with certain technical and administrative requirements in operating plant TS. These issues are of minor safety significance, but their resolution requires a considerable amount of NRC staff and Industry resources. The general approach for making these types of changes will be to revise specific requirements in the existing STS, issue a Generic Letter with the revised STS enclosed, and then process individual operating reactor license amendment requests based on the Generic Letter.

In order to expedite the review process so that short term improvements can be implemented as soon as possible, two parallel paths for developing and processing the STS changes have been established. The first path is through TSCB and the second is through the three NRR Licensing Divisions. The types of improvements that would follow each of these paths are discussed in Subsections 3.1 and 3.2 below. Regardless of which path is followed, the actual change to the STS and the Generic Letter implementing it would be prepared by TSCB.

3.1 Short Term STS Improvements to be Developed by TSCB

As a general rule, short term STS improvements which are applicable to all plants without regard to vendor design, e.g., fire protection, general requirements applicable to limiting conditions for operation and surveillance requirements, and administrative control requirements, will be developed by TSCB. These types of changes can be initiated and developed by the staff without significant additional input from the Industry. TSCB will develop the changes, coordinate NRR and CRGR approval (where required), and prepare a Generic Letter for notifying licensees of approved STS changes.

3.2 Short Term STS Improvements to be Developed by the NRR Licensing Divisions

The review and development of vendor specific short term STS improvements will be the responsibility of the applicable NRR Licensing Division. In addition certain other generic (i.e., applicable to more than one vendor design) changes will also be developed by the Licensing Divisions. The types of changes that will be handled by the Licensing Divisions are generally initiated by the Industry and must be reviewed by a technical specialist branch within the NRC.

Two types of submittals to the Licensing Divisions have been designated by the staff for use by the Industry in initiating staff action on these types of short term improvements. The first type is a Topical Report to justify changes to the Allowed Outage Times (AOTs) and Surveillance Intervals (SIs) associated with STS requirements. The second type of submittal which will initiate an NRR Licensing Division review is a plant specific license amendment which has been endorsed by the Industry (e.g., an Owners Group) as a candidate for consideration under the Technical Specifications Improvement Program.

4.0 OTHER STS IMPROVEMENT ACTIVITIES

The main focus of both the NRC and Industry Technical Specification improvement activities discussed above has been on the LCOs in Section 3 of the STS. However, consideration will be given to the need for improvements to the other STS sections, particularly Sections 5.0 and 6.0. AIF recommended rule changes and the relocation of surveillance requirements to other controlled documents will also be considered.

Another area related to STS improvements is the TSIP and AIF recommendations for continued development and application of probabilistic risk assessment (PRA) methods to address TS requirements. And finally, policy guidance for selecting appropriate controls on requirements which will be transferred from the control of the current STS, through the application of the selection criteria discussed in Subsection 2.1, needs to be established. This policy guidance must be established before a new set of STS can be approved by NRC.

4.1 Improvements to Sections 5.0 and 6.0 of STS

The Policy Statement will only establish selection criteria for LCOs. Improvements to the Design Features and Administrative Controls sections will be developed by TSCB and incorporated into the existing STS as short term improvements.

4.2 Rule Changes

AIF recommended that NRC initiate rulemaking to codify the criteria for TS requirements in place of the current requirements of 10 CFR 50.36. In addition, several changes in the regulations referencing Technical Specification were recommended to conform with the new STS requirements (e.g., 50.36 on RETS, Part 50 Appendices I, J, K, H, and R on duplicate or overlap TS requirements). Proposed rule changes will be developed by TSCB with input from the Licensing Divisions. A major rule change to codify the criteria will not be initiated until some experience using the criteria under a Policy Statement has been gained.

4.3 Surveillance Requirements

AIF recommended that surveillance requirements for items listed in the new STS should be relocated to documents not controlled by the license amendment process. Further, it was recommended that the details associated with surveillance, frequency and methodology, may be more effectively controlled by a program with an appropriate administrative control process. TSCB will work with Industry to develop the justification for a change in the process by which surveillance requirements are addressed in TS.

4.4 PRA Methods for STS Improvements

The NRC Office of Research is developing a Procedure for Evaluating Technical Specifications (PETS) which addresses PRA methods to evaluate changes to AOTs and SIs. Guidance on this subject is needed to facilitate Licensee's preparation of changes that are based on risk assessments. TSCB will interface with RES on the results of the PETS program which will be used to provide guidance to Industry and the staff on PRA methods for evaluating changes to Technical Specifications.

4.5 Controls for Requirements Transferred from the Control of the TS

Various mechanisms exist which can be used to control those requirements which would be removed from the TS when the proposed selection criteria are applied. There is a need to establish guidance for determining which controls are appropriate for particular requirements based on their safety significance. TSCB will develop and issue this guidance with input from Industry.

5.0 COORDINATION, AND POLICY STATEMENT SCHEDULE

5.1 Coordination

TSCB will be responsible for managing and coordinating all NRC activities within the scope of the Program Plan and will serve as the point of contact at the NRC for all Industry related activities with the exception of the specific short term STS improvements to be developed by the NRR Licensing Divisions (see Subsection 3.2). The Industry will work directly with the Licensing Divisions on these specific short term improvements with the TSCB role being only coordination to assure consistency with the overall objectives of the improvement program.

5.2 Policy Statement Schedule

The schedule for the activities related to the issuance of a Commission Policy Statement on TS Improvements is provided in in Appendix B.

APPENDIX A

ISSUES RAISED IN THE COMMISSION'S STAFF

REQUIREMENTS MEMORANDUM DATED FEBRUARY 21, 1986

a. Whether implementation of the Policy should be backfitted, forwardfitted or both?

b. If the Program is to be voluntary, how can, or should the NRC encourage participation by individual licensees?

c. Is the 10 CFR 50.109 Backfit Rule applicable?

d. Should the Policy Statement be codified by a change to 10 CFR 50.36, and if so, on what time schedule (perhaps after some trial use with the Policy Statement)?

e. Whether the Policy Statement should be applicable to custom TS or should licensees wishing to take advantage of the program be required to convert to STS?

f. Are the control mechanisms available for those items that would be removed from the TS adequate (e.g., 10 CFR 50.59)? If not, what changes are necessary?

g. What are the NRC resource impacts in terms of both the initial implementation of the New STS and any additional staff actions related to a greater reliance on 10 CFR 50.59 or plant procedures for control?

h. What are the risk implications of the proposed criteria? Can the risk impact of the resultant changes in TS under the criteria be quantified and if so, what is the effect? To what extent does the application of the criteria increase the uncertainty in current estimates of risk?

i. What would be the effect of implementing of the proposed criteria on the amount of testing at power that is currently required? How does this compare to the current testing practices of other countries? To what extent can any differences that will exist be attributed to differences in design (e.g., greater redundancy and diversity of safety systems) or preventive maintenance programs?

APPENDIX B
POLICY STATEMENT SCHEDULE

POLICY STATEMENT FOR NEW STS — PROGRAM PLAN ACTIVITIES — SECTION 2.0

REPORT TYPE :PERIOD BARCHART — PRINTING SEQUENCE :Most Critical Activities First

SELECTION CRITERIA :ALL

PLAN I.D. :EXECSUMM VERSION 1 — TIME NOW DATE :10/MAR/86

	1986											1987						
PERIOD COMMENCING DATE	10	7	5	2	7	4	1	6	3	1	5	2	2	6	4	1	
MONTH	MAR	APR	MAY	JUN	JUL	AUG	SEP	OCT	NOV	DEC	JAN	FEB	MAR	APR	MAY	JUN	
PERIOD COMMENCING TIME UNIT	258	278	298	318	343	363	383	408	428	448	473	493	513	538	558	578	
13 Criteria Risk Impact (2.1.3)	CCCC	CCCC															
14 Finalize Criteria (2.1.4)	CCCC	CCCC															
20 Policy Paper Draft #1 (2.2.1)		CC	CCCC														
21 Internal Consent Period (2.2.2)			C	CCCC													
22 Policy Paper Draft #2 (2.2.2)				CCCCC													
23 ACRS Briefing/Review (2.2.3)					C												
24 Policy Paper Final Draft (2.2.3)					CCC												
25 CRGR Presentation (2.2.4)						CCCC											
26 Commission Briefing/Review (2.2.5)							CCCC										
27 Notice of Policy Statement (2.2.5)								CC									
28 Public Consent Period (2.2.6)									CC	CCCC	C						
29 Finalize Policy Statement (2.2.6)										CCC	CCCC						
30 Final Commission Approval (2.2.6)											C	CCC					
31 Issue Policy Statement (2.2.6)												C					

Barchart Key:- CCC :Critical Activities *** :Non Critical Activities NNN :Activity with neg float ... :Float

BWROG TECHNICAL SPECIFICATIONS COMMITTEE

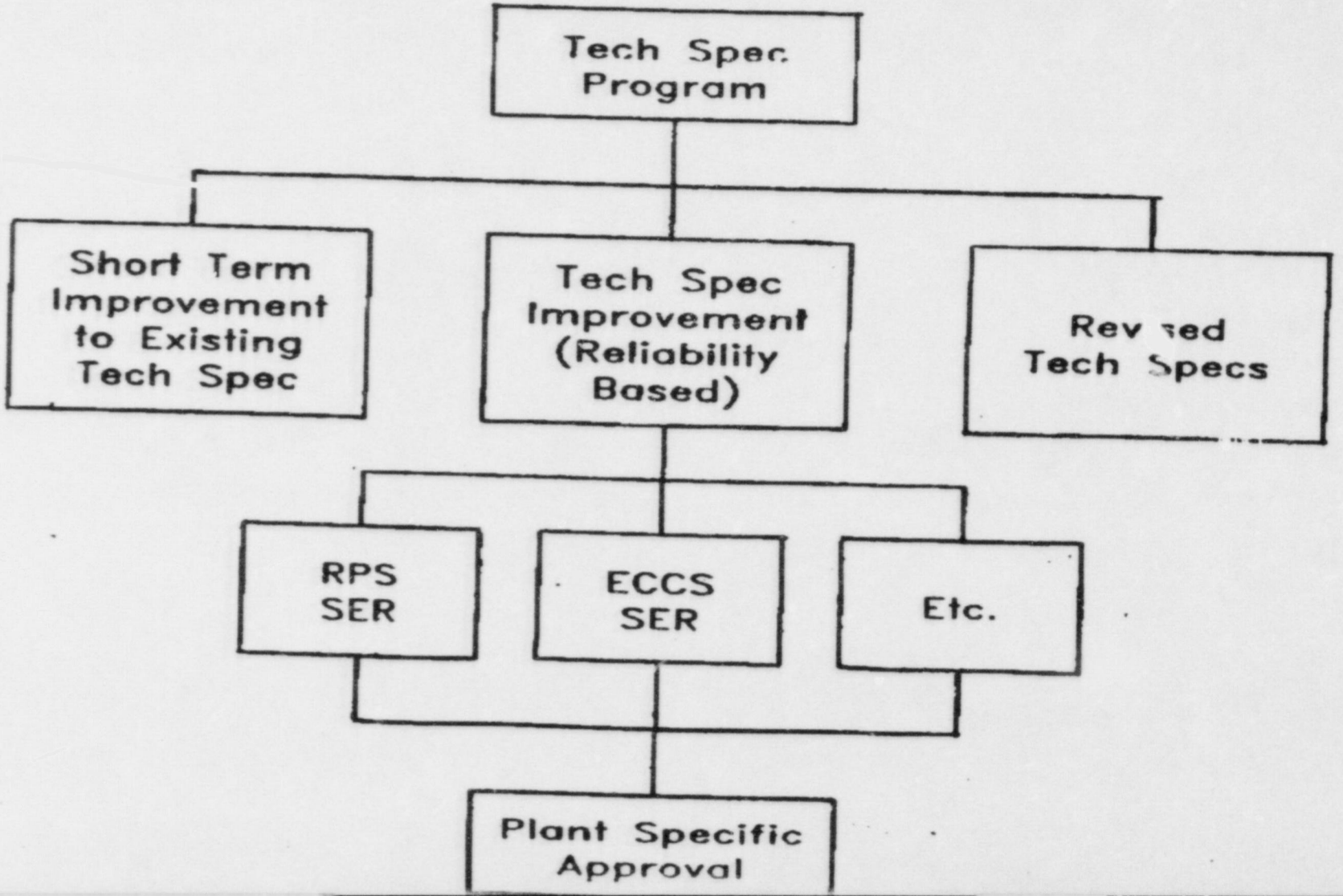
- FORMED: JANUARY 1986
- CHAIRMAN: R. E. BRADLEY, GPC
- 22 PARTICIPATING DOMESTIC UTILITIES

BECO	MP&L
CP&L	NMPC
CEI	NPPD
CECO	NSP
DECO	NYPA
GPC	PECO
GPUN	PP&L
GSU	PSE&G
IEL&P	TVA
IP	VYNPC
LILCO	WPPSS

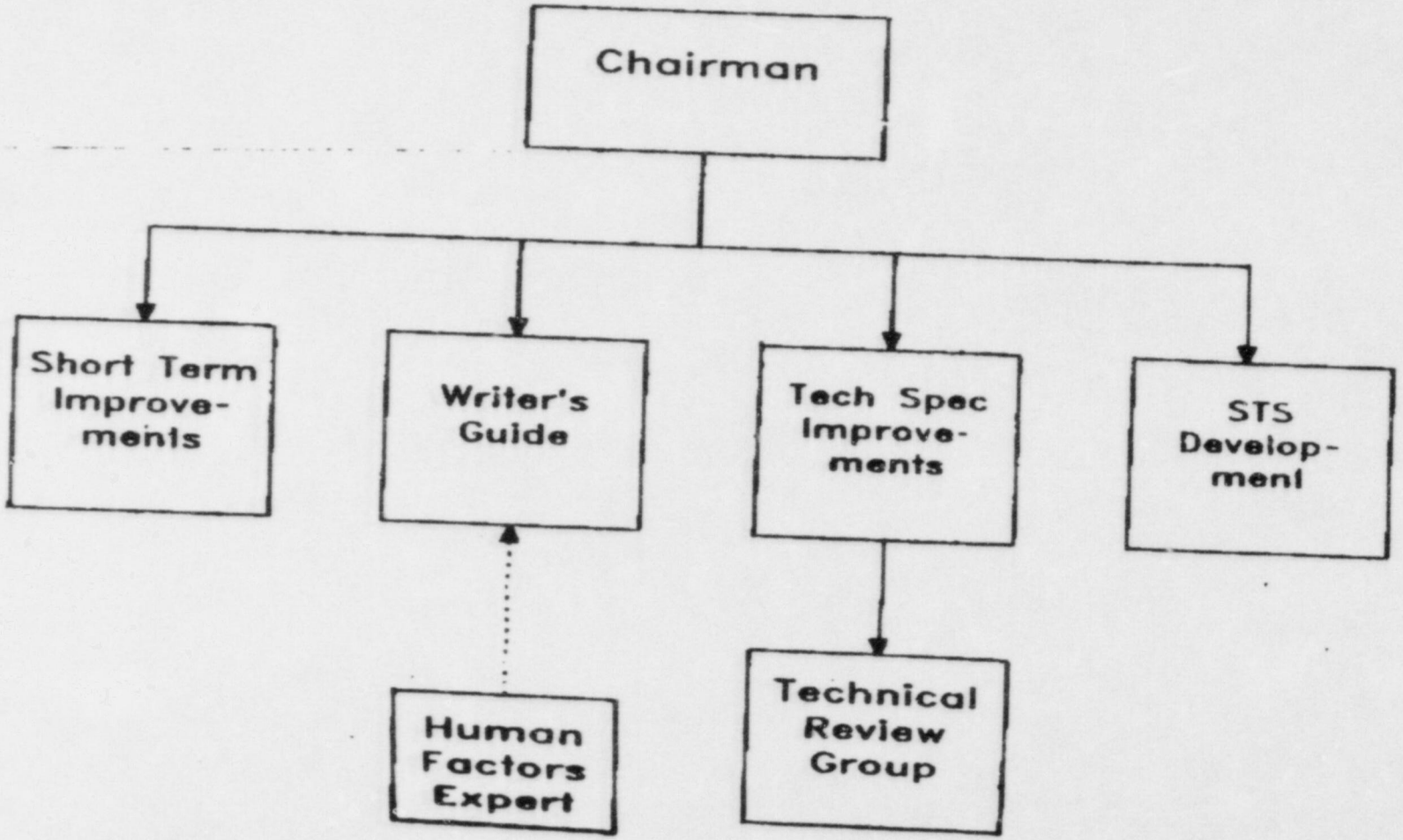
- GENESIS OF ISSUE
  - UTILITY INTEREST IN DEVELOPING NEW TECHNICAL SPECIFICATIONS
  - UTILITY INTEREST IN OPTIMIZING EXISTING TECH SPECS
  - NEED FOR BWROG TO PARTICIPATE IN INDUSTRY TECH SPEC ACTIVITIES
  - NEED TO COORDINATE BWROG TECH SPEC ACTIVITIES

Rocky Sgarro (PP&L)
September 9, 1986

BWROG TECH SPEC PROGRAM



BWROG TSC ORGANIZATION



SHORT TERM IMPROVEMENTS-CATEGORIES

CATEGORY I: NRC APPROVED ON AN INDIVIDUAL PLANT DOCKET.

- O DO NOT REQUIRE LEAD PLANT SPONSORSHIP
- O TSC RESOLVES NRC COMMENTS FROM GENERIC REVIEW OF PREVIOUS APPROVAL
- O NRC ISSUES GENERIC LETTER

CATEGORY II: SUBMITTED TO NRC ON AN INDIVIDUAL PLANT DOCKET, BUT NOT YET APPROVED.

- O FOLLOW LEAD PLANT REVIEW PROCEDURE

CATEGORY III: NOT YET SUBMITTED FOR NRC REVIEW.

- O FOLLOW LEAD PLANT REVIEW PROCEDURE

ATTACHMENT
BWROG SHORT TERM TECHNICAL SPECIFICATION
IMPROVEMENTS

CATEGORY	ITEM	REFERENCE DOCUMENT
I.	Increase Relief Valve Setpoint Tolerance to Coincide with Design Specification (±13psi)	MP&L, Amendment 9 to License No. NPF-13 (Sept. 15, 1983)
I.	Snubbers - Reduction in Functional Retest Requirements from 10% to 5%	Fermi 2 Full Power License #NPF-43, July 15, 1985
I.	Integrated Leak Rate Test Duration Decrease from 24 to 8 hours.	NMPC, Amendment 52 to License No. DPR-63 (May 4, 1983)
I.	Relaxation of Shutdown Requirements Associated with H_2/O_2 Analyzers	IE&P, Amendment 134 to License No. DPR-49 (July 21, 1986)
I.	Deletion of Isolation Actuation Instrumentation Response Times	(CE) NUREG 1162, March 1986
II.	Deletion of Primary Containment Isolation Valve Listing	
II.	Scram Discharge Volume (SDV)/ Vent and Drain Valve 30-Second Closure Time Elimination	
III.	Diesel Generator Testing Requirements Relaxation	
III.	Accident Monitoring Instrumentation Allowable Out-of-Service Times Relaxation	
III.	Area Temperature Monitoring Requirements Relaxation	
III.	SDV Testing - Eliminate Requirement to Perform 50% Rod Density Scram Surveillance Test	

TECHNICAL SPECIFICATION IMPROVEMENTS WORKING GROUP

SCOPE - RELIABILITY BASED ANALYSIS FOR IMPROVEMENTS IN RPS AND ECCS INSTRUMENTATION TECH SPECS

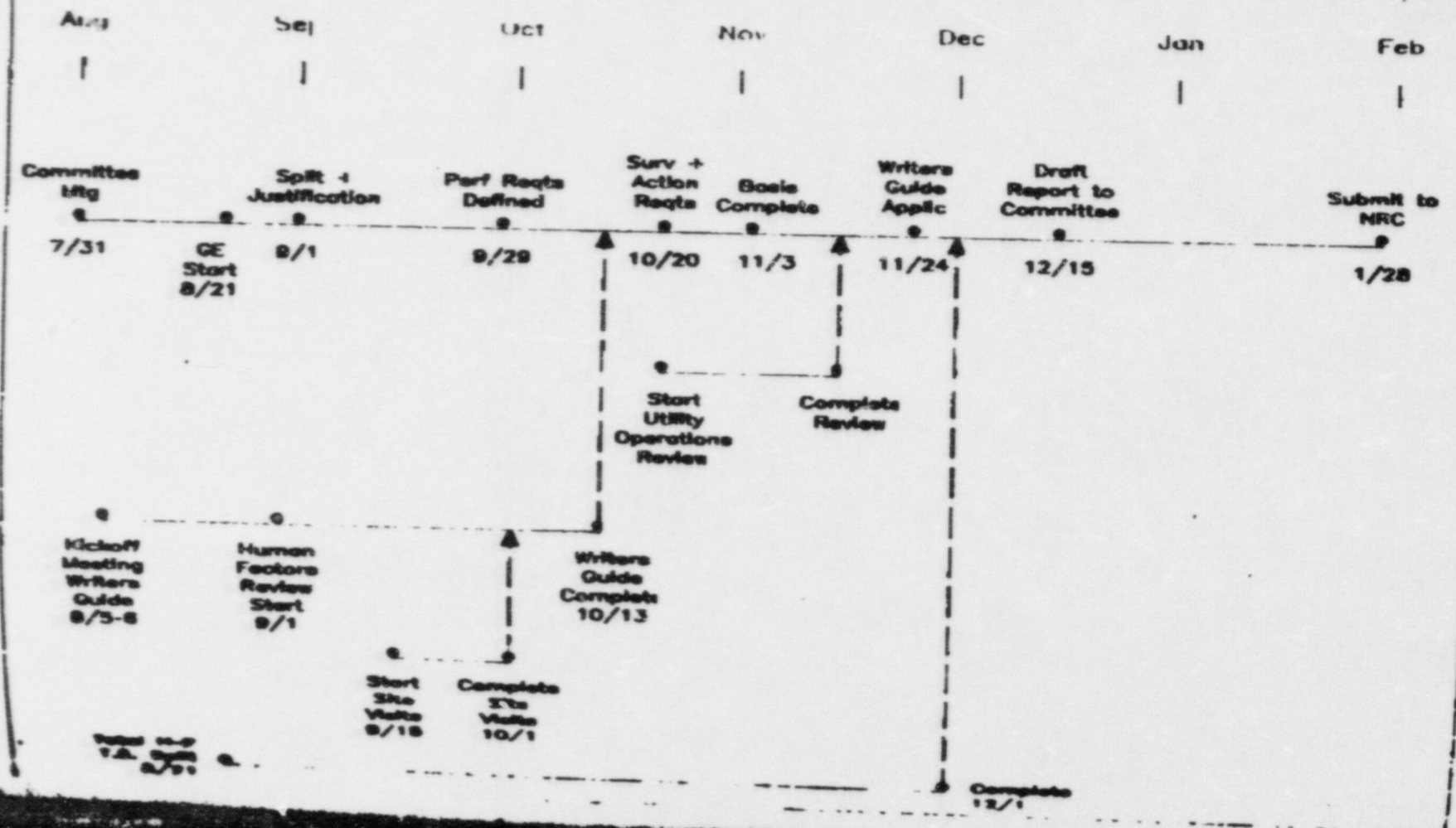
DELIVERABLES

Submittal Date	Report	Review Status
January 1985	NEDC 30844, BWR Owners' Group Response to NRC Generic Letter 83-28, Item 4.5.3	NRC technical review complete SER delayed due to staggered testing issue.
May 1985	NEDC-30851P, BWR Owners' Group Technical Specification Improvement Analysis for BWR Reactor Protection Systems	NRC Technical Review Complete for relay plants. SER delayed due to staggered testing issue. Solid state plant evalution currently under technical review.
November 1985	NEDC-30936P, Part 1 BWR Owners' Group Technical Specification Improvement Methodology (with Demonstration for BWR ECCS Actuation Instrumentation	Currently under review by NRC/ Brookhaven. SER expected in Nov. 1986.
June 1986	NEDC-30851P, Supplement 1, Technical Specification Improvement Analysis for BWR Control Rod Block Instrumentation	NRC is currently preparing contract for BNL review.
August 1986	NEDC-30851P, Supplement 2, Technical Specification Improvement Analysis for BWR Isolation Instrumentation Common to Reactor Protection System and ECCS	
September 1986 (Planned)	NEDC 30936P, Part 2 BWR Owners' Group Technical Specification Improvement Methodology (with Demonstration for BWR ECCS Actuation Instrumentation)	

POTENTIAL ADDITIONAL ACTIVITIES

- SUPPORT NRC REVIEWS
- DEVELOP UTILITY MANUAL
- HOLD TRAINING SESSION
- EVALUATE ADDITIONAL RELIABILITY-BASED SPEC IMPROVEMENTS FOR TECH SPEC COMMITTEE LONG-RANGE PROGRAM

Proposed Schedule
Development Of A New Section 3/4.1



TECHNICAL SPECIFICATION IMPROVEMENTS SUBCOMMITTEE

- REPORTS TO THE COMMITTEE ON REACTOR LICENSING AND SAFETY
- CHAIRMAN: ALAN PASSWATER, SUPERINTENDENT OF LICENSING, UNION ELECTRIC COMPANY
- PURPOSE: COORDINATE, PROPOSE AND PROMOTE TECHNICAL SPECIFICATION IMPROVEMENTS
- MEMBERSHIP: REPRESENTATIVES FROM:

  CHAIRMAN OF EACH OF THE FOUR VENDOR OWNERS GROUPS TECHNICAL SPECIFICATION SUBCOMMITTEES

  CHAIRMAN, NUCLEAR POWER PLANT STANDARDS WORKING GROUP, ANS-58.4

  NINETEEN UTILITIES

  EPRI

  FOUR VENDORS

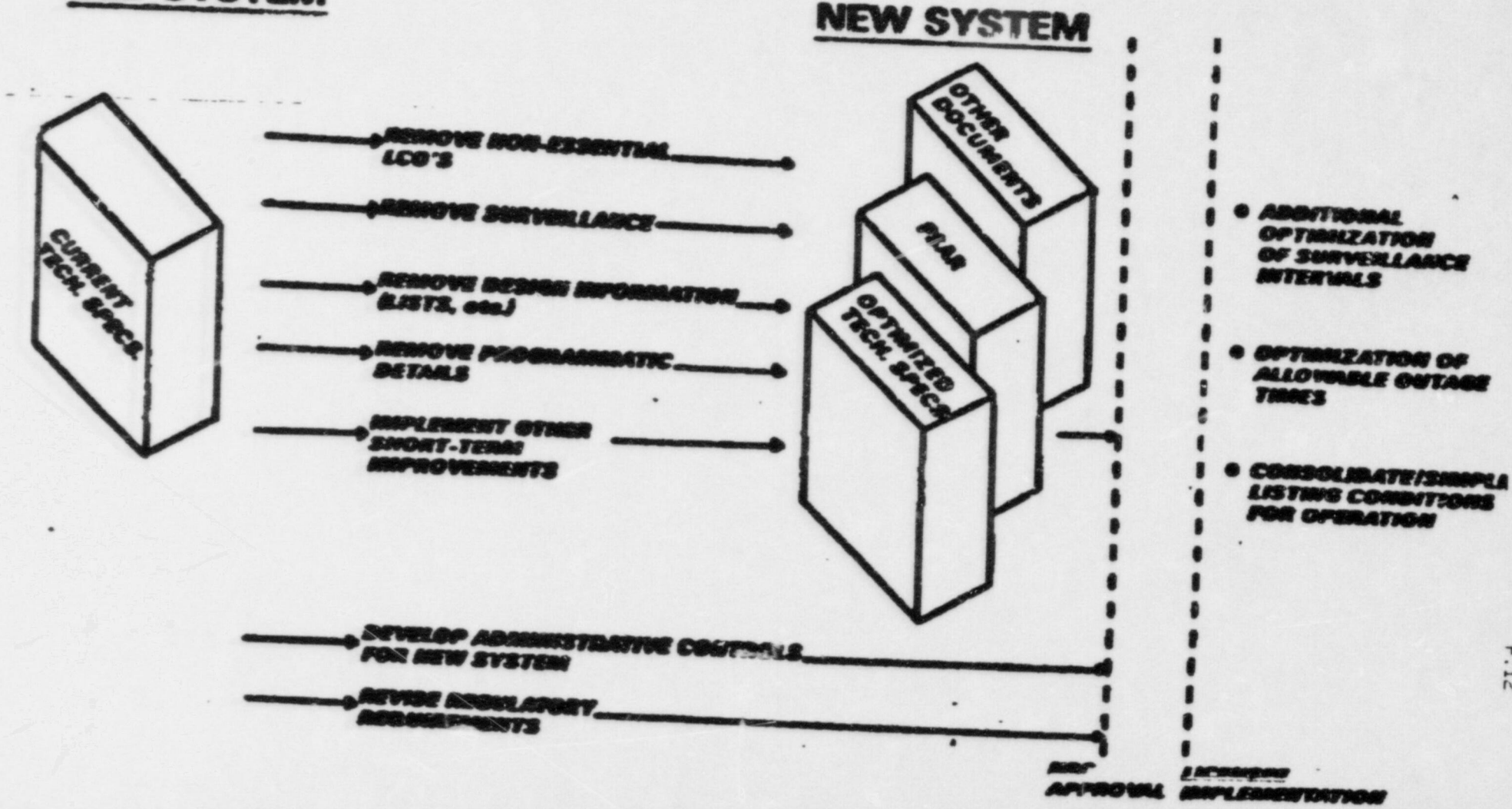
  THREE ARCHITECT ENGINEERING FIRMS

  TWO CONSULTING FIRMS

ESTABLISHED WORKING GROUPS

- WORKING GROUP ON CRITERIA DEVELOPMENT
- WORKING GROUP ON THE ADMINISTRATIVE PROCESS
- WORKING GROUP ON REGULATORY CHANGES
- WORKING GROUP ON PROBABILISTIC METHODOLOGY

PLAN TO REFORM NRC REQUIREMENTS RELATED TO THE TECHNICAL SPECIFICATIONS



3:~~00~~ 15 P.M. SEVERE ACCIDENT POLICY/DEGRADED CORE PROGRAM

R. BERNERO (NRC)

SAFETY OBJECTIVES

- THE LIKELIHOOD OF SEVERE ACCIDENT (CORE DAMAGE OR CORE MELT) SHOULD BE VERY LOW

AND

- IF A SEVERE ACCIDENT OCCURS THERE SHOULD BE SUBSTANTIAL ASSURANCE THAT THE CONTAINMENT WILL MITIGATE ITS CONSEQUENCES

CONTAINMENT ISSUES

- EARLY REACTORS
  - LOW POWER/BIG CONTAINMENTS
  - COULD MEET CONTAINMENT OBJECTIVE

- EVOLUTION OF DESIGN
  - MUCH HIGHER POWER
  - FOCUS ON PREVENTION OBJECTIVE
  - CONTAINMENT GOOD FOR FISSION PRODUCTS BUT QUESTIONS ABOUT HEAT AND GAS

- REACTOR SAFETY STUDY (1975)
  - BIGGER REACTORS
  - 1 PWR (SURRY)
  - 1 BWR (PEACH BOTTOM)
  - BWR RESULTS INDICATED LOWER PROBABILITY BUT POOR CONTAINMENT

US BWRS

- 2 SMALL UNITS WITH LARGE CONTAINMENTS
- 24 BWR 2/3/4 WITH MARK I CONTAINMENT (ALL LICENSED)
- 9 BWR 4/5 WITH MARK II CONTAINMENT (7 LICENSED)
- 4 BWR 6 WITH MARK III CONTAINMENT (3 LICENSED)

BWR CONTAINMENT IN SEVERE ACCIDENTS - SINCE TMI

- TMI ACTION PLAN I.C
  - LETTERS OF SEPT-NOV, 1979 ON PROCEDURES
  - BWR EPG, REV 1, REV 2, REV 3, REV 4

- IDCOR ANALYSIS
  - IDCOR FOUNDED DECEMBER 1980
  - STILL DELIBERATING ANALYSIS WITH NRC

- NRC/CONTRACTOR ANALYSIS
  - SOURCE TERM STUDIES
  - SARRP - WHAT WILL NUREG-1150 SAY?
  - BNL GUIDELINES AND CRITERIA

- CHERNOBYL
  - UNIT 4 HAD PRESSURE SUPPRESSION CONTAINMENT FEATURES - A STRIKING RESEMBLANCE?

WHAT ARE THE PROBLEMS
AND SOLUTIONS

- 5 ELEMENTS TO CONSIDER
  - HYDROGEN
  - SPRAYS
  - PRESSURE
  - CORE DEBRIS
  - TRAINING & PROCEDURES

- MANY CHANGES ARE ALREADY IN PLACE

- FINAL IMPROVEMENTS ARE NOW UNDER HIGH PRIORITY STUDY
  - GENERIC ACTION WITH IDCOR AND BWROG
  - GENERIC WORK BY NRC
  - VERMONT YANKEE STUDY
  - PILGRIM PROGRAM

CHRONOLOGY

- JUNE 16, 1986: MEETING WITH BWROG/IDCOR PROPOSED A GENERIC LETTER, PRESCRIPTIVE SOLUTION, BY BACKFIT
- JUNE 30, 1986: VERMONT YANKEE COMMITS TO GOV. KUNIN TO DO A SPECIAL 60-DAY CONTAINMENT STUDY
- JULY 25, 1986: BOSTON EDISON COMPANY BOARD DECIDES TO FIX PILGRIM CONTAINMENT
- AUGUST 19, 1986: BWROG EXECUTIVES VOTE TO FUND AND CONTINUE DIALOGUE ON THIS WITH NRC, CONTACT NUMARC ABOUT BWR VS. PWR
- SEPTEMBER 11, 1986: MEETING WITH BWROG TO COMPARE BACKFIT NOTES AND STRAWMAN GENERIC REQUIREMENTS
- SEPTEMBER 11, 1986: MEETING WITH VERMONT YANKEE TO REVIEW CONTAINMENT STUDY
- SEPTEMBER 23, 1986: NRC/IDCOR MEETING ON BWR/MARK I ANALYSES
- SEPTEMBER 23, 1986: ACRS SUBCOMMITTEE ON CONTAINMENT PERFORMANCE TO DISCUSS HARPERS FERRY WORKSHOP RESULTS AND BWR CONTAINMENT GENERIC APPROACH
- SEPTEMBER 24, 1986: ACRS SUBCOMMITTEE ON CLASS 9 ACCIDENTS TO DISCUSS BWR/MARK I ANALYSES AND SEVERE ACCIDENT PROGRAM
- NOVEMBER 19, 1986: CRGR REVIEW OF DRAFT GENERIC LETTER ON BWR CONTAINMENT REQUIREMENTS (TO BE PUBLISHED FOR COMMENT)
- DECEMBER 17, 1986: ISSUE DRAFT GENERIC LETTER ON BWR CONTAINMENT REQUIREMENTS FOR PUBLIC COMMENT
- APRIL 1987: ISSUE FINAL GENERIC LETTER ON BWR CONTAINMENT REQUIREMENTS