NUREG-0020
Vol. 10, No. 8
August 1986

LICENSED OPERATING REACTORS

STATUS SUMMARY REPORT

DATA AS OF 07-31-86

UNITED STATES NUCLEAR REGULATORY COMMISSION



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OFFICE OF INFORMATION RESOURCES MANAGEMENT
U.S. NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555



STATEMENT OF PURPOSE

The U.S. Nuclear Regulatory Commission's monthly LICENSED OPERATING REACTORS Status Summary Report provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Information Resources Management, from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices, IE Headquarters and the Utilities; and the third section is an appendix for miscellaneous information such as spent fuel storage capability, reactor years of experience and non-power reactors in the United States.

The percentage computations, Items 20 through 24 in Section 2, the vendor capacity factors on page 1-7, and actual vs. potential energy production on Page 1-2 are computed using actual data for the period of consideration. The percentages listed in power generation on Page 1-2 are computed as an arithmetic average. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U.S. energy situation as a whole.

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G L O S S A R Y

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours, divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (NET MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an offnormal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.

HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g., turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilization. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous minth, report the same outage date but change "Method of Shutting Down Reactor" to "4 (continuations)" and add a note: "Continued from previous month."
OUTAGE DURATION	The Total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Average Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.

REACTOR AVAILABLE HOURS	The Total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours} \times 100}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period that the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical} \times 100}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calender month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)

UNIT AVAILABILITY FACTOR	$\frac{\text{Unit Available Hours} \times 100}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
- Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated} \times 100}{\text{Period Hours} \times \text{Lic. Thermal Power}}$
- Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{Nameplate Rating}}$
- Using DER	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{DER}}$
- Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Gross}}$
- Using MDC Net	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Net}}$

NOTE: if MDC GROSS and/or MDC NET have not been determined, the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours} + \text{Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours} \times 100}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

NOTE:

At the end of each statement in the Enforcement Summary for any given facility may be found numbers in parentheses. These numbers are related to the inspection, e.g., 8111 (the 11th inspection of the plant in 1981); and the severity level, e.g., 4 (severity level IV). Violations are ranked by severity levels from I through V with level I being the most serious. The severity level is used in the determination of any resulting enforcement action. Gray Book lists severity level by Arabic numbers corresponding to the Roman numerals. Details on the various severity levels and enforcement actions can be found in Appendix C to 10 CFR Part 2 published in the Federal Register of March 9, 1982 pages 9987 through 9995, and as corrected April 14, 1982.

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SECTION 1

CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS

LICENSED POWER REACTORS		93	IN COMMERCIAL OPERATION	77,703	CAPACITY MWe (Net) --Based upon maximum dependable capacity; design elec. rating used if MDC not determined
	(a)	3	IN POWER ASCENSION	3,250	
	(b)	96	LICENSED TO OPERATE	80,953	TOTAL
	(c)	4	LICENSED FOR FUEL LOADING AND LOW POWER TESTING		

	MDC NET
(a) ENRICO FERMI 2	1093
RIVER BEND	936
PALO VERDE 2	1221

(b) Excludes these plants licensed for operation which are shut down indefinitely

	DER
1. DRESDEN 1	200
2. HUMBOLDT BAY	65
3. TMI 2	906

(c)	DATE	DER
SHOREHAM	07/03/85	820
CATAWBA 2	02/27/86	1145
PERRY 1	03/18/86	1205
HOPE CREEK 1	04/11/86	1067

POWER GENERATION

	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWHE)	37,117,973	32,282,943	236,300,508
2. NET ELECTRICAL (MWHE)	35,207,047	30,567,716	224,339,334
3. AVG. UNIT SERVICE FACTOR (%)	67.4	61.4	63.4
4. AVG. UNIT AVAILABILITY FACTOR (%)	67.4	61.4	63.7
5. AVG. UNIT CAPACITY FACTOR (MDC) (%)	62.0	56.2	58.6
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	60.6	54.9	57.3
7. FORCED OUTAGE RATE (%)	20.9	21.2	16.6

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

			% OF POTENTIAL PRODUCTION
1. ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD	35,207,047	NET	60.9
2. ENERGY NOT PRODUCED DUE TO SCHEDULED OUTAGES (NET)	6,403,509	MWHe	11.1
3. ENERGY NOT PRODUCED DUE TO FORCED OUTAGES (NET)	13,668,521	MWHe	23.6
4. ENERGY NOT PRODUCED FOR OTHER REASONS (NET)	2,531,955	MWHe	4.4
POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net)	57,811,032	MWHe	100.0% TOTAL
5. ENERGY NOT PRODUCED DUE TO NRC-REQUIRED OUTAGES	370,402	MWHe	
6. ENERGY NOT PRODUCED DUE TO NRC RESTRICTED POWER LEVELS	.....	MWHe	0 UNIT(S) WITH NRC RESTRICTION

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING REPORT PERIOD	69	15,170.3	21.9	13,668,521
2. SCHEDULED OUTAGES DURING REPORT PERIOD	24	8,100.2	11.7	6,403,509
TOTAL	93	23,270.5	33.6	20,072,030

MWHE LOST PRODUCTION = Down time X maximum dependable capacity net

MONTHLY HIGHLIGHTS

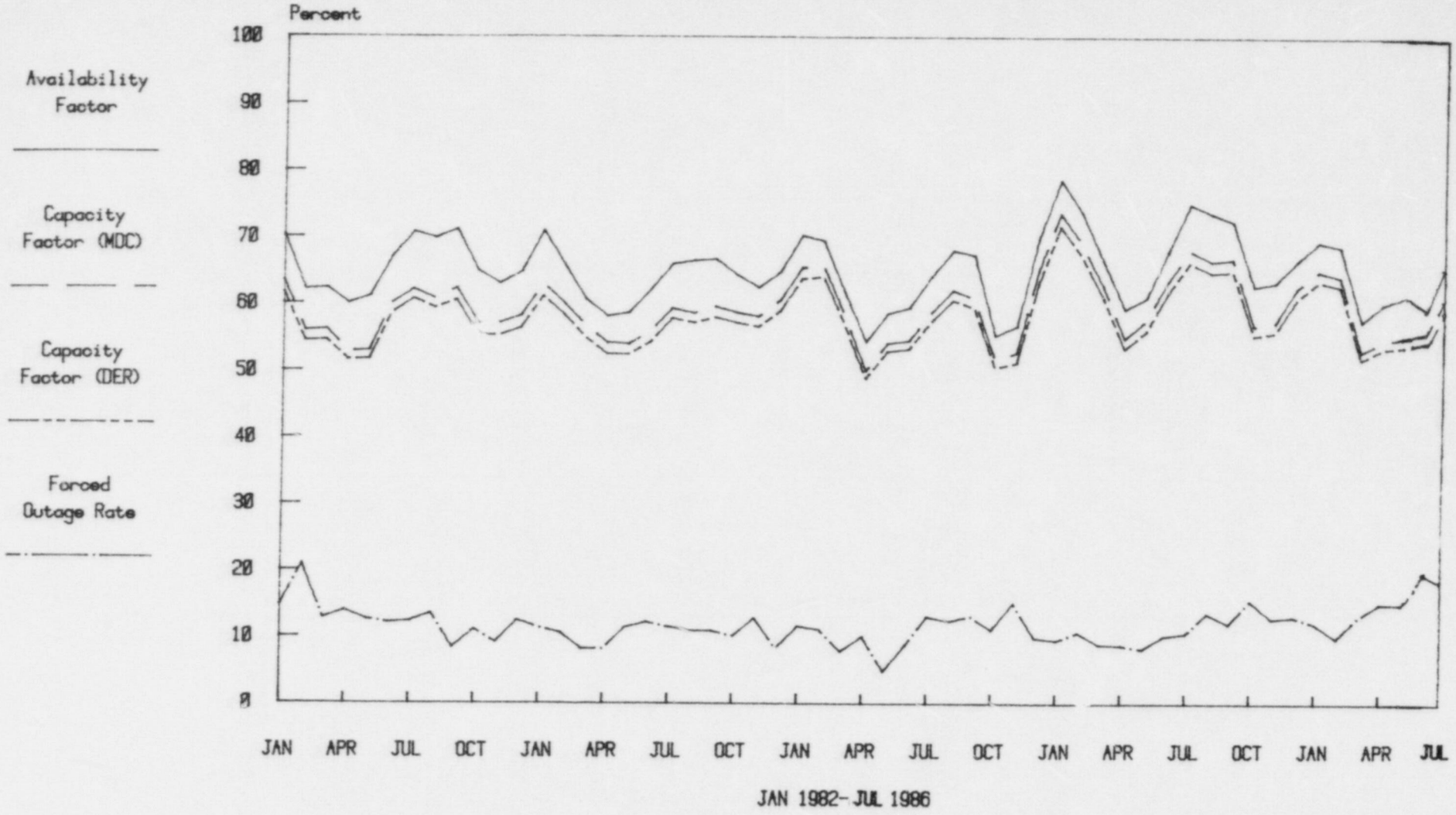
**********		NUMBER	HOURS LOST
* REASONS *	A - Equipment Failure	50	9,233.9
* FOR *	B - Maintenance or Test	14	1,446.0
* SHUTDOWNS *	C - Refueling	12	6,843.4
**********	D - Regulatory Restriction	3	903.9
	E - Operator Training & License Examination	0	0.0
	F - Administrative	6	4,464.0
	G - Operational Error	5	315.5
	H - Other	3	63.8
	TOTAL	93	23,270.5

**********		MDC (MWe Net)	POWER LIMIT (MWe Net)	TYPE
* DERATED *	ARKANSAS 1	836	616	Self-imposed
* UNITS *	BYRON 1	*29	*52	Self-imposed
**********	INDIAN POINT 3	965	515	Self-imposed
	SAN ONOFRE 1	436	390	Self-imposed

**********	UNIT	REASON	UNIT	REASON	UNIT	REASON	UNIT	REASON
* SHUTDOWNS *	ARKANSAS 2	C	BEAVER VALLEY 1	C	BIG ROCK POINT 1	B	BROWNS FERRY 1	F
* GREATER *	BROWNS FERRY 2	F	BROWNS FERRY 3	F	BRUNSWICK 2	B	BYRON 1	A
* THAN 72 HRS *	CALVERT CLIFFS 2	A	CATAWBA 1	A	COOK 1	A,G	COOK 2	C
* EACH *	DAVIS-BESSE 1	A	DRESDEN 3	C	FARLEY 2	A	FORT ST VRAIN	D
**********	HADDAM NECK	A	HATCH 2	A	INDIAN POINT 3	A	LA CROSSE	A
	LASALLE 1	C	LASALLE 2	A	LIMERICK 1	A	MAINE YANKEE	B
	MCGUIRE 1	C	MILLSTONE 3	A	MONTICELLO	C	NINE MILE POINT 1	A
	OYSTER CREEK 1	C	PALISADES	A	PALO VERDE 1	A	PEACH BOTTOM 3	A
	PILGRIM 1	A	RANCHO SECO 1	A	SALEM 1	A	SAN ONOFRE 1	C
	SAN ONOFRE 2	A	SEQUOYAH 1	F,F	SEQUOYAH 2	F	SURRY 1	C
	SURRY 2	D	SUSQUEHANNA 2	A	TURKEY POINT 3	B	TURKEY POINT 4	C
	WATERFORD 3	A	ZION 2	A				

Unit Availability, Capacity, Forced Outage

Avg. Unit Percentage as of 07-31-86



JAN 1982- JUL 1986

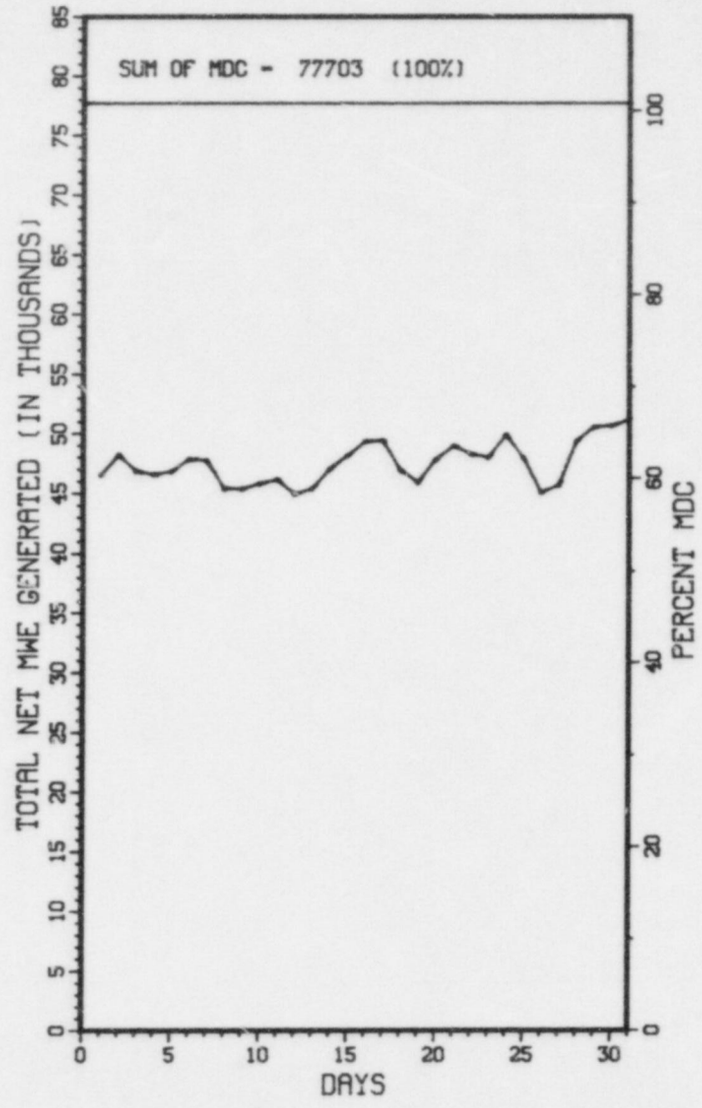
AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MDC" is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the same units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MDC".

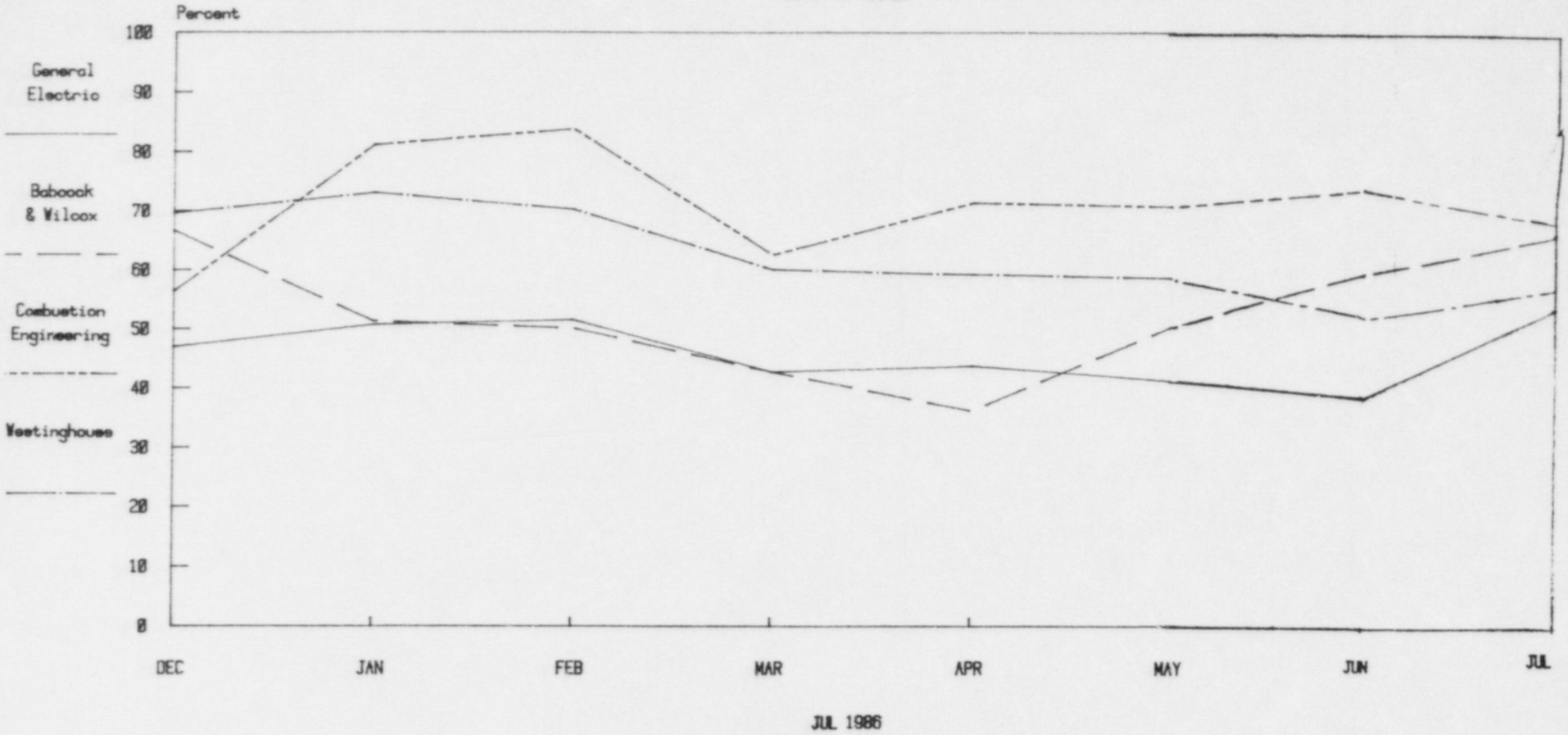
It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.



JULY 1986

Vendor Average Capacity Factors

As Of 07-31-86



JUL 1986

NOTE: This display of average capacity factors provides a general performance comparison of plants supplied by the four nuclear steam supply system vendors. One must be careful when drawing conclusions regarding the reasons for the performance levels indicated, since plant performance may be affected by unspecified factors such as: (1) various plant designs and models are included for each vendor; (2) turbine/generators and (3) different architect/engineers are also involved.

AVERAGE CAPACITY FACTORS BY VENDORS

GENERAL ELECTRIC

CFMDC		CFMDC		CFMDC		CFMDC	
0.0	BROWNS FERRY 1	0.0	BROWNS FERRY 2	0.0	BROWNS FERRY 3	94.9	BRUNSWICK 1
79.7	BRUNSWICK 2	89.7	COOPER STATION	83.7	DRESDEN 2	0.0	DRESDEN 3
92.6	DUANE ARNOLD	91.5	FITZPATRICK	68.8	GRAND GULF 1	96.2	HATCH 1
57.9	HATCH 2	0.0	LASALLE 1	0.0	LASALLE 2	66.4	LIMERICK 1
97.5	MILLSTONE 1	51.0	MONTICELLO	68.0	NINE MILE POINT 1	0.0	OYSTER CREEK 1
93.2	PEACH BOTTOM 2	57.2	PEACH BOTTOM 3	0.0	PILGRIM 1	90.1	QUAD CITIES 1
95.5	QUAD CITIES 2	97.7	SUSQUEHANNA 1	63.2	SUSQUEHANNA 2	72.5	VERMONT YANKEE 1
81.9	WASHINGTON NUCLEAR 2						

BABCOCK & WILCOX

CFMDC		CFMDC		CFMDC		CFMDC	
74.3	ARKANSAS 1	97.7	CRYSTAL RIVER 3	0.0	DAVIS-BESSE 1	97.8	OCONEE 1
91.7	OCONEE 2	89.0	OCONEE 3	0.0	RANCHO SECO 1	99.5	THREE MILE ISLAND 1

COMBUSTION ENGINEERING

CFMDC		CFMDC		CFMDC		CFMDC	
0.0	ARKANSAS 2	94.6	CALVERT CLIFFS 1	80.4	CALVERT CLIFFS 2	87.5	FORT CALHOUN 1
38.5	MAINE YANKEE	98.1	MILLSTONE 2	0.0	PALISADES	63.0	PALO VERDE 1
85.8	SAN ONOFRE 2	88.7	SAN ONOFRE 3	103.4	ST LUCIE 1	103.0	ST LUCIE 2
62.6	WATERFORD 3						

WESTINGHOUSE

CFMDC		CFMDC		CFMDC		CFMDC	
0.0	BEAVER VALLEY 1	22.2	BYRON 1	86.9	CALLAWAY 1	54.0	CATAWBA 1
31.2	COOK 1	35.3	COOK 2	70.3	DIABLO CANYON 1	66.5	DIABLO CANYON 2
92.0	FARLEY 1	81.0	FARLEY 2	93.6	GINNA	30.1	HADDAM NECK
89.6	INDIAN POINT 2	6.8	INDIAN POINT 3	103.2	KEWAUNEE	0.0	MCGUIRE 1
94.8	MCGUIRE 2	74.8	MILLSTONE 3	98.9	NORTH ANNA 1	92.6	NORTH ANNA 2
97.2	POINT BEACH 1	95.6	POINT BEACH 2	100.4	PRAIRIE ISLAND 1	96.4	PRAIRIE ISLAND 2
106.5	ROBINSON 2	75.2	SALEM 1	78.8	SALEM 2	3.4	SAN ONOFRE 1
0.0	SEQUOYAH 1	0.0	SEQUOYAH 2	90.2	SUMMER 1	33.1	SURRY 1
69.7	SURRY 2	101.5	TROJAN	45.0	TURKEY POINT 3	0.0	TURKEY POINT 4
86.8	WOLF CREEK 1	90.9	YANKEE-ROWE 1	82.6	ZION 1	52.5	ZION 2

OTHER INFO

Units excluded are:
- BIG ROCK POINT
- DRESDEN 1
- FORT ST VRAIN
- HUMBOLDT BAY
- LACROSSE
- THREE MILE ISLAND 2

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the glossary. The vendor averages are computed by the formula:

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production by Vendor in this Month}} \times 100\%$$

	GE BWRs	West PWRs	Comb PWRs	B&W PWRs	ALL PWRs
NET ELECTRICAL PRODUCTION	10,236,051	15,515,893	6,000,081	3,412,006	24,927,980
MDC NET	24,552	34,436	11,522	6,746	52,704
CFMDC	56.0	60.6	70.0	68.0	63.6

MEMORANDA

THE FOLLOWING UNITS USE WEIGHTED AVERAGES TO CALCULATE CAPACITY FACTORS:

ITEM 22

BIG ROCK POINT 1
CALVERT CLIFFS 1 & 2
FARLEY 1
FITZPATRICK
FORT CALHOUN 1
INDIAN POINT 2*
KEWAUNEE
OYSTER CREEK 1
POINT BEACH 1 & 2
THREE MILE ISLAND 1
TURKEY POINT 3 & 4

ITEM 22 & 23

GINNA
HADDAM NECK (CONNECTICUT YANKEE)
MAINE YANKEE
MILLSTONE 2
OCONEE 1, 2, & 3
YANKEE-ROWE 1

*COMPUTED SINCE 7/1/74, THE DATE OF COMPLETION OF A 100 DAY - 100% POWER OPERATION TEST.

THE FOLLOWING UNITS USE THE DATE OF FIRST ELECTRICAL GENERATION INSTEAD OF COMMERCIAL OPERATION, FOR THEIR CUMULATIVE DATA:

ITEMS 20 THROUGH 24

COOK 1 & 2
BEAVER VALLEY 1
SAN ONOFRE 1

ITEM 24 ONLY

BIG ROCK POINT 1

ERRATA

CORRECTIONS TO PREVIOUSLY REPORTED DATA

NOTE: THESE CHANGES ARE REFLECTED IN THE DATA CONTAINED IN THE CURRENT REPORT

REVISED MONTHLY HIGHLIGHTS

N O N E
N O N E
N O N E
N O N E

SECTION 2

OPERATING POWER REACTORS

1. Docket: 50-313 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: J. N. GOBELL (501) 964-3251
4. Licensed Thermal Power (MWt): 2568
5. Nameplate Rating (Gross MWe): 1003 X 0.9 = 903
6. Design Electrical Rating (Net MWe): 850
7. Maximum Dependable Capacity (Gross MWe): 883
8. Maximum Dependable Capacity (Net MWe): 836
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe): 616
11. Reasons for Restrictions, If Any:
FOULING AT THE TUBE SUPPORT PLATE CREVICE.

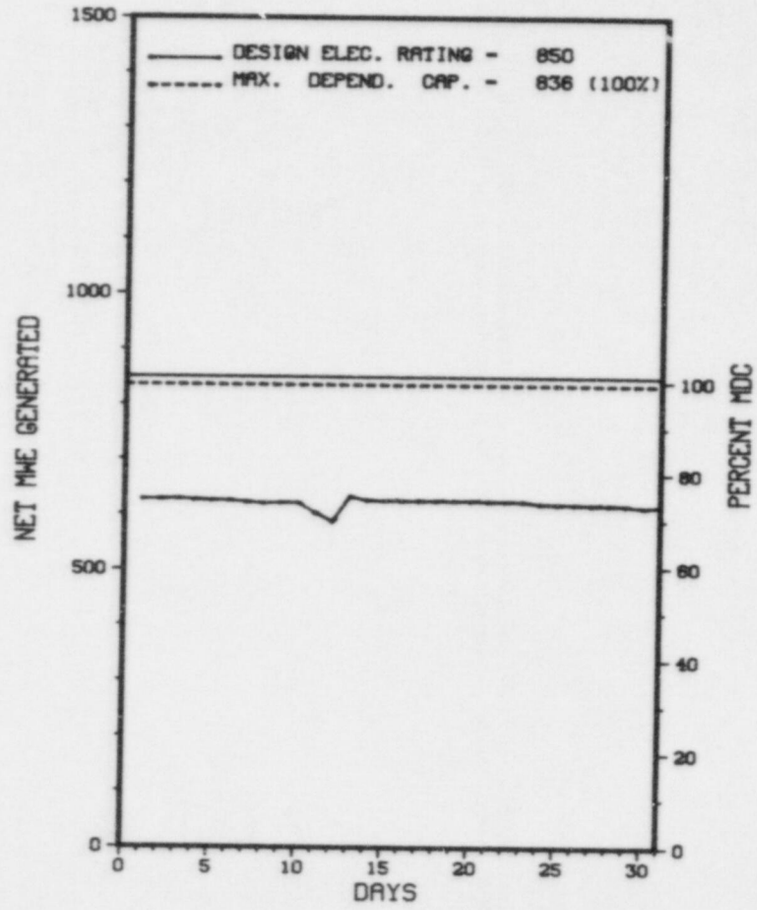
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	101,826.0
13. Hours Reactor Critical	744.0	4,644.5	70,307.6
14. Rx Reserve Shtdwn Hrs	.0	.0	5,044.0
15. Hrs Generator On-Line	744.0	4,626.0	68,884.1
16. Unit Reserve Shtdwn Hrs	.0	.0	817.5
17. Gross Therm Ener (MWH)	1,448,799	9,561,784	162,146,546
18. Gross Elec Ener (MWH)	489,315	3,262,455	53,697,180
19. Net Elec Ener (MWH)	462,105	3,094,035	51,146,911
20. Unit Service Factor	100.0	90.9	67.6
21. Unit Avail Factor	100.0	90.9	68.5
22. Unit Cap Factor (MDC Net)	74.3	72.8	60.1
23. Unit Cap Factor (DER Net)	73.1	71.6	59.1
24. Unit Forced Outage Rate	.0	9.1	14.9
25. Forced Outage Hours	.0	461.0	12,021.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
REFUELING: 09/03/86 - 10 WEEKS.
27. If Currently Shutdown Estimated Startup Date: N/A

ARKANSAS 1

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ARKANSAS 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* ARKANSAS 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
8605	07/11/86	F	0.0	H	5				POWER REDUCTION TO MAKE CONTROL ROD ADJUSTMENTS.

****
* SUMMARY *
****

ARKANSAS 1 OPERATED WITH 1 REDUCTION DURING JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* ARKANSAS 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....ARKANSAS

COUNTY.....POPE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...6 MI WNW OF
RUSSELLVILLE, AR

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 6, 1974

DATE ELEC ENER 1ST GENER...AUGUST 17, 1974

DATE COMMERCIAL OPERATE....DECEMBER 19, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....DARDANELLE RESERVOIR

ELECTRIC RELIABILITY
COUNCIL.....SOUTHWEST POWER POOL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....ARKANSAS POWER & LIGHT

CORPORATE ADDRESS.....NINTH & LOUISIANA STREETS
LITTLE ROCK, ARKANSAS 72203

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....B. JOHNSON

LICENSING PROJ MANAGER.....G. VISSING
DOCKET NUMBER.....50-313

LICENSE & DATE ISSUANCE....DPR-51, MAY 21, 1974

PUBLIC DOCUMENT ROOM.....ARKANSAS TECH UNIVERSITY
RUSSELLVILLE, ARKANSAS 72801

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION CONDUCTED MAY 1-31, 1986 (86-15) ROUTINE, UNANNOUNCED INSPECTION INCLUDING OPERATIONAL SAFETY VERIFICATION, MAINTENANCE, SURVEILLANCE, FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS, CONDENSATE STORAGE TANK CONSTRUCTION, RECEIVING INSPECTION AND PROCUREMENT CONTROL, AND PROCUREMENT DOCUMENT CONTROL. WITHIN THE SEVEN AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED.

INSPECTION CONDUCTED MAY 19-23, 1986 (86-17) A REGION IV TEAM INSPECTED LICENSEE MAINTENANCE ACTIVITIES INCLUDING MAINTENANCE PROGRAM, MAINTENANCE PROGRAM IMPLEMENTATION, INSTRUMENTATION AND CONTROL (I&C) MAINTENANCE, AND ELECTRICAL MAINTENANCE. OF THE FOUR AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED.

INSPECTION CONDUCTED JUNE 9-13, 1986 (86-18) ROUTINE, UNANNOUNCED INSPECTION OF ORGANIZATION AND ADMINISTRATION, TESTS AND EXPERIMENTS PROGRAM, AND SURVEILLANCE TEST AND CALIBRATION CONTROL PROGRAM. WITHIN THE THREE AREAS INSPECTED, TWO APPARENT VIOLATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED JUNE 3-6, 1986 (86-19) ROUTINE, UNANNOUNCED INSPECTION OF THE LOCK AND KEY CONTROL SYSTEM, DETECTION AIDS - PROTECTED AREA (PA), ASSESMENT AIDS, LIGHTING, ACCESS CONTROL - PACKAGES, ACCESS CONTROL - VEHICLES, AND PHYSICAL BARRIERS - PA. WITHIN THE AREAS INSPECTED, ONE APPARANT VIOLATION WAS IDENTIFIED.

INSPECTION CONDUCTED JUNE 23-27, 1986 (86-20) ROUTINE, UNANNOUNCED INSPECTION OF LICENSED OPERATOR TRAINING AND NONLICENSED STAFF TRAINING. WITHIN THE TWO AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED JUNE 1-30, 1986 (86-21) ROUTINE, UNANNOUNCED INSPECTION INCLUDING OPERATIONAL SAFETY VERIFICATION,

INSPECTION SUMMARY

MAINTENANCE, SURVEILLANCE, FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS, FOLLOWUP ON LICENSEE EVENT REPORTS, FOLLOWUP ON IE BULLETIN 85-03, REVIEW OF TRANSIENT REPORTS, AND 10 CFR 50.59 EVALUATIONS. WITHIN THE AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED.

ENFORCEMENT SUMMARY

FAILURE TO CONTROL ACCESS. SAFEGUARDS INFO. FAILURE TO CONTROL KEYS. SAFEGUARDS INFO
(8600 4)

ANO UNITS 1 AND 2 TECHNICAL SPEICIFICATIONS 6.10 AND 6.11 RESPECTIVELY, REQUIRE, IN PART, THAT "PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10CFR PART 20 AND SHALL BE APPROVED, MAINTAINED AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE." CONTRARY TO THE ABOVE, THE NRC INSPECTOR DETERMINED ON MAY 21, 1986, THAT AN OBSOLETE PROCEDURE WAS BEING USED FOR THE SETUP AND OPERATION OF A PERSONNEL THERMOLUMINESCENT DOSIMETRY READER.
DETECTION AIDS - PA; SAFEGUARDS INFO.
(8601 4)

CONTRARY TO 10CFR 50.59(B), THE LICENSEE DID NOT PROVIDE A WRITTEN SAFETY EVALUATION WHICH PROVIDED THE BASES FOR THE DETERMINATION THAT A TEST PROCEDURE AND FIVE DESIGN CHANGES DID NOT INVOLVE AN UNREVIEWED SAFETY QUESTION.
(8602 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

POWER OPERATION

LAST IE SITE INSPECTION DATE: JUNE 1-30, 1986

INSPECTION REPORT NO: 50-313/86-21

REPORTS FROM LICENSEE

ARKANSAS 1

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

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1. Docket: 50-368 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: D. F. HARRISON (501) 964-3743
4. Licensed Thermal Power (MWt): 2815
5. Nameplate Rating (Gross MWe): 943
6. Design Electrical Rating (Net MWe): 912
7. Maximum Dependable Capacity (Gross MWe): 897
8. Maximum Dependable Capacity (Net MWe): 858
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	55,655.0
13. Hours Reactor Critical	.0	3,853.3	39,535.3
14. Rx Reserve Shtdwn Hrs	.0	.0	1,430.1
15. Hrs Generator On-Line	.0	3,831.8	38,267.5
16. Unit Reserve Shtdwn Hrs	.0	.0	75.0
17. Gross Therm Ener (MWH)	0	10,664,834	97,702,184
18. Gross Elec Ener (MWH)	0	3,545,089	32,034,331
19. Net Elec Ener (MWH)	-2,608	3,381,883	30,490,999
20. Unit Service Factor	.0	75.3	68.8
21. Unit Avail Factor	.0	75.3	68.9
22. Unit Cap Factor (MDC Net)	.0	77.5	63.9
23. Unit Cap Factor (DER Net)	.0	72.9	60.1
24. Unit Forced Outage Rate	.0	2.6	15.6
25. Forced Outage Hours	.0	101.5	7,058.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

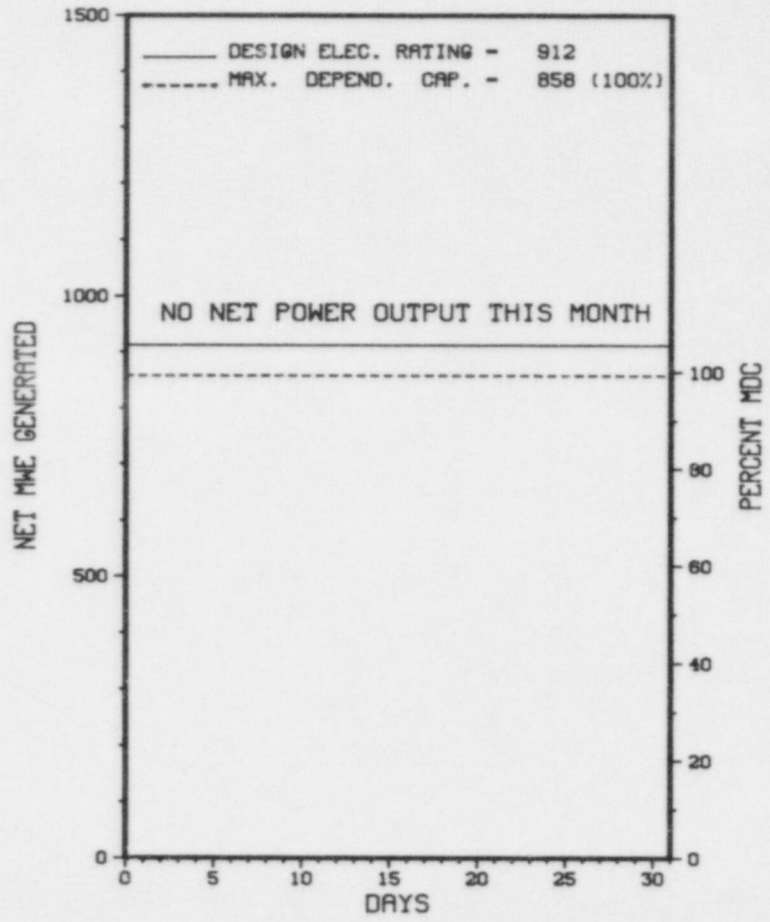
NONE

27. If Currently Shutdown Estimated Startup Date: 08/23/86

****
* ARKANSAS 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ARKANSAS 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* ARKANSAS 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
8607	06/13/86	S	744.0	C	4		RC	FUELXX	CONTINUATION OF REFUELING OUTAGE 2R5.

****
* SUMMARY *
****

ARKANSAS 2 REMAINS SHUTDOWN FOR REFUELING.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* ARKANSAS 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....ARKANSAS

COUNTY.....POPE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...6 MI WNW OF
RUSSELLVILLE, AR

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...DECEMBER 5, 1978

DATE ELEC ENER 1ST GENER...DECEMBER 26, 1978

DATE COMMERCIAL OPERATE....MARCH 26, 1980

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....DARDANELLE RESERVOIR

ELECTRIC RELIABILITY
COUNCIL.....SOUTHWEST POWER POOL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....ARKANSAS POWER & LIGHT

CORPORATE ADDRESS.....NINTH & LOUISIANA STREETS
LITTLE ROCK, ARKANSAS 72203

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....W. JOHNSON

LICENSING PROJ MANAGER.....R. LEE
DOCKET NUMBER.....50-368

LICENSE & DATE ISSUANCE....NPF-6, SEPTEMBER 1, 1978

PUBLIC DOCUMENT ROOM.....ARKANSAS TECH UNIVERSITY
RUSSELLVILLE, ARKANSAS 72801

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION CONDUCTED MAY 1-31, 1986 (86-15) ROUTINE, UNANNOUNCED INSPECTION INCLUDING OPERATIONAL SAFETY VERIFICATION, MAINTENANCE, SURVEILLANCE, FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS, CONDENSATE STORAGE TANK CONSTRUCTION, RECEIVING INSPECTION AND PROCUREMENT CONTROL, AND PROCUREMENT DOCUMENT CONTROL, FOLLOWUP ON LICENSEE EVENT REPORT, IMPLEMENTATION OF VENDOR OPERATION AND MAINTENANCE RECOMMENDATIONS. WITHIN THE AREAS INSPECTED, THREE VIOLATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED MAY 19-23, 1986 (86-18) A REGION IV TEAM INSPECTED LICENSEE MAINTENANCE ACTIVITIES INCLUDING MAINTENANCE PROGRAM, MAINTENANCE PROGRAM IMPLEMENTATION, INSTRUMENTATION AND CONTROL (I&C) MAINTENANCE, AND ELECTRICAL MAINTENANCE. OF THE FOUR AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED.

INSPECTION CONDUCTED JUNE 9-13, 1986 (86-19) ROUTINE, UNANNOUNCED INSPECTION OF ORGANIZATION AND ADMINISTRATION, TESTS AND EXPERIMENTS PROGRAM, AND SURVEILLANCE TEST AND CALIBRATION CONTROL PROGRAM. WITHIN THE THREE AREAS INSPECTED, TWO APPARENT VIOLATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED JUNE 3-6, 1986 (86-20) ROUTINE, UNANNOUNCED INSPECTION OF THE LOCK AND KEY CONTROL SYSTEM, DETECTION AIDS - PROTECTED AREA (PA), ASSESMENT AIDS, LIGHTING, ACCESS CONTROL - PACKAGES, ACCESS CONTROL - VEHICLES, AND PHYSICAL BARRIERS - PA. WITHIN THE AREAS INSPECTED, ONE APPARANT VIOLATION WAS IDENTIFIED.

INSPECTION CONDUCTED JUNE 23-27, 1986 (86-21) ROUTINE, UNANNOUNCED INSPECTION OF LICENSED OPERATOR TRAINING AND NONLICENSED STAFF TRAINING. WITHIN THE TWO AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

****
* ARKANSAS 2 *
****

INSPECTION SUMMARY

INSPECTION CONDUCTED JUNE 1-30, 1986 (86-22) ROUTINE, UNANNOUNCED INSPECTION INCLUDING OPERATIONAL SAFETY VERIFICATION, MAINTENANCE, SURVEILLANCE, FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS, FOLLOWUP ON LICENSEE EVENT REPORTS, FOLLOWUP ON IE BULLETIN 85-03, REVIEW OF TRANSIENT REPORTS, AND 10 CFR 50.59 EVALUATIONS. WITHIN THE AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED.

ENFORCEMENT SUMMARY

FAILURE TO CONTROL ACCESS. SAFEGUARDS INFO. FAILURE TO CONTROL KEYS. SAFEGUARDS INFO.
(8600 4)

ANO UNITS 1 AND 2 TECHNICAL SPEICIFICATIONS 6.10 AND 6.11 RESPECTIVELY, REQUIRE, IN PART, THAT "PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10CFR PART 20 AND SHALL BE APPROVED, MAINTAINED AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE." CONTRARY TO THE ABOVE, THE NRC INSPECTOR DETERMINED ON MAY 21, 1986, THAT AN OBSOLETE PROCEDURE WAS BEING USED FOR THE SETUP AND OPERATION OF A PERSONNEL THERMOLUMINESCENT DOSIMETRY READER.
(8601 4)

DETECTION AIDS - PA; SAFEGUARDS INFO CONTRARY TO TECHNICAL SPECIFICATION 6.8.1, THE POST-MAINTENANCE TEST PROCEDURE FOR BATTERY CHARGER 2D34 WAS NOT FOLLOWED.
(8602 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

LAST IE SITE INSPECTION DATE: JUNE 1-30, 1986

INSPECTION REPORT NO: 50-368/86-22

REPORTS FROM LICENSEE

ARKANSAS 2

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

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1. Docket: 50-334 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-Line Hrs: 744.0
3. Utility Contact: P. A. SMITH (412) 643-1825
4. Licensed Thermal Power (MWt): 2652
5. Nameplate Rating (Gross MWe): 1026 X 0.9 = 923
6. Design Electrical Rating (Net MWe): 835
7. Maximum Dependable Capacity (Gross MWe): 860
8. Maximum Dependable Capacity (Net MWe): 810
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	89,855.0
13. Hours Reactor Critical	.0	3,220.1	48,825.0
14. Rx Reserve Shtdwn Hrs	.0	.0	4,482.7
15. Hrs Generator On-Line	.0	3,217.5	47,352.4
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	8,049,760	111,073,182
18. Gross Elec Ener (MWH)	0	2,638,550	35,419,990
19. Net Elec Ener (MWH)	-3,120	2,480,490	33,016,703
20. Unit Service Factor	.0	63.2	55.1
21. Unit Avail Factor	.0	63.2	55.1
22. Unit Cap Factor (MDC Net)	.0	60.2	48.8
23. Unit Cap Factor (DER Net)	.0	58.4	47.3
24. Unit Forced Outage Rate	.0	1.4	22.6
25. Forced Outage Hours	.0	44.7	18,534.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

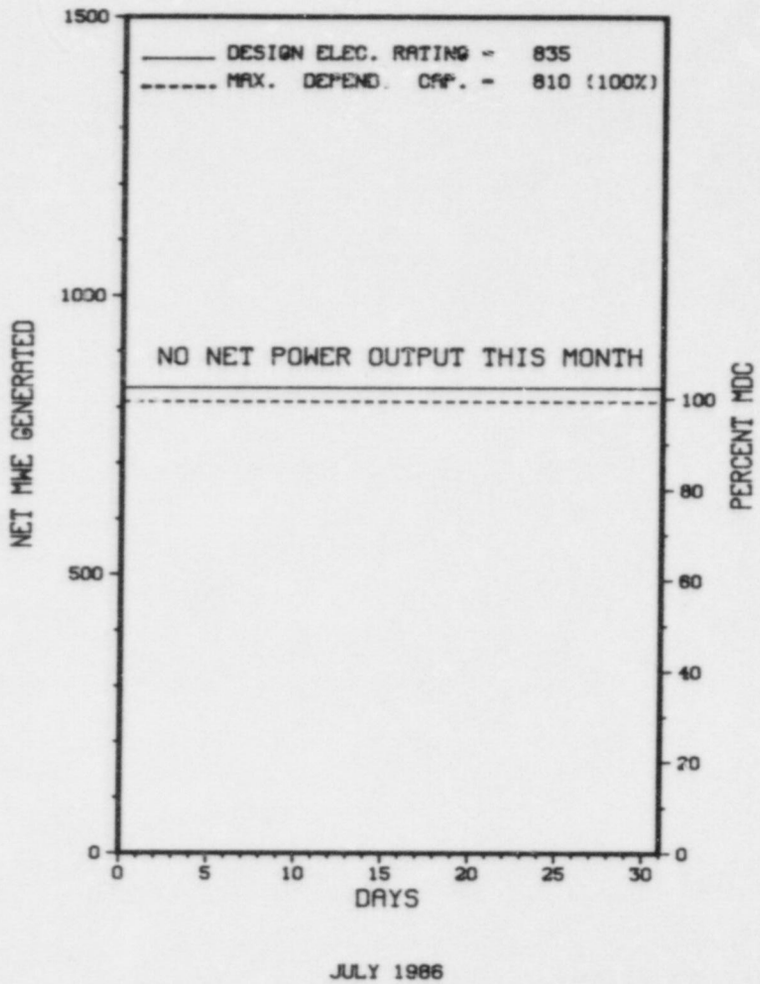
NONE

27. If Currently Shutdown Estimated Startup Date: 08/17/86

****
* BEAVER VALLEY 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BEAVER VALLEY 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

BEAVER VALLEY 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
7	08/17/86	S	744.0	C	4		RC	FUELXX	STATION REMAINED SHUTDOWN FOR THE FIFTH REFUELING.

SUMMARY

BEAVER VALLEY 1 REMAINS SHUTDOWN FOR REFUELING.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* BEAVER VALLEY 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....PENNSYLVANIA

COUNTY.....BEAVER

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI E OF
E. LIVERPOOL, OH

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 10, 1976

DATE ELEC ENER 1ST GENER...JUNE 14, 1976

DATE COMMERCIAL OPERATE....OCTOBER 1, 1976

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....OHIO RIVER

ELECTRIC RELIABILITY
COUNCIL.....EAST CENTRAL AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUQUESNE LIGHT

CORPORATE ADDRESS.....ONE OXFORD CENTRE, 301 GRANT STREET
PITTSBURGH, PENNSYLVANIA 15279

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....W. TROSKOSKI

LICENSING PROJ MANAGER.....P. TAM
DOCKET NUMBER.....50-334

LICENSE & DATE ISSUANCE....DPR-66, JULY 2, 1976

PUBLIC DOCUMENT ROOM.....B.F. JONES MEMORIAL LIBRARY
633 FRANKLIN AVENUE
ALIQUIPPA, PA 15001

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

10 CFR 71.5 PROHIBITS DELIVERY OF LICENSED MATERIAL TO A CARRIER FOR TRANSPORT UNLESS THE LICENSEE COMPLIES WITH APPLICABLE REGULATIONS OF THE DEPARTMENT OF TRANSPORTATION IN 49 CFR PARTS 170-189. 49 CFR 173.425(B)(1), "TRANSPORT REQUIREMENTS FOR LOW SPECIFIC ACTIVITY (LSA) RADIOACTIVE MATERIALS," REQUIRES THAT PACKAGED SHIPMENTS OF LSA MATERIAL CONSIGNED AS EXCLUSIVE USE BE PACKAGED IN STRONG, TIGHT PACKAGES SO THAT THERE WILL BE NO LEAKAGE OF RADIOACTIVE MATERIAL UNDER CONDITIONS NORMALLY INCIDENT TO TRANSPORTATION. CONTRARY TO THE ABOVE ON OCTOBER 17, 1985, TWO 55-GALLON STEEL DRUMS (DRUMS NO. 14 AND 24) CONTAINING LOW SPECIFIC ACTIVITY MATERIAL WERE CONSIGNED TO A CARRIER FOR EXCLUSIVE USE TO TRANSPORT TO QUADREX CORPORATION IN OAK RIDGE, TENNESSEE, AND THE DRUMS WERE NOT STRONG AND TIGHT. SPECIFICALLY, DRUM NO. 14 CONTAINING 0.021 MILLICURIES OF RADIOACTIVE MATERIAL HAD FOUR HOLES, APPROXIMATELY 1/4 INCH IN DIAMETER, THAT PENETRATED TO THE INSIDE OF THE BARREL AND WERE LOCATED ABOUT ONE INCH BELOW THE BARREL LID LOCKING RING. DRUM NO. 24 WAS PUNCTURED ON THE BOTTOM, AND THE PUNCTURES CONSISTED OF TWO "SLASHES" ABOUT THREE INCHES LONG BY 1/4 INCH WIDE, WHICH HAD BEEN COVERED WITH YELLOW TAPE. 10 CFR 71.101(B) REQUIRES EACH LICENSEE TO ESTABLISH A QUALITY ASSURANCE PROGRAM FOR PACKAGES. 10 CFR 71.101(F) STATES THAT A COMMISSION APPROVED QUALITY ASSURANCE PROGRAM THAT SATISFIES THE APPLICABLE CRITERIA OF APPENDIX B, PART 50, OF THIS CHAPTER, AND WHICH IS ESTABLISHED, MAINTAINED, AND EXECUTED WITH REGARD TO TRANSPORT PACKAGES WILL BE ACCEPTED AS SATISFYING THE REQUIREMENTS OF PARAGRAPH (B) OF

ENFORCEMENT SUMMARY

THIS SECTION. CRITERION XVIII OF APPENDIX B, PART 50, REQUIRES, IN PART, THAT A COMPREHENSIVE SYSTEM OF PLANNED AND PERIODIC AUDITS BE CARRIED OUT TO VERIFY COMPLIANCE WITH ALL ASPECTS OF THE QUALITY ASSURANCE PROGRAM AND TO DETERMINE THE EFFECTIVENESS OF THE PROGRAM. THE AUDITS SHALL BE PERFORMED BY APPROPRIATELY TRAINED PERSONNEL. CONTRARY TO THE ABOVE, DURING THE PERIOD OF JUNE-JULY 1985, QUALITY ASSURANCE MAINTENANCE PERSONNEL CONDUCTED AN AUDIT OF THE SOLID WASTE MANAGEMENT AREA, AND THE THREE AUDITORS PERFORMING THE AUDIT HAD NOT BEEN ADEQUATELY TRAINED TO ASSURE THAT SUITABLE PROFICIENCY WAS ACHIEVED AND MAINTAINED. SPECIFICALLY, ALTHOUGH THE LEAD AUDITOR RECEIVED TWO TO THREE DAYS TRAINING IN TRANSPORTATION ACTIVITIES IN MAY 1984, AND THE OTHER AUDITORS RECEIVED TWO DAYS TRAINING DURING THE PERIOD OF OCTOBER-NOVEMBER 1985, THIS TRAINING WAS NOT ADEQUATE IN THAT THE TRAINING MATERIAL PROVIDED WAS TOO VOLUMINOUS AND COMPLEX TO BE ABSORBED AND RETAINED IN A TWO DAY TRAINING PROGRAM. CONTRARY TO TS 4.3.3.5, MONTHLY REMOTE SHUTDOWN MONITORING INSTRUMENTATION CHANNEL CHECK FAILED TO QUALITATIVELY ASSESS TRB-RH-606 AS INOPERABLE WHEN INDICATED READINGS WERE OFF-SCALE LOW AT LESS THAN EXPECTED AMBIENT. CONTRARY TO TS 4.3.3.6.1, THE LICENSEE FAILED TO DEMONSTRATE INDIVIDUAL SMOKE DETECTOR OPERABILITY IN THE NO. 2 DG ROOM DURING PERFORMANCE OF A SURVEILLANCE TEST.
(8600 4)

TECHNICAL SPECIFICTION 6.8, PROCEDURES, REQUIRES THAT PROCEDURES BE ESTABLISHED, IMPLEMENTED AND MAINTAINED. ITEM E.12 OF STATION ADMINISTRATION PROCEDURE, CHAPTER 6, RADIOLOGICAL CONTROL GROUP ADMINISTRATION, DEVELOPED PURSUANT TO THE ABOVE, REQUIRES THAT PROCEDURES BE REVIEWED AT A MINIMUM INTERVAL OF EVERY TWO YEARS, OR AFTER SIGNIFICANT CHANGES OR INCIDENTS, TO DETERMINE IF CHANGES TO THE PROCEDURES ARE NECESSARY OR DESIRABLE. CONTRARY TO THE ABOVE, AS OF MARCH 14, 1986, PROCEDURE NO. FO-OP-004, "DEWATERING PROCEDURE FOR THE 24 INCH DIAMETER PRESSURE DEMINIERALIZER VESSEL CONTAINING ION EXCHANGE RESINS," USED ON AT LEAST ONE OCCASION DURING 1985, HAD NOT BEEN REVIEWED SINCE JUNE 15, 1983, AN INTERVAL OF MORE THAN TWO YEARS.

10 CFR 71.12(A) PERMITS A GENERAL LICENSE TO BE ISSUED TO ANY LICENSEE OF THE COMMISSION TO TRANSPORT, OR DELIVER TO A CARRIER FOR TRANSPORT, LICENSED MATERIAL IN A PACKAGE FOR WHICH A CERTIFICATE OF COMPLIANCE HAS BEEN ISSUED BY THE NRC. 10 CFR 71.12(C)(1) STATES IN PART, THAT THIS GENERAL LICENSE APPLIES ONLY TO A LICENSEE WHO HAS A COPY OF THE CERTIFICATE OF COMPLIANCE AND HAS THE DRAWINGS AND OTHER DOCUMENTS REFERENCED IN THE APPROVAL RELATING TO THE USE AND MAINTENANCE OF THE PACKAGING AND TO THE ACTIONS TO BE TAKEN PRIOR TO SHIPMENT. CONTRARY TO THE ABOVE, ON APRIL 23, 1985, THE LICENSEE DELIVERED TO A CARRIER FOR TRANSPORT 29.14 CURIES OF LICENSED MATERIAL IN A PACKAGE, MODEL NO. CNS 6-80-2, CERTIFICATE OF COMPLIANCE (C OF C) NO. 9111, AND ALTHOUGH THE LICENSEE HAD A REDUCED COPY OF A DRAWING, IT COULD NOT BE VERIFIED AS THE REFERENCED DRAWING BECAUSE IDENTIFICATION NUMBER WAS NOT LEGIBLE.
(8600 5)

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

OTHER ITEMS

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

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1. Docket: 50-155 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: J. P. JOHNSTON (616) 547-6537

4. Licensed Thermal Power (MWt): 240

5. Nameplate Rating (Gross MWe): 70.6 X 0.85 = 60

6. Design Electrical Rating (Net MWe): 72

7. Maximum Dependable Capacity (Gross MWe): 73

8. Maximum Dependable Capacity (Net MWe): 69

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	204,618.0
13. Hours Reactor Critical	498.9	4,714.3	145,946.1
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	482.9	4,688.7	143,329.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	101,893	978,605	27,023,874
18. Gross Elec Ener (MWH)	32,675	313,248	8,554,436
19. Net Elec Ener (MWH)	30,965	297,468	8,089,631
20. Unit Service Factor	64.9	92.2	70.0
21. Unit Avail Factor	64.9	92.2	70.0
22. Unit Cap Factor (MDC Net)	60.3	84.7	58.8*
23. Unit Cap Factor (DER Net)	57.8	81.2	54.9
24. Unit Forced Outage Rate	35.1	7.8	14.8
25. Forced Outage Hours	261.1	398.3	11,747.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

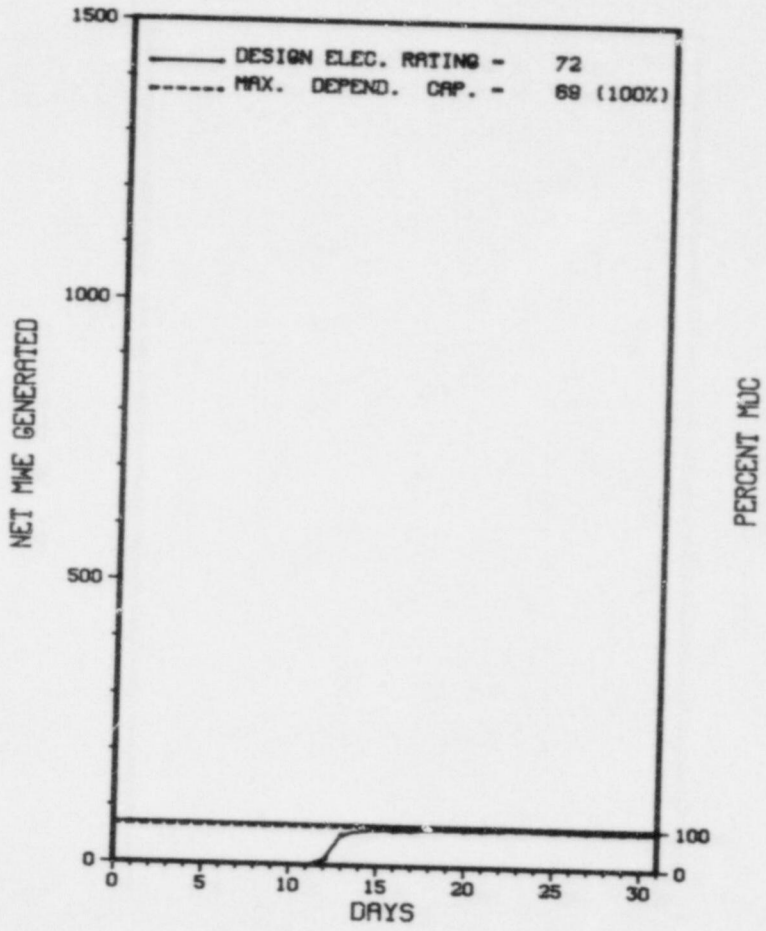
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* BIG ROCK POINT 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BIG ROCK POINT 1



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

BIG ROCK POINT 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-03	07/01/86	F	261.1	B	3	86-05			THE PLANT TRIPPED DURING THE PERFORMANCE OF A ROUTINE REACTOR PROTECTION SYSTEM SURVEILLANCE TEST.

SUMMARY

BIG ROCK POINT 1 OPERATED WITH 1 OUTAGE DUE TO SURVEILLANCE TESTING.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* BIG ROCK POINT 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....MICHIGAN

COUNTY.....CHARLEVOIX

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...4 MI NE OF
CHARLEVOIX, MICH

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...SEPTEMBER 27, 1962

DATE ELEC ENER 1ST GENER...DECEMBER 8, 1962

DATE COMMERCIAL OPERATE....MARCH 29, 1963

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....EAST CENTRAL AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CONSUMERS POWER

CORPORATE ADDRESS.....212 WEST MICHIGAN AVENUE
JACKSON, MICHIGAN 49201

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....S. GUTHRIE

LICENSING PROJ MANAGER.....T. ROTELLA
DOCKET NUMBER.....50-155

LICENSE & DATE ISSUANCE....DPR-6, AUGUST 30, 1962

PUBLIC DOCUMENT ROOM.....NORTH CENTRAL MICHIGAN COLLEGE
1515 HOWARD STREET
PETOSKEY, MICHIGAN 49770

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON AUGUST 13, 1985 THROUGH MARCH 14, 1986 (85015): SPECIAL INSPECTION OF ALLEGATIONS PERTAINING TO THE SECURITY PROGRAM. ALTHOUGH SOME ALLEGATIONS WERE SUBSTANTIATED, THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITHIN THE AREAS EXAMINED.

INSPECTION ON APRIL 9 THROUGH JULY 10 (86007): ROUTINE, UNANNOUNCED INSPECTION CONDUCTED BY THE SENIOR RESIDENT INSPECTOR OF LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS, OPERATIONAL SAFETY, MAINTENANCE OPERATION, SURVEILLANCE OBSERVATION, REACTOR TRIPS, IE BULLETINS, TMI ACTION ITEMS, LICENSEE EVENT REPORT FOLLOWUP, LICENSING ACTIONS, AND TRAINING. ONE VIOLATION WAS IDENTIFIED (FAILURE OF THE OPERATOR TO MAINTAIN DESIRED CONTROL ROD POSITION AND TO INVESTIGATE THE ABNORMAL BEHAVIOR OF SLUGGISH CONTROL ROD MOVEMENT).

INSPECTION ON JUNE 24-27 (86010): ROUTINE, ANNOUNCED INSPECTION OF THE EFFECTIVENESS OF NON-LICENSED PERSONNEL TRAINING (INSPECTION PROCEDURE 41400) AND LICENSED PERSONNEL TRAINING (INSPECTION PROCEDURE 41701). IN THE TWO AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

RESTART ON 07/12/86 - NOW OPERATING ROUTINELY

LAST IE SITE INSPECTION DATE: 08/08/86

INSPECTION REPORT NO: 86012

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-04	06/04/86	07/25/86	BATTERY SERVICE TEST TECHNICAL SPECIFICATION CONTRADICTION
86-05	07/01/86	07/31/86	NEUTRON MONITORING SYSTEM CABLE FAILURE RESULTING IN REACTOR TRIP

1. Docket: 50-259 O P E R A T I N G S T A T U S

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: K. L. CREAMER (205) 729-2955

4. Licensed Thermal Power (MWt): 3293

5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152

6. Design Electrical Rating (Net MWe): 1065

7. Maximum Dependable Capacity (Gross MWe): 1098

8. Maximum Dependable Capacity (Net MWe): 1065

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	105,193.0
13. Hours Reactor Critical	.0	.0	59,520.9
14. Rx Reserve Shtdwn Hrs	.0	.0	6,996.8
15. Hrs Generator On-Line	.0	.0	58,276.4
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	167,963,338
18. Gross Elec Ener (MWH)	0	0	55,393,130
19. Net Elec Ener (MWH)	-5,300	-21,872	53,695,137
20. Unit Service Factor	.0	.0	55.4
21. Unit Avail Factor	.0	.0	55.4
22. Unit Cap Factor (MDC Net)	.0	.0	47.9
23. Unit Cap Factor (DER Net)	.0	.0	47.9
24. Unit Forced Outage Rate	100.0	100.0	32.7
25. Forced Outage Hours	744.0	5,087.0	28,265.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

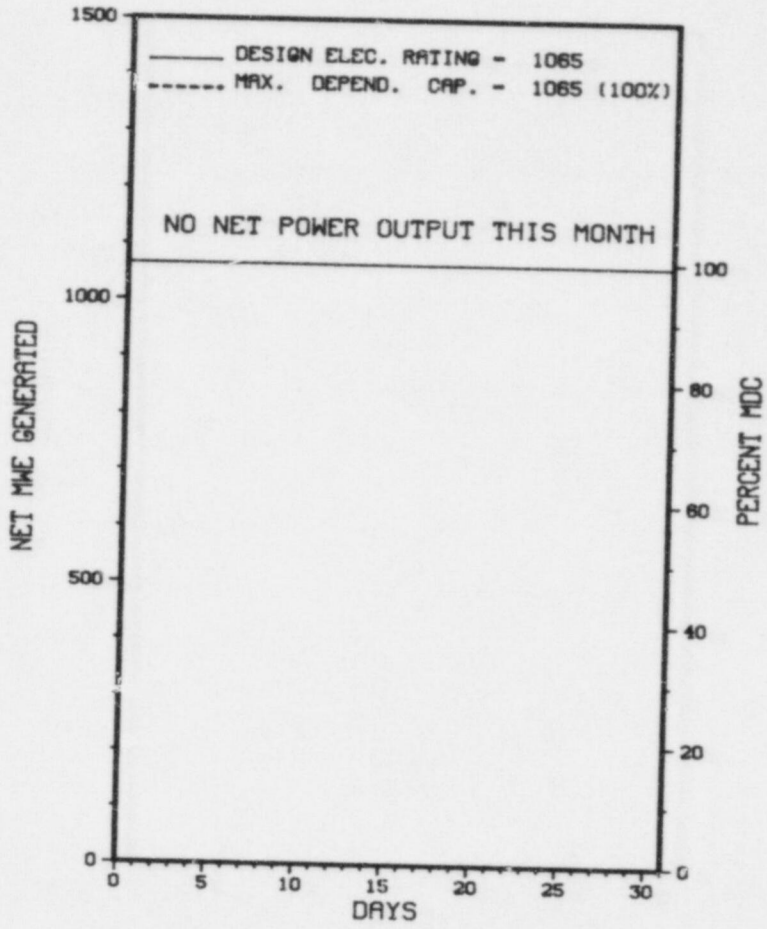
NONE

27. If Currently Shutdown Estimated Startup Date: 03/15/89

****
* BROWNS FERRY 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BROWNS FERRY 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

BROWNS FERRY 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
315	06/01/85	F	744.0	F	4				ADMINISTRATIVE HOLD TO RESOLVE VARIOUS TVA AND NRC CONCERNS.

SUMMARY

BROWNS FERRY 1 REMAINS SHUTDOWN FOR AN ADMINISTRATIVE OUTAGE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* BROWNS FERRY 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....ALABAMA

COUNTY.....LIMESTONE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...10 MI NW OF
DECATUR, ALA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...AUGUST 17, 1973

DATE ELEC ENER 1ST GENER...OCTOBER 15, 1973

DATE COMMERCIAL OPERATE....AUGUST 1, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....TENNESSEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....TENNESSEE VALLEY AUTHORITY

CORPORATE ADDRESS.....500A CHESTNUT STREET TOWER II
CHATTANOOGA, TENNESSEE 37401

CONTRACTOR
ARCHITECT/ENGINEER.....TENNESSEE VALLEY AUTHORITY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....TENNESSEE VALLEY AUTHORITY

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....J. PAULK

LICENSING PROJ MANAGER.....J. GEARS
DOCKET NUMBER.....50-259

LICENSE & DATE ISSUANCE....DPR-33, DECEMBER 20, 1973

PUBLIC DOCUMENT ROOM.....ATHENS PUBLIC LIBRARY
SOUTH AND FORREST
ATHENS, ALABAMA 35611

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MAY 19-23 (86-17): THIS WAS A ROUTINE, UNANNOUNCED INSPECTION IN THE AREAS OF OBSERVATION OF NONDESTRUCTIVE EXAMINATION ACTIVITIES PERFORMED IN ACCORDANCE WITH NRC GENERIC LETTER 84-11, NONDESTRUCTIVE EXAMINATION DATA REVIEW AND EVALUATION, INDEPENDENT RE-EXAMINATION OF RECIRCULATION SYSTEM WELDS AND FOLLOWUP ON PREVIOUS ENFORCEMENT ITEMS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 16-20 (86-21): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED BY TWO INSPECTORS AND CONSISTED OF A REVIEW OF THE LICENSEE'S: SECURITY PROGRAM AUDIT; RECORDS AND REPORTS; TESTING AND MAINTENANCE; PHYSICAL BARRIERS-VITAL AREAS; SECURITY SYSTEMS POWER SUPPLY; ALARM STATIONS; AND PERSONNEL TRAINING AND QUALIFICATIONS-GENERAL REQUIREMENTS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH REGULATORY REQUIREMENTS WITHIN THE SEVEN AREAS INSPECTED.

INSPECTION JUNE 1-30 (86-22): THIS ROUTINE INSPECTION WAS IN THE AREAS OF OPERATIONAL SAFETY, MAINTENANCE OBSERVATION, SURVEILLANCE TESTING OBSERVATION, REPORTABLE OCCURRENCES AND DESIGN MODIFICATIONS. ONE VIOLATION - 10CFR 50, APPENDIX B, CRITERION III FOR DESIGN CONTROL OF CABLE PULLING ACTIVITIES.

INSPECTION JUNE 23-25 (86-23): THIS ROUTINE, UNANNOUNCED INSPECTION WAS IN THE AREA OF ULTRASONIC EXAMINATION OF THE REACTOR VESSEL NOZZLE THERMAL SLEEVE WELDS UTILIZING TVA'S AUTOMATED ULTRASONIC SYSTEM (INTRASPECT-98). NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

MANAGEMENT MEETING JUNE 24 (86-24): A MANAGEMENT MEETING WAS HELD AT REGION II ON JUNE 24, 1986, AT 11:00 A.M. EDT, TO DISCUSS

INSPECTION SUMMARY

CABLE PULLING ISSUES. TVA'S REVISED SPECIFICATIONS FOR INSTALLATION OF ELECTRICAL CABLE AND CONDUIT WERE NOT IMPLEMENTED FOR MODIFICATIONS AT BROWNS FERRY WHEN ISSUED. TVA GAVE A CHRONOLOGY OF EVENTS CONCERNING CABLE PULLING INCLUDING: (1) WHICH SITES WERE USING THE NEW SPECIFICATIONS; (2) THE RATIONALE FOR USING THE OLD SPECIFICATION AT BFNP UNIT 2; (3) BFNP CORRECTIVE ACTIONS TAKEN SINCE THE JUNE 17TH TVA HOLD ON CABLE PULLING; (4) THE ONGOING BFNP CABLE INTEGRITY VERIFICATION PROGRAM; AND (5) THE REQUIREMENTS FOR LIFTING THE STOP WORK ORDER. TVA CHARACTERIZED THEIR EARLIER DECISION NOT TO IMPLEMENT THE G38/40 SPECIFICATIONS AT BFNP UNIT 2 AS NOT AGGRESSIVE. THE NRC CONCURRED IN THE REQUIREMENTS FOR LIFTING THE STOP WORK ORDER AND WANTED TO LOOK AT THE TEST RESULTS JUSTIFYING THE OLD INSTALLATIONS. TVA WAS REQUESTED TO PROVIDE A FORMAL WRITTEN POSITION CONCERNING CABLE PULLING ACTIVITIES AT BROWNS FERRY.

ENFORCEMENT SUMMARY

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION VI, CHANGES WERE MADE TO THE TVA NATURAL RESOURCE OPERATIONS, FIELD OPERATIONS, QUALITY ASSURANCE PROCEDURE NO. NR-FO-NRE-62.2, CALIBRATION OF ORNL RADIATION MONITORS, BROWNS FERRY NUCLEAR PLANT ONLY WITHOUT REVIEW AND APPROVAL BY AUTHORIZED PERSONNEL. THE AIRFLOW CALIBRATION SECTION OF PROCEDURE NR-FO-NRE-62.2 BECAME INADEQUATE FOLLOWING A MODIFICATION TO THE EQUIPMENT IN THE SPRING OF 1985. SINCE THE MODIFICATION WAS NOT COMPLETED, AN UNOFFICIAL SUBSTITUTE CALIBRATION PROCEDURE, WHICH ITSELF CONTAINED PEN-AND-INK CHANGES TO THE TYPED DOCUMENT, HAD BEEN USED. THE SUBSTITUTE CALIBRATION PROCEDURE AND CHANGES FOR THIS PROCEDURE HAD NOT BEEN REVIEWED AND APPROVED BY AUTHORIZED PERSONNEL AS OF MAY 21, 1986. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION VI, CHANGES WERE MADE TO THE TVA NATURAL RESOURCE OPERATIONS, FIELD OPERATIONS, QUALITY ASSURANCE PROCEDURE NO. NR-FO-NRE-62.2, CALIBRATION OF ORNL RADIATION MONITORS, BROWNS FERRY NUCLEAR PLANT ONLY WITHOUT REVIEW AND APPROVAL BY AUTHORIZED PERSONNEL. THE AIRFLOW CALIBRATION SECTION OF PROCEDURE NR-FO-NRE-62.2 BECAME INADEQUATE FOLLOWING A MODIFICATION TO THE EQUIPMENT IN THE SPRING OF 1985. SINCE THE MODIFICATION WAS NOT COMPLETED, AN UNOFFICIAL SUBSTITUTE CALIBRATION PROCEDURE, WHICH ITSELF CONTAINED PEN-AND-INK CHANGES TO THE TYPED DOCUMENT, HAD BEEN USED. THE SUBSTITUTE CALIBRATION PROCEDURE AND CHANGES FOR THIS PROCEDURE HAD NOT BEEN REVIEWED AND APPROVED BY AUTHORIZED PERSONNEL AS OF MAY 21, 1986. CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION VI, CHANGES WERE MADE TO THE TVA NATURAL RESOURCE OPERATIONS, FIELD OPERATIONS, QUALITY ASSURANCE PROCEDURE NO. NR-FO-NRE-62.2, CALIBRATION OF ORNL RADIATION MONITORS, BROWNS FERRY NUCLEAR PLANT ONLY WITHOUT REVIEW AND APPROVAL BY AUTHORIZED PERSONNEL. THE AIRFLOW CALIBRATION SECTION OF PROCEDURE NR-FO-NRE-62.2 BECAME INADEQUATE FOLLOWING A MODIFICATION TO THE EQUIPMENT IN THE SPRING OF 1985. SINCE THE MODIFICATION WAS NOT COMPLETED, AN UNOFFICIAL SUBSTITUTE CALIBRATION PROCEDURE, WHICH ITSELF CONTAINED PEN-AND-INK CHANGES TO THE TYPED DOCUMENT, HAD BEEN USED. THE SUBSTITUTE CALIBRATION PROCEDURE AND CHANGES FOR THIS PROCEDURE HAD NOT BEEN REVIEWED AND APPROVED BY AUTHORIZED PERSONNEL AS OF MAY 21, 1986. CONTRARY TO TS 6.8.1.A, PROCEDURE WAS NOT IMPLEMENTED IN THAT, ON MARCH 20, 1986, THE BREAKER FOR 2-SW-V255 WAS NOT MAINTAINED CLOSED.
(8601 5)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

ENVIRONMENTAL QUALIFICATION WORK.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

****
* BROWNS FERRY 1 *
****

OTHER ITEMS

PLANT STATUS:

SHUTDOWN FOR REPAIRS ON 03/19.

LAST IE SITE INSPECTION DATE: JUNE 24, 1986 +

INSPECTION REPORT NO: 50-259/86-24 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
85-057	10/09/85	05/23/86	WIDE RANGE TORUS WATER LEVEL TRANSMITTERS NOT QUALIFIED.
86-004	11/28/85	06/18/86	RHR SYSTEM SHOWING CONDUCTIVITY AND CHLORIDE CONCENTRATION IN EXCESS OF PERMISSIBLE LEVELS; CAUSE - SLIGHT LEAKAGE INTO RHR SIDE FROM AROUND INNER FLOATING HEAD GASKETS ON BOTH HEAT EXCHANGERS.
86-013	04/22/86	05/21/86	TECHNICAL SPECIFICATION REQUIREMENTS FOR SECONDARY CONTAINMENT INTEGRITY IN REFUEL ZONE NOT FULLY MET; CAUSE - AIR ISOLATION VALVE IDENTIFICATION TAGS REVERSED ON THE 2 DAMPERS.
86-015	04/29/86	05/28/86	REACTOR PROTECTION SYSTEM TRIP DUE TO VOLTAGE REGULATOR CONTROL SWITCH BEING ACCIDENTLY BUMPED.
86-016	04/01/86	06/24/86	FLUID LEAKAGE PROBLEM WITH LARGE BORE SNUBBERS FOR TORUS DYNAMIC RESTRAINT; SEAL DESIGN IS THE SUSPECTED PROBLEM.
86-017	02/28/86	05/23/86	FAILURE TO PROPERLY TORQUE ENCLOSING TUBE ASSEMBLY NUT.
86-019	06/08/86	07/08/86	INADVERTENT SECONDARY CONTAINMENT ISOLATION FROM RADIATION MONITOR SPIKE; CAUSE - UNKNOWN.

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1. Docket: 50-260 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: K. L. CREAMER (205) 729-2955
4. Licensed Thermal Power (MWt): 3293
5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152
6. Design Electrical Rating (Net MWe): 1065
7. Maximum Dependable Capacity (Gross MWe): 1098
8. Maximum Dependable Capacity (Net MWe): 1065
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	100,104.0
13. Hours Reactor Critical	.0	.0	55,859.6
14. Rx Reserve Shtdwn Hrs	.0	.0	14,200.4
15. Hrs Generator On-Line	.0	.0	54,338.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	153,245,167
18. Gross Elec Ener (MWH)	0	0	50,771,798
19. Net Elec Ener (MWH)	-3,048	-32,766	49,232,598
20. Unit Service Factor	.0	.0	54.3
21. Unit Avail Factor	.0	.0	54.3
22. Unit Cap Factor (MDC Net)	.0	.0	46.2
23. Unit Cap Factor (DER Net)	.0	.0	46.2
24. Unit Forced Outage Rate	100.0	100.0	30.8
25. Forced Outage Hours	744.0	5,087.0	24,320.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

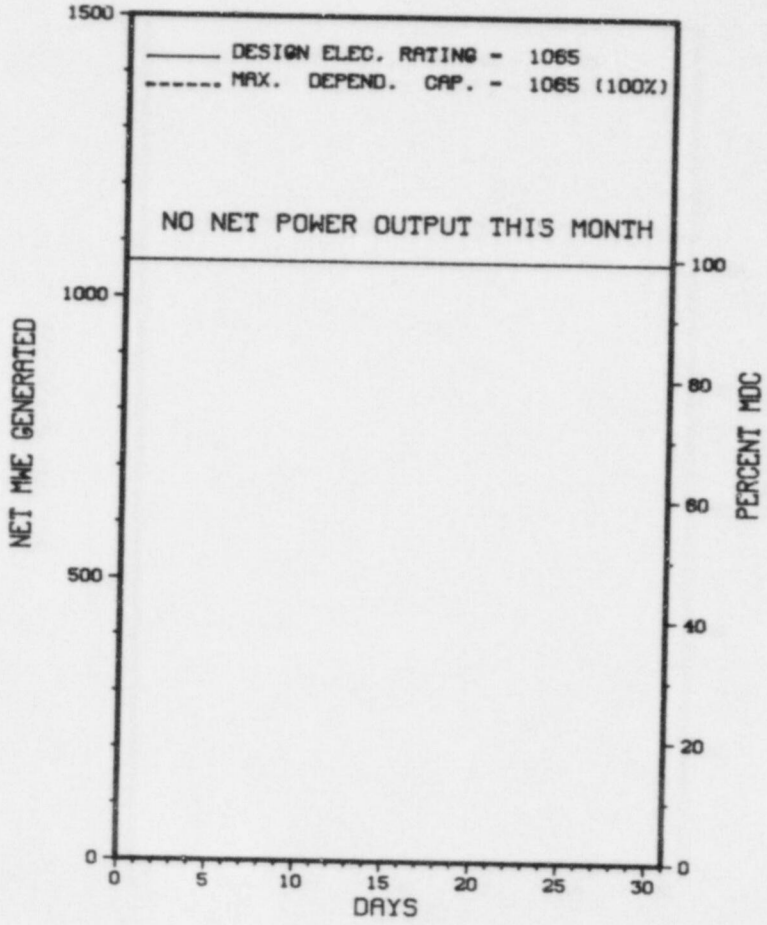
NONE

27. If Currently Shutdown Estimated Startup Date: 12/25/86

****
* BROWNS FERRY 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BROWNS FERRY 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

BROWNS FERRY 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
305	09/15/84	F	744.0	F	4				ADMINISTRATIVE HOLD TO RESOLVE VARIOUS TVA AND NRC CONCERNS.

SUMMARY

BROWNS FERRY 2 REMAINS SHUTDOWN FOR AN ADMINISTRATIVE OUTAGE.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training & License Examination	5-Reduced Load	Licensee Event Report
		9-Other	(LER) File (NUREG-0161)

****
* BROWNS FERRY 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....ALABAMA

COUNTY.....LIMESTONE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...10 MI NW OF
DECATUR, ALA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JULY 20, 1974

DATE ELEC ENER 1ST GENER...AUGUST 28, 1974

DATE COMMERCIAL OPERATE....MARCH 1, 1975

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....TENNESSEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....TENNESSEE VALLEY AUTHORITY

CORPORATE ADDRESS.....500A CHESTNUT STREET TOWER II
CHATTANOOGA, TENNESSEE 37401

CONTRACTOR
ARCHITECT/ENGINEER.....TENNESSEE VALLEY AUTHORITY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....TENNESSEE VALLEY AUTHORITY

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....J. PAULK

LICENSING PROJ MANAGER.....J. GEARS
DOCKET NUMBER.....50-260

LICENSE & DATE ISSUANCE....DPR-52, AUGUST 2, 1974

PUBLIC DOCUMENT ROOM.....ATHENS PUBLIC LIBRARY
SOUTH AND FORREST
ATHENS, ALABAMA 35611

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MAY 19-23 (86-17): THIS WAS A ROUTINE, UNANNOUNCED INSPECTION IN THE AREAS OF OBSERVATION OF NONDESTRUCTIVE EXAMINATION ACTIVITIES PERFORMED IN ACCORDANCE WITH NRC GENERIC LETTER 84-11, NONDESTRUCTIVE EXAMINATION DATA REVIEW AND EVALUATION, INDEPENDENT RE-EXAMINATION OF RECIRCULATION SYSTEM WELDS AND FOLLOWUP ON PREVIOUS ENFORCEMENT ITEMS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 16-20 (86-21): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED BY TWO INSPECTORS AND CONSISTED OF A REVIEW OF THE LICENSEE'S: SECURITY PROGRAM AUDIT; RECORDS AND REPORTS; TESTING AND MAINTENANCE; PHYSICAL BARRIERS-VITAL AREAS; SECURITY SYSTEMS POWER SUPPLY; ALARM STATIONS; AND PERSONNEL TRAINING AND QUALIFICATIONS-GENERAL REQUIREMENTS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH REGULATORY REQUIREMENTS WITHIN THE SEVEN AREAS INSPECTED.

INSPECTION JUNE 1-30 (86-22): THIS ROUTINE INSPECTION WAS IN THE AREAS OF OPERATIONAL SAFETY, MAINTENANCE OBSERVATION, SURVEILLANCE TESTING OBSERVATION, REPORTABLE OCCURRENCES AND DESIGN MODIFICATIONS. ONE VIOLATION - 10CFR 50, APPENDIX B, CRITERION III FOR DESIGN CONTROL OF CABLE PULLING ACTIVITIES.

INSPECTION JUNE 23-25 (86-23): THIS ROUTINE, UNANNOUNCED INSPECTION WAS IN THE AREA OF ULTRASONIC EXAMINATION OF THE REACTOR VESSEL NOZZLE THERMAL SLEEVE WELDS UTILIZING TVA'S AUTOMATED ULTRASONIC SYSTEM (INTRASPECT-98). NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

MANAGEMENT MEETING JUNE 24 (86-24): A MANAGEMENT MEETING WAS HELD AT REGION II ON JUNE 24, 1986, AT 11:00 A.M. EDT, TO DISCUSS

****
* BROWNS FERRY 2 *
****

INSPECTION SUMMARY

CABLE PULLING ISSUES. TVA'S REVISED SPECIFICATIONS FOR INSTALLATION OF ELECTRICAL CABLE AND CONDUIT WERE NOT IMPLEMENTED FOR MODIFICATIONS AT BROWNS FERRY WHEN ISSUED. TVA GAVE A CHRONOLOGY OF EVENTS CONCERNING CABLE PULLING INCLUDING: (1) WHICH SITES WERE USING THE NEW SPECIFICATIONS; (2) THE RATIONALE FOR USING THE OLD SPECIFICATION AT BFNP UNIT 2; (3) BFNP CORRECTIVE ACTIONS TAKEN SINCE THE JUNE 17TH TVA HOLD ON CABLE PULLING; (4) THE ONGOING BFNP CABLE INTEGRITY VERIFICATION PROGRAM; AND (5) THE REQUIREMENTS FOR LIFTING THE STOP WORK ORDER. TVA CHARACTERIZED THEIR EARLIER DECISION NOT TO IMPLEMENT THE G38/40 SPECIFICATIONS AT BFNP UNIT 2 AS NOT AGGRESSIVE. THE NRC CONCURRED IN THE REQUIREMENTS FOR LIFTING THE STOP WORK ORDER AND WANTED TO LOOK AT THE TEST RESULTS JUSTIFYING THE OLD INSTALLATIONS. TVA WAS REQUESTED TO PROVIDE A FORMAL WRITTEN POSITION CONCERNING CABLE PULLING ACTIVITIES AT BROWNS FERRY.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SHUTDOWN ON SEPTEMBER 15, 1984 FOR REFUELING OUTAGE.

LAST IE SITE INSPECTION DATE: JUNE 24, 1986 +

INSPECTION REPORT NO: 50-260/86-24 +

****
* BROWNS FERRY 2 *
****

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-004	04/18/86	05/21/86	INADVERTENT CONTAINMENT ISOLATION AND HALF SCRAM OCCURRED DURING REPLACEMENT OF A RELAY COIL ON 480V REACTOR MOTOR - OPERATED VALVE BOARD 2B; CAUSE - PERSONNEL ERROR.
86-009	05/30/86	06/27/86	INCORRECT PERFORMANCE OF LOCAL LEAK RATE TEST; PROCEDURE HAS BEEN REVISED TO CORREC; ALL IDENTIFIED DEFICIENCIES.

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1. Docket: 50-296 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: K. L. CREAMER (205) 729-2955
4. Licensed Thermal Power (MWt): 3293
5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152
6. Design Electrical Rating (Net MWe): 1065
7. Maximum Dependable Capacity (Gross MWe): 1098
8. Maximum Dependable Capacity (Net MWe): 1065
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

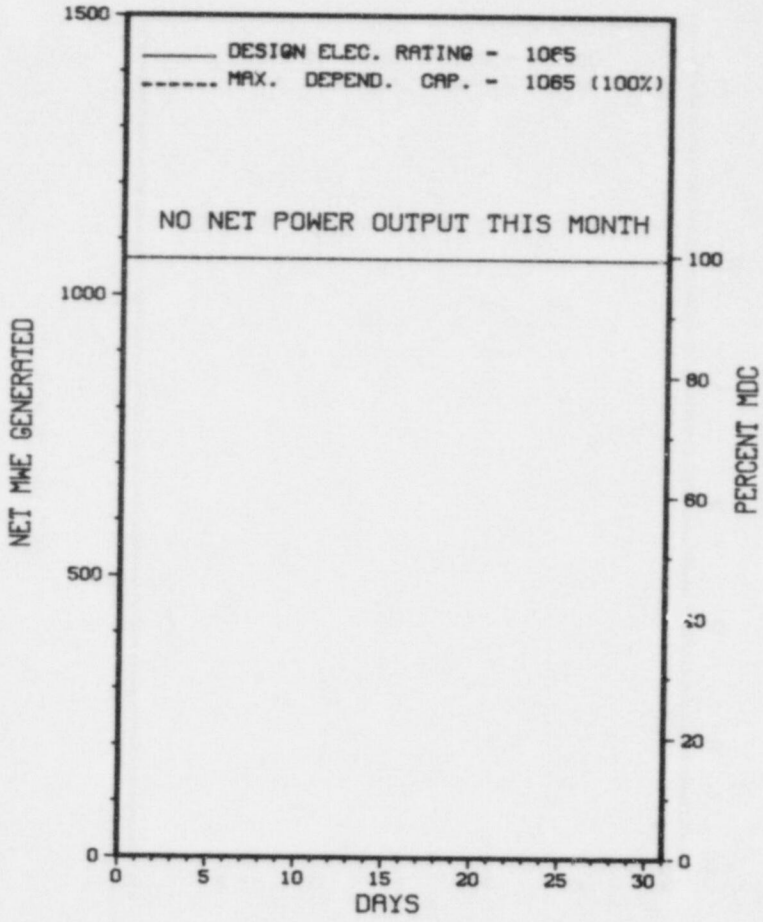
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	82,559.0
13. Hours Reactor Critical	.0	.0	45,306.8
14. Rx Reserve Shtdwn Hrs	.0	.0	5,149.4
15. Hrs Generator On-Line	.0	.0	44,195.6
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	131,846,076
18. Gross Elec Ener (MWH)	0	0	43,473,760
19. Net Elec Ener (MWH)	-3,846	-23,590	42,111,083
20. Unit Service Factor	.0	.0	53.5
21. Unit Avail Factor	.0	.0	53.5
22. Unit Cap Factor (MDC Net)	.0	.0	47.9
23. Unit Cap Factor (DER Net)	.0	.0	47.9
24. Unit Forced Outage Rate	100.0	100.0	29.3
25. Forced Outage Hours	744.0	5,087.0	18,304.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: 03/17/87

****
* BROWNS FERRY 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BROWNS FERRY 3



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

BROWNS FERRY 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
157	12/01/85	F	744.0	F	4				ADMINISTRATIVE HOLD TO RESOLVE VARIOUS TVA AND NRC CONCERNS.

SUMMARY

BROWNS FERRY 3 REMAINS SHUTDOWN FOR AN ADMINISTRATIVE OUTAGE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* BROWNS FERRY 3 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....ALABAMA

COUNTY.....LIMESTONE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...10 MI NW OF
DECATUR, ALA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...AUGUST 8, 1976

DATE ELEC ENER 1ST GENER...SEPTEMBER 12, 1976

DATE COMMERCIAL OPERATE....MARCH 1, 1977

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....TENNESSEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....TENNESSEE VALLEY AUTHORITY

CORPORATE ADDRESS.....500A CHESTNUT STREET TOWER II
CHATTANOOGA, TENNESSEE 37401

CONTRACTOR
ARCHITECT/ENGINEER.....TENNESSEE VALLEY AUTHORITY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....TENNESSEE VALLEY AUTHORITY

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....J. PAULK

LICENSING PROJ MANAGER.....J. GEARS
DOCKET NUMBER.....50-296

LICENSE & DATE ISSUANCE....DPR-68, AUGUST 18, 1976

PUBLIC DOCUMENT ROOM.....ATHENS PUBLIC LIBRARY
SOUTH AND FORREST
ATHENS, ALABAMA 35611

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MAY 19-23 (86-17): THIS WAS A ROUTINE, UNANNOUNCED INSPECTION IN THE AREAS OF OBSERVATION OF NONDESTRUCTIVE EXAMINATION ACTIVITIES PERFORMED IN ACCORDANCE WITH NRC GENERIC LETTER 84-11, NONDESTRUCTIVE EXAMINATION DATA REVIEW AND EVALUATION, INDEPENDENT RE-EXAMINATION OF RECIRCULATION SYSTEM WELDS AND FOLLOWUP ON PREVIOUS ENFORCEMENT ITEMS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 16-20 (86-21): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED BY TWO INSPECTORS AND CONSISTED OF A REVIEW OF THE LICENSEE'S: SECURITY PROGRAM AUDIT; RECORDS AND REPORTS; TESTING AND MAINTENANCE; PHYSICAL BARRIERS-VITAL AREAS; SECURITY SYSTEMS POWER SUPPLY; ALARM STATIONS; AND PERSONNEL TRAINING AND QUALIFICATIONS-GENERAL REQUIREMENTS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH REGULATORY REQUIREMENTS WITHIN THE SEVEN AREAS INSPECTED.

INSPECTION JUNE 1-30 (86-22): THIS ROUTINE INSPECTION WAS IN THE AREAS OF OPERATIONAL SAFETY, MAINTENANCE OBSERVATION, SURVEILLANCE TESTING OBSERVATION, REPORTABLE OCCURRENCES AND DESIGN MODIFICATIONS. ONE VIOLATION - 10CFR 50, APPENDIX B, CRITERION III FOR DESIGN CONTROL OF CABLE PULLING ACTIVITIES.

INSPECTION JUNE 23-25 (86-23): THIS ROUTINE, UNANNOUNCED INSPECTION WAS IN THE AREA OF ULTRASONIC EXAMINATION OF THE REACTOR VESSEL NOZZLE THERMAL SLEEVE WELDS UTILIZING TVA'S AUTOMATED ULTRASONIC SYSTEM (INTRASPECT-98). NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

MANAGEMENT MEETING JUNE 24 (86-24): A MANAGEMENT MEETING WAS HELD AT REGION II ON JUNE 24, 1986, AT 11:00 A.M. EDT, TO DISCUSS

****
* BROWNS FERRY 3 *
****

INSPECTION SUMMARY

CABLE PULLING ISSUES. TVA'S REVISED SPECIFICATIONS FOR INSTALLATION OF ELECTRICAL CABLE AND CONDUIT WERE NOT IMPLEMENTED FOR MODIFICATIONS AT BROWNS FERRY WHEN ISSUED. TVA GAVE A CHRONOLOGY OF EVENTS CONCERNING CABLE PULLING INCLUDING: (1) WHICH SITES WERE USING THE NEW SPECIFICATIONS; (2) THE RATIONALE FOR USING THE OLD SPECIFICATION AT BFNP UNIT 2; (3) BFNP CORRECTIVE ACTIONS TAKEN SINCE THE JUNE 17TH TVA HOLD ON CABLE PULLING; (4) THE ONGOING BFNP CABLE INTEGRITY VERIFICATION PROGRAM; AND (5) THE REQUIREMENTS FOR LIFTING THE STOP WORK ORDER. TVA CHARACTERIZED THEIR EARLIER DECISION NOT TO IMPLEMENT THE G38/40 SPECIFICATIONS AT BFNP UNIT 2 AS NOT AGGRESSIVE. THE NRC CONCURRED IN THE REQUIREMENTS FOR LIFTING THE STOP WORK ORDER AND WANTED TO LOOK AT THE TEST RESULTS JUSTIFYING THE OLD INSTALLATIONS. TVA WAS REQUESTED TO PROVIDE A FORMAL WRITTEN POSITION CONCERNING CABLE PULLING ACTIVITIES AT BROWNS FERRY.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

LICENSEE EVALUATING CAUSE OF REACTOR VESSEL WATER LEVEL INDICATION PROBLEMS.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SHUTDOWN ON MARCH 9, 1985.

LAST IE SITE INSPECTION DATE: JUNE 24, 1986 +

INSPECTION REPORT NO: 50-296/86-24 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-325 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: FRANCES HARRISON (919) 457-9521
4. Licensed Thermal Power (MWt): 2436
5. Nameplate Rating (Gross MWe): 963 X 0.9 = 867
6. Design Electrical Rating (Net MWe): 821
7. Maximum Dependable Capacity (Gross MWe): 815
8. Maximum Dependable Capacity (Net MWe): 790
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	82,152.0
13. Hours Reactor Critical	744.0	4,894.6	51,726.0
14. Rx Reserve Shtdwn Hrs	.0	.0	1,647.1
15. Hrs Generator On-Line	744.0	4,778.7	48,917.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,787,764	11,354,050	101,460,705
18. Gross Elec Ener (MWH)	575,235	3,723,164	33,474,434
19. Net Elec Ener (MWH)	557,818	3,612,243	32,170,931
20. Unit Service Factor	100.0	93.9	59.5
21. Unit Avail Factor	100.0	93.9	59.5
22. Unit Cap Factor (MDC Net)	94.9	89.9	49.6
23. Unit Cap Factor (DER Net)	91.3	86.5	47.7
24. Unit Forced Outage Rate	.0	6.1	17.1
25. Forced Outage Hours	.0	308.3	10,005.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

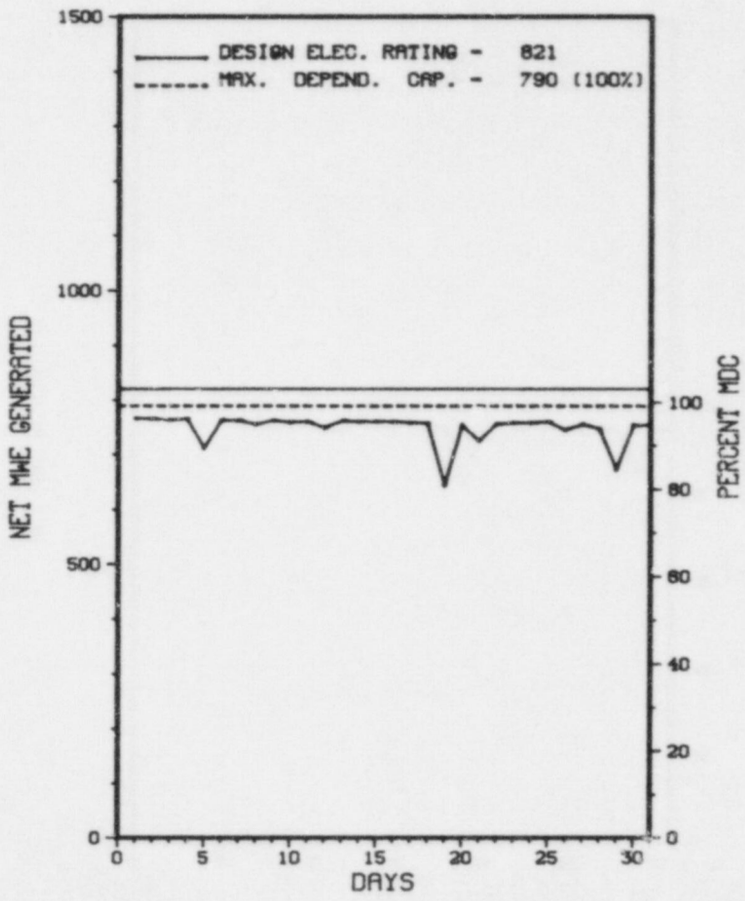
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* BRUNSWICK 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BRUNSWICK 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

BRUNSWICK 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86037	07/18/86	S	0.0	B	5				REDUCED POWER FOR ROD IMPROVEMENT, SCRAM TIME TESTING ON VARIOUS RODS AND STEAM LEAK REPAIR.
86038	07/20/86	F	0.0	A	5				REDUCED POWER TO REDUCE TURBINE BEARING VIBRATION WHILE REMOVING SECONDSTAGE REHEAT FROM SERVICE.
86-040	07/28/86	F	0.0	A	5				REDUCED POWER WHILE REMOVING HEATER DRAIN PUMP 1A FROM SERVICE DUE TO PROBLEM WITh DISCHARGE CHECK VALVE.

SUMMARY

BRUNSWICK 1 OPERATED WITH 3 REDUCTIONS DURING THE JULY REPORT PERIOD.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* BRUNSWICK 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....NORTH CAROLINA

COUNTY.....BRUNSWICK

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...3 MI N OF
SOUTHPORT, NC

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...OCTOBER 8, 1976

DATE ELEC ENER 1ST GENER...DECEMBER 4, 1976

DATE COMMERCIAL OPERATE....MARCH 18, 1977

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CAPE FEAR RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CAROLINA POWER & LIGHT

CORPORATE ADDRESS.....P. O. BOX 1551
RALEIGH, NORTH CAROLINA 27602

CONTRACTOR
ARCHITECT/ENGINEER.....UNITED ENG. & CONSTRUCTORS

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BROWN & ROOT

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....W. RULAND

LICENSING PROJ MANAGER.....E. SYLVESTER
DOCKET NUMBER.....50-325

LICENSE & DATE ISSUANCE....DPR-71, NOVEMBER 12, 1976

PUBLIC DOCUMENT ROOM.....SOUTHPORT-BRUNSWICK COUNTY LIBRARY
108 N. MOORE STREET
SOUTHPORT, NORTH CAROLINA 28461

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JUNE 1-30 (86-15): THIS ROUTINE SAFETY INSPECTION INVOLVED THE AREAS OF FOLLOWUP ON UNRESOLVED ITEM, MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, AND ONSITE FOLLOWUP OF EVENTS. TWO VIOLATIONS - FAILURE TO TAKE ADEQUATE CORRECTIVE ACTION CONCERNING JUMPER CONTROL; FAILURE TO FOLLOW PROCEDURES BY NOT DECLARING A SUPPORT INOPERABLE WHEN REQUIRED.

INSPECTION JUNE 9-13 (86-16): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED IN THE AREAS OF POST-REFUELING INITIAL CRITICALITY AND TESTS AND ROUTINE CORE PERFORMANCE SURVEILLANCE. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 8-10 (86-18): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED A REVIEW OF ALLEGATIONS RELATED TO THE CONDUCT OF OPERATIONS AT THE LICENSEE'S FACILITY. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JULY 8-10, 1986 +

INSPECTION REPORT NO: 50-325/86-18 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-013	05/30/86	06/25/86	TRAIN 2A OF CONTROL BUILDING EMERGENCY AIR FILTRATION SYSTEM AUTOMATICALLY STARTED DUE TO ACTUATION OF AREA RADIATION MONITOR/INDICATOR TRIP UNIT 1-D22-RM-K600-1-2 FOR CONTROL BUILDING COMMON AIR INTAKE SUPPLY DUCT.
86-017	06/14/86	07/14/86	LATE PERFORMANCE OF REQUIRED HOURLY FIRE WATCHES.

1. Docket: 50-324 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: FRANCES HARRISON (919) 457-9521
4. Licensed Thermal Power (MWt): 2436
5. Nameplate Rating (Gross MWe): 963 X 0.9 = 867
6. Design Electrical Rating (Net MWe): 821
7. Maximum Dependable Capacity (Gross MWe): 815
8. Maximum Dependable Capacity (Net MWe): 790
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	94,176.0
13. Hours Reactor Critical	667.4	956.2	55,468.4
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	652.2	822.0	51,833.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,512,107	1,767,167	100,595,625
18. Gross Elec Ener (MWH)	486,417	561,680	33,357,068
19. Net Elec Ener (MWH)	468,352	510,392	31,948,081
20. Unit Service Factor	87.7	16.2	55.0
21. Unit Avail Factor	87.7	16.2	55.0
22. Unit Cap Factor (MDC Net)	79.7	12.7	42.9
23. Unit Cap Factor (DER Net)	76.7	12.2	41.3
24. Unit Forced Outage Rate	.0	3.3	16.8
25. Forced Outage Hours	.0	28.4	10,898.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

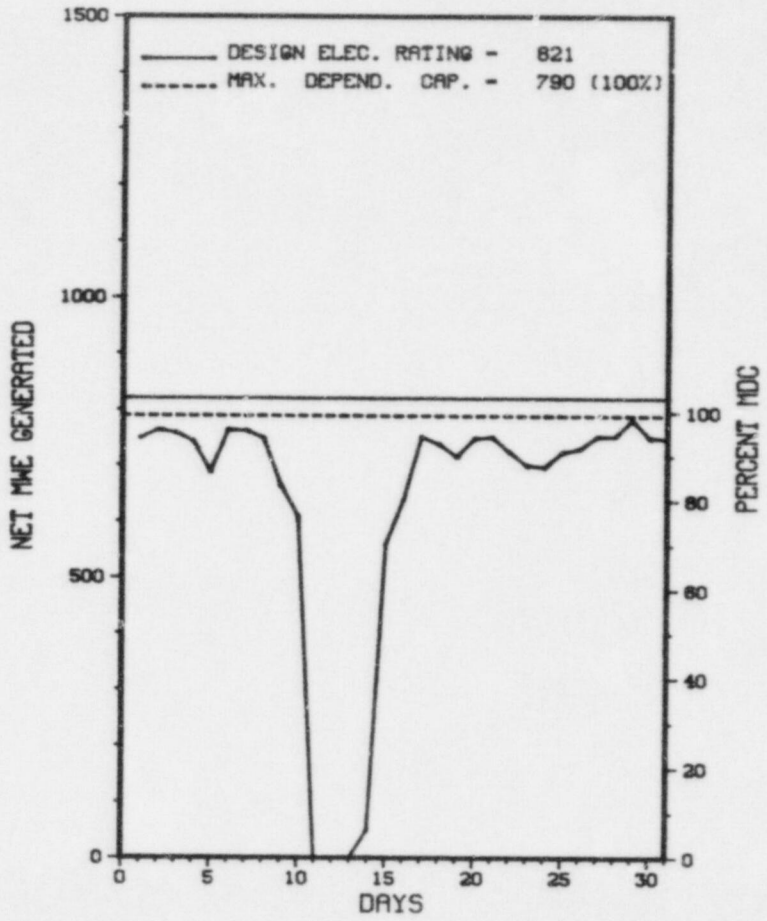
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* BRUNSWICK 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BRUNSWICK 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* BRUNSWICK 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-012	07/10/86	S	91.8	B	1				PLANNED OFF-LINE OUTAGE TO REPAIR REACTOR WATER CLEANUP VALVE G31-F001.
86-016	07/18/86	F	0.0	A	5				REDUCED POWER DUE TO RECIRC PUMP "B" RUNBACK.
86-017	07/22/86	F	0.0	A	5				REDUCED POWER TO RESOLVE TIP PROBLEM (TIP LOCATIONS SWITCHED).

****
* SUMMARY *
****

BRUNSWICK 2 OPERATED WITH 1 OUTAGE AND 2 REDUCTIONS DURING THE JULY REPORT PERIOD.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report (LER) File (NUREG-0161)
			9-Other	

****
* BRUNSWICK 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....NORTH CAROLINA

COUNTY.....BRUNSWICK

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...3 MI N OF
SOUTHPORT, NC

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...MARCH 20, 1975

DATE ELEC ENER 1ST GENER...APRIL 29, 1975

DATE COMMERCIAL OPERATE....NOVEMBER 3, 1975

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CAPE FEAR RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CAROLINA POWER & LIGHT

CORPORATE ADDRESS.....411 FAYETTEVILLE STREET
RALEIGH, NORTH CAROLINA 27602

CONTRACTOR
ARCHITECT/ENGINEER.....UNITED ENG. & CONSTRUCTORS

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BROWN & ROOT

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....W. RULAND

LICENSING PROJ MANAGER.....E. SYLVESTER
DOCKET NUMBER.....50-324

LICENSE & DATE ISSUANCE....DPR-62, DECEMBER 27, 1974

PUBLIC DOCUMENT ROOM.....SOUTHPORT-BRUNSWICK COUNTY LIBRARY
108 W. MOORE STREET
SOUTHPORT, NORTH CAROLINA 28461

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JUNE 1-30 (86-16): THIS ROUTINE SAFETY INSPECTION INVOLVED THE AREAS OF FOLLOWUP ON UNRESOLVED ITEM, MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, AND ONSITE FOLLOWUP OF EVENTS. TWO VIOLATIONS - FAILURE TO TAKE ADEQUATE CORRECTIVE ACTION CONCERNING JUMPER CONTROL; FAILURE TO FOLLOW PROCEDURES BY NOT DECLARING A SUPPORT INOPERABLE WHEN REQUIRED.

INSPECTION JUNE 9-13 (86-17): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED IN THE AREAS OF POST-REFUELING INITIAL CRITICALITY AND TESTS AND ROUTINE CORE PERFORMANCE SURVEILLANCE. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 8-10 (86-19): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED A REVIEW OF ALLEGATIONS RELATED TO THE CONDUCT OF OPERATIONS AT THE LICENSEE'S FACILITY. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATIONS.

LAST IE SITE INSPECTION DATE: JULY 8-10, 1986 +

INSPECTION REPORT NO: 50-324/86-19 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-014	04/29/86	05/20/86	UPSCALE TRIP OF REACTOR POWER INTERMEDIATE RANGE MONITOR 'D' OCCURRED WHEN CONTROL ROD 02-19 WAS SELECTED IN REACTOR MANUAL CONTROL SYSTEM WHILE PERFORMING PT-18.1; CAUSE - ELECTRONIC NOISE SPIKE.
86-015	05/30/86	06/27/86	AUTO INITIATION OF CORE SPRAY AND RESIDUAL HEAT REMOVAL LOW PRESSURE COOLANT INJECTION SYSTEM AND EXCEEDING ALLOWED TIME FRAME FOR INOPERABILITY OF 2-B21-N021A.

1. Docket: 50-454 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: P. DANDREA (815) 234-5441 X2341
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1175
6. Design Electrical Rating (Net MWe): 1120
7. Maximum Dependable Capacity (Gross MWe): 1175
8. Maximum Dependable Capacity (Net MWe): 1129
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe): 1152
11. Reasons for Restrictions, If Any:
SPLIT FEEDWATER FLOW

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	7,656.0
13. Hours Reactor Critical	241.7	4,202.8	5,483.8
14. Rx Reserve Shtdwn Hrs	.0	21.7	37.8
15. Hrs Generator On-Line	236.1	4,152.1	5,344.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	615,385	12,234,086	15,573,278
18. Gross Elec Ener (MWH)	208,906	4,136,033	5,224,623
19. Net Elec Ener (MWH)	186,841	3,894,103	4,907,001
20. Unit Service Factor	31.7	81.6	69.8
21. Unit Avail Factor	31.7	81.6	69.8
22. Unit Cap Factor (MDC Net)	22.2	67.8	56.8
23. Unit Cap Factor (DER Net)	22.4	68.3	57.2
24. Unit Forced Outage Rate	68.3	12.8	11.8
25. Forced Outage Hours	507.9	610.6	713.2

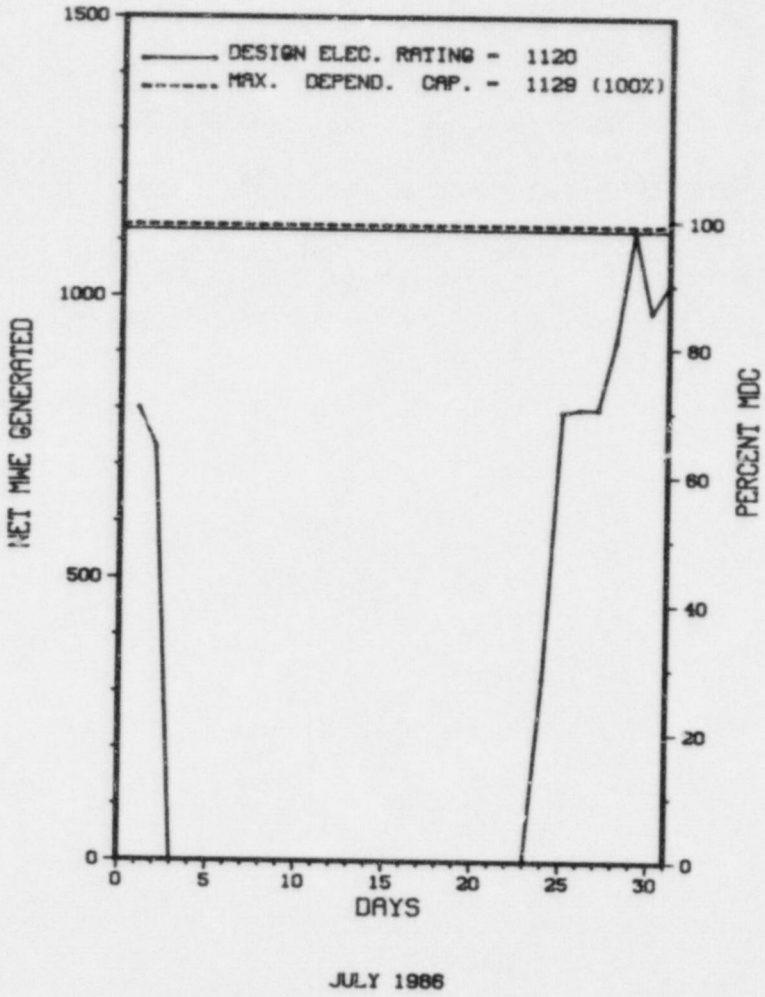
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* BYRON 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

BYRON 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* BYRON 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
10	07/03/86	F	507.9	A	1	86-020	RC	VALVE	PLANT SHUTDOWN DUE TO EXCESSIVE REACTOR COOLANT SYSTEM LEAKAGE FROM RESISTANCE TEMPERATURE DETECTOR ISOLATION VALVE.

****
* SUMMARY *
****

BYRON 1 OPERATED WITH 1 OUTAGE DUE TO EQUIPMENT FAILURE IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* BYRON 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....OGLE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...17 MI SW OF
ROCKFORD, ILL

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...FEBRUARY 2, 1985

DATE ELEC ENER 1ST GENER...MARCH 1, 1985

DATE COMMERCIAL OPERATE....SEPTEMBER 16, 1985

CONDENSER COOLING METHOD...CC HNDCT

CONDENSER COOLING WATER....ROCK RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....COMMONWEALTH EDISON

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....J. HINDS

LICENSING PROJ MANAGER.....L. OLSHAN
DOCKET NUMBER.....50-454

LICENSE & DATE ISSUANCE....NPF-37, FEBRUARY 14, 1985

PUBLIC DOCUMENT ROOM.....LIBRARIAN
BUSINESS SCIENCE & TECHNOLOGY DEPT.
ROCKFORD PUBLIC LIBRARY
215 NORTH WYMAN STREET
ROCKFORD, ILLINOIS 61101

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON MAY 31 THROUGH JUNE 30 (86021): ROUTINE UNANNOUNCED SAFETY INSPECTION BY THE RESIDENT INSPECTORS AND A HEADQUARTERS INSPECTOR OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; IEBS; SERS; LERS; OPERATIONS SUMMARY; SURVEILLANCE; MAINTENANCE; OPERATIONAL SAFETY AND ESF WALKDOWN; HEADQUARTERS AND REGION III REQUESTS; EVENT FOLLOWUP; LICENSEE ACTIONS CONCERNING SUSPECTED DRUG USE; MANAGEMENT MEETINGS; AND OTHER ACTIVITIES. OF THE 11 AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN 10 AREAS; TWO VIOLATIONS WERE IDENTIFIED IN THE REMAINING AREA; HOWEVER, IN ACCORDANCE WITH 10 CFR 2, APPENDIX C, SECTION V.A., A NOTICE OF VIOLATION WAS NOT ISSUED. NO ITEMS WERE IDENTIFIED WHICH COULD IMPACT THE PUBLIC HEALTH AND SAFETY.

INSPECTION ON JUNE 19-26 (86022): INCLUDED A REVIEW OF ALLEGATIONS RECEIVED BY THE NRC RESIDENT INSPECTOR OFFICE AT THE BYRON STATION OF PERCEIVED SECURITY VIOLATION COVER-UP. THE INSPECTION BEGAN DURING THE DAY SHIFT AND SOME OFF-SHIFT INSPECTION ACTIVITIES WERE PERFORMED. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THIS INSPECTION. THE ALLEGATIONS WERE NOT SUBSTANTIATED.

INSPECTION ON JUNE 24-26 (86023; 86020): INCLUDED A REVIEW OF LICENSEE SECURITY EVENT REPORT NO. 86-001, AND THE LICENSEE'S PROGRESS TOWARDS FULL IMPLEMENTATION OF THE SECURITY PROGRAM REQUIRED FOR THE LICENSING OF UNIT NO. 2. SPECIFICALLY, REGARDING UNIT 2, THE INSPECTION INCLUDED THE SPECIFIED SECTIONS 02 "INSPECTION REQUIREMENTS" OF THE 81100 SERIES INSPECTION PROCEDURE. THE REVIEW OF LICENSEE SECURITY EVENT REPORT NO. 86-001 SHOWED THAT THE EVENT WAS A VIOLATION OF SECURITY PLAN COMMITMENTS REQUIRED FOR UNIT 1, BUT THAT THE VIOLATION MET ALL OF THE CRITERIA FOR NOT ISSUING A NOTICE OF VIOLATION IN ACCORDANCE WITH THE "GENERAL

INSPECTION SUMMARY

POLICY AND PROCEDURES FOR NRC ENFORCEMENT ACTIONS," 10 CFR PART 2, APPENDIX C (1985). THE LICENSEE'S PROGRESS IN THE IMPLEMENTATION OF SECURITY FOR UNIT NO. 2 APPEARS ADEQUATE AT THIS TIME. NO VIOLATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

R. KROWZACK REPLACED T. BLACKMAN AS GSEP COORDINATOR FOR COMMONWEALTH EDISON APPROXIMATELY APRIL 1, 1986.

PLANT STATUS:

REDUCED POWER IS DUE TO DISCRETIONARY LOAD REDUCTION TO MINIMIZE COOLANT TEMPERATURES.

LAST IE SITE INSPECTION DATE: 08/08/86

INSPECTION REPORT NO: 86029

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-19	06/30/86	06/30/86	TECHNICAL SPECIFICATION ACTION STATEMENT FOR POWER RANGE NUCLEAR INSTRUMENT NOT SATISFIED DUE TO PERSONNEL ERROR
86-20	07/02/86	07/02/86	PLANT SHUTDOWN DUE TO EXCESSIVE REACTOR COOLANT SYSTEM LEAKAGE FROM RESISTANCE TEMPERATURE DETECTOR ISOLATION VALVE
86-21	06/14/86	07/14/86	TECHNICAL SPECIFICATION ACTION STATEMENT EXCEEDED DUE TO MISINTERPRETATION OF THE REQUIREMENT
86-22	07/03/86	07/03/86	NUCLEAR INSTRUMENTATION ACTION RESPONSE EXCEEDED DUE TO PERSONNEL ERROR

1. Docket: 50-483 O P E R A T I N G S T A T U S

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: DAN PRATTE (314) 676-8460

4. Licensed Thermal Power (MWt): 3411

5. Nameplate Rating (Gross MWe): 1373 X .9 = 1236

6. Design Electrical Rating (Net MWe): 1171

7. Maximum Dependable Capacity (Gross MWe): 1174

8. Maximum Dependable Capacity (Net MWe): 1120

9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	14,149.5
13. Hours Reactor Critical	718.6	3,681.3	12,144.8
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	697.9	3,510.4	11,697.8
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,279,207	10,964,179	36,891,673
18. Gross Elec Ener (MWH)	761,723	3,679,861	12,493,804
19. Net Elec Ener (MWH)	723,828	3,490,183	11,858,970
20. Unit Service Factor	93.8	69.0	82.7
21. Unit Avail Factor	93.8	69.0	82.7
22. Unit Cap Factor (MDC Net)	86.9	61.3	74.8
23. Unit Cap Factor (DER Net)	83.1	58.6	71.6
24. Unit Forced Outage Rate	6.2	9.6	7.2
25. Forced Outage Hours	46.1	373.1	909.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

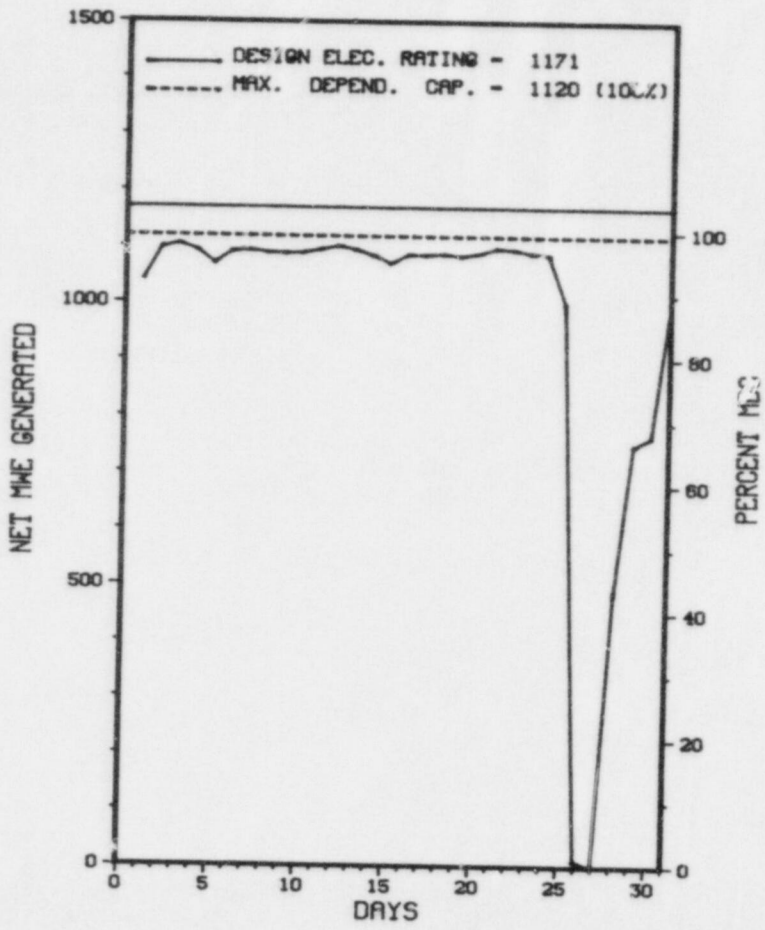
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* CALLAWAY 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

CALLAWAY 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* CALLAWAY 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
15	07/25/86	F	0.0	A	5				REDUCTION IN POWER DUE TO TURBINE RUNBACK ON LOSS OF CIRCULATING WATER.
16	07/16/86	F	46.1	A	3				TURBINE TRIP/REACTOR TRIP DURING TURBINE SURVEILLANCE TESTING. LER 86-027-00.

****
* SUMMARY *
****

CALLAWAY OPERATED WITH 1 REDUCTION AND 1 OUTAGE FOR EQUIPMENT FAILURE IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* CALLAWAY 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....MISSOURI

COUNTY.....CALLAWAY

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...10 MI SE OF
FULTON, MO

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...OCTOBER 2, 1984

DATE ELEC ENER 1ST GENER...OCTOBER 24, 1984

DATE COMMERCIAL OPERATE....DECEMBER 19, 1984

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....MISSOURI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....UNION ELECTRIC

CORPORATE ADDRESS.....P.O. BOX 149
ST LOUIS, MISSOURI 63166

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....DANIEL INTERNATIONAL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....B. LITTLE

LICENSING PROJ MANAGER.....T. ALEXION
DOCKET NUMBER.....50-483

LICENSE & DATE ISSUANCE....NPF-30, OCTOBER 18, 1984

PUBLIC DOCUMENT ROOM.....CALLAWAY COUNTY PUBLIC LIBRARY
WASHINGTON UNIVERSITY
JOHN M. OLIN LIBRARY
SKINKER & LINDELL BLVD.
ST. LOUIS, MO. 63130

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON JUNE 4 THROUGH 11 (86017): A SPECIAL UNANNOUNCED SAFETY INSPECTION BY THE SENIOR RESIDENT INSPECTOR REGARDING THE BLOCKING OF AUXILIARY FEEDWATER ACTUATION INSTRUMENTATION AND NRC REGION III MANAGEMENT SITE VISIT. ONE APPARENT VIOLATION WAS IDENTIFIED (FAILURE TO MAINTAIN THE AUXILIARY FEEDWATER ACTUATION SYSTEM INSTRUMENTATION OPERABLE WHILE IN MODES 1 AND 2).

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

OTHER ITEMS

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT HAS BEEN OPERATING AT POWER LEVELS UP TO 100%

LAST IE SITE INSPECTION DATE: 09/27/86

INSPECTION REPORT NO: 86019

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-19	06/06/86	07/30/86	LOW PRESSURIZER PRESSURE REACTOR TRIP
86-20	06/14/86	07/11/86	ENGINEERED SAFETY FEATURES ACTUATIONS ON CONTAINMENT NOBLE GAS ACTIVITY
86-21	06/17/86	07/17/86	PERSONNEL ERROR CAUSES T/S 3.3.3.1 VIOLATION AFTER RAD MONITOR FAILED
86-22	06/21/86	07/21/86	S/G LOW LEVEL REACTOR TRIP WHEN FEEDWATER ISOLATION VALVE CARD FAILED
86-23	05/19/86	07/24/86	PERSONNEL OPENED PENETRATION IN MODE 4 - VIOLATION OF TECH SPECS 3/4.6.1.1 AND 3/4.6.3
86-24	07/01/86	07/30/86	TECHNICAL SPECIFICATION 3.0.3 ENTERED DUE TO A ROD CONTROL CARD FAILURE
86-26	07/07/86	08/01/86	TECH SPEC 125-VOLT BATTERIES NOT SURVEILLED DUE TO PERSONNEL ERROR

1. Docket: 50-317 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: H. PORTER (301) 260-4868
4. Licensed Thermal Power (MWt): 2700
5. Nameplate Rating (Gross MWe): 1020 X 0.9 = 918
6. Design Electrical Rating (Net MWe): 845
7. Maximum Dependable Capacity (Gross MWe): 860
8. Maximum Dependable Capacity (Net MWe): 825
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	98,460.0
13. Hours Reactor Critical	706.4	4,882.8	77,748.3
14. Rx Reserve Shtdwn Hrs	.0	.0	2,299.2
15. Hrs Generator On-Line	704.1	4,849.4	76,208.9
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,850,976	12,869,980	190,194,173
18. Gross Elec Ener (MWH)	607,564	4,324,370	62,927,147
19. Net Elec Ener (MWH)	580,549	4,143,440	60,059,742
20. Unit Service Factor	94.6	95.3	77.4
21. Unit Avail Factor	94.6	95.3	77.4
22. Unit Cap Factor (MDC Net)	94.6	98.7	74.3*
23. Unit Cap Factor (DER Net)	92.3	96.4	72.2
24. Unit Forced Outage Rate	5.4	1.7	7.8
25. Forced Outage Hours	39.9	81.6	6,376.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

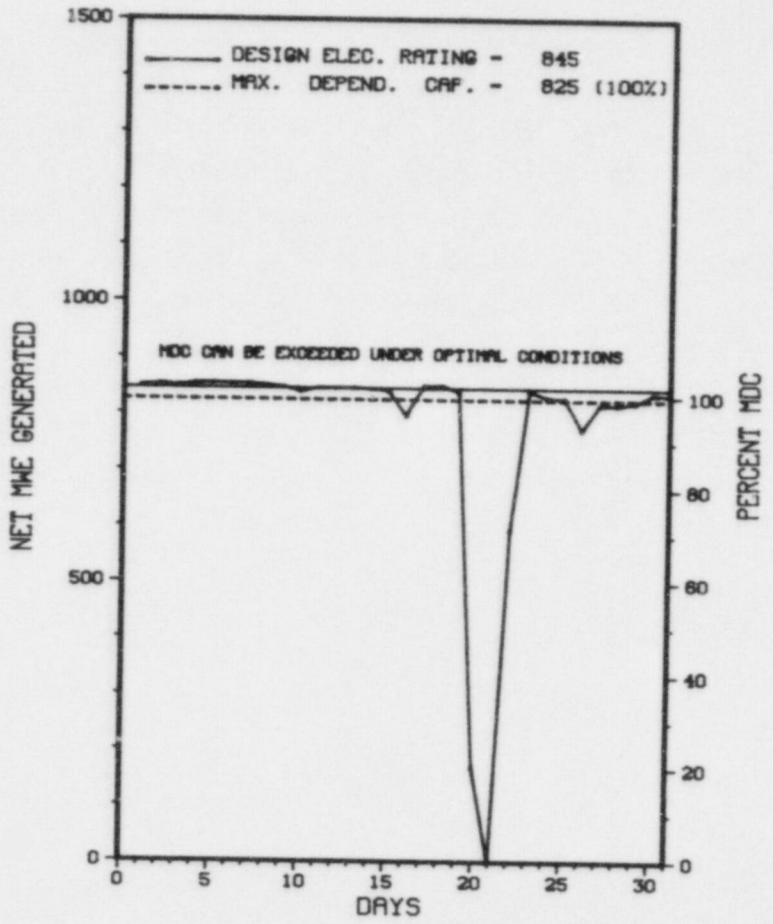
REFUELING: OCTOBER 25, 1986 - JANUARY 4, 1987.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* CALVERT CLIFFS 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

CALVERT CLIFFS 1



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

CALVERT CLIFFS 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-06	07/20/86	F	39.9	A	3	86-04	CB	CKTBKR	THE UNIT TRIPPED WHEN A SURGE CAPACITOR FOR 12B REACTOR COOLANT PUMP FAILED.

SUMMARY

CALVERT CLIFFS 1 OPERATED WITH 1 OUTAGE DUE TO EQUIPMENT FAILURE DURING JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* CALVERT CLIFFS 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....MARYLAND

COUNTY.....CALVERT

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...40 MI S OF
ANNAPOLIS, MD

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...OCTOBER 7, 1974

DATE ELEC ENER 1ST GENER...DECEMBER 30, 1974

DATE COMMERCIAL OPERATE....MAY 8, 1975

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CHESAPEAKE BAY

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....BALTIMORE GAS & ELEC

CORPORATE ADDRESS.....P.O. BOX 1475
BALTIMORE, MARYLAND 21203

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....T. FOLEY

LICENSING PROJ MANAGER.....S. MCNEIL
DOCKET NUMBER.....50-317

LICENSE & DATE ISSUANCE....DPR-53, JULY 31, 1974

PUBLIC DOCUMENT ROOM.....CALVERT COUNTY LIBRARY
FOURTH STREET
PRINCE FREDERICK, MARYLAND 20678

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

FAILURE TO FOLLOW SECURITY EVENT REPORTING PROCED URES. FAILURE TO INCLUDE AN INTRUSION DEVICE IN THE TES TING PROGRAM. TECHNICAL SPECIFICATIONS 4.6.3.1, 4.6.6.1, 4.7.6. 1, 4.7.7.1, AND 4.9.12 REQUIRE THAT A LABORATORY ANALYSIS OF A CARBON SAMPLE FROM AT LEAST TWO CAR BON SAMPLES REMOVED FROM ONE OF THE CHARCOAL ABSO RBERS DEMONSTRATES A REMOVAL EFFICIENCY OF GREATE R THAN 90% FOR RADIOACTIVE METHYL IODIDE. CONTRA RY TO THE ABOVE, ONLY ONE CARBON SAMPLE HAD BEEN REMOVED AND TESTED FOR SURVEILLANCE CONDUCTED DUR ING 1984-85. FAILURE TO FOLLOW SECURITY EVENT REPORTING PROCED URES. FAILURE TO INCLUDE AN INTRUSION DEVICE IN THE TES TING PROGRAM. TECHNICAL SPECIFICATIONS 4.6.3.1, 4.6.6.1, 4.7.6. 1, 4.7.7.1, AND 4.9.12 REQUIRE THAT A LABORATORY ANALYSIS OF A CARBON SAMPLE FROM AT LEAST TWO CAR BON SAMPLES REMOVED FROM ONE OF THE CHARCOAL ABSO RBERS DEMONSTRATES A REMOVAL EFFICIENCY OF GREATE R THAN 90% FOR RADIOACTIVE METHYL IODIDE. CONTRA RY TO THE ABOVE, ONLY ONE CARBON SAMPLE HAD BEEN REMOVED AND TESTED FOR SURVEILLANCE CONDUCTED DUR ING 1984-85. TECHNICAL SPECIFICATION 6.11 REQUIRES ADHERENCE TO RADIATION PROTECTION PROCEDURES. PROCEDURE 12 THP 6010.RAD.404, ESTABLISHING POSTED AREAS, REQUIRES ANY AREA IN WHICH THE REMOVABLE CONTAMINATION ON ANY ACCESSIBLE SURFACE OR EQUIPMENT EXCEEDS 500 DPM/100 CM2 BETA-GAMMA TO BE POSTED AS A CONTAMINATION AREA. CONTRARY TO THE ABOVE, ON MAY 6, 1986, THE INSPECTORS FOUND EQUIPMENT WITH REMOVABLE CONTAMINATION GREATER THAN 500 DPM/100 CM2 BETA-GAMMA LOCATED IN UNPOSTED AREAS ON THE 633-FOOT ELEVATION LEVEL OF THE AUXILIARY BUILDING, IN THAT: (A) PIPE FITTINGS STORED IN AN OPEN CABINET ADJACENT TO

ENFORCEMENT SUMMARY

THE FREON TOOL DECONTAMINATION AREA HAD REMOVABLE CONTAMINATION OF 4000 DPM/100 CM2 BETA-GAMMA; (B) A CART LOCATED ADJACENT TO THE HOT TOOL CRIB DECONTAMINATION AREA HAD REMOVABLE CONTAMINATION OF 5 000 DPM/100 CM2 BETA-GAMMA; (C) A SLAG HAMMER STORED ON A RACK AFFIXED TO THE OUTSIDE OF AN EQUIPMENT CABINET, LOCATED ADJACENT TO THE HOT TOOL CRIB, HAD REMOVABLE CONTAMINATION OF 220,000 DPM/100 CM2 BETA-GAMMA. TECHNICAL SPECIFICATION 6.8.1 REQUIRES ADHERENCE TO THE PROCEDURES COVERING PROCESS CONTROL PROGRAM IMPLEMENTATION. PROCEDURE 12 PMP 3150 PCP.001, RADIOACTIVE WASTE PROCESS CONTROL MANUAL, REQUIRES THAT ALL PLANT GENERATED RADIOACTIVE WASTES BE TRANSFERRED, PACKAGED, AND SHIPPED SUCH THAT RADIOACTIVE WASTE SHIPMENT AND BURIAL REGULATIONS ARE SATISFIED. CONTRARY TO THE ABOVE, ON SEPTEMBER 17, 1985, AT THE BARNWELL WASTE BURIAL FACILITY, A STATE OF SOUTH CAROLINA INSPECTOR FOUND THAT THE LICENSEE WAS IN VIOLATION OF SOUTH CAROLINA DEPARTMENT OF HEALTH AND ENVIRONMENTAL CONTROL REGULATION 61-83, SECTION 1.2, FOR RADIOACTIVE WASTE SHIPMENT NO. 0985-292-A FROM D. C. COOK TO BARNWELL, IN THAT: (A) THE DISPOSAL CONTAINER WAS FOUND TO HAVE ONE END OF THE LIFTING CABLE NOT PROPERLY ATTACHED, CONTRARY TO THE REQUIREMENTS OF CONDITION 64 OF SOUTH CAROLINA RADIOACTIVE MATERIAL LICENSE NO. 097; (B) THE SHIPPING CASK WAS FOUND TO CONTAIN LOOSE CONTAMINATED MATERIAL, CONTRARY TO REQUIREMENTS OF CONDITION 60 OF SOUTH CAROLINA RADIOACTIVE MATERIAL LICENSE NO. 097. 10 CFR 71.5 PROHIBITS TRANSPORT OF ANY LICENSED MATERIAL OUTSIDE THE CONFINES OF A PLANT OR OTHER PLACE OF USE OR DELIVERY OF LICENSED MATERIAL TO A CARRIER FOR TRANSPORT UNLESS THE LICENSEE COMPLIES WITH APPLICABLE REGULATIONS OF THE DEPARTMENT OF TRANSPORTATION IN 49 CFR PARTS 170-189. 49 CFR 173.425(B)(6) REQUIRES THAT EXCLUSIVE USE SHIPMENTS OF LOW SPECIFIC ACTIVITY (LSA) MATERIAL MUST BE BRACED SO AS TO PREVENT SHIFTING OF LADING. CONTRARY TO THE ABOVE, ON APRIL 25, 1986, AT THE BARNWELL WASTE BURIAL FACILITY, A STATE OF SOUTH CAROLINA INSPECTOR FOUND THAT, DUE TO INADEQUATE BLOCKING AND BRACING, ALL THREE PACKAGES ON THE FLATBED TRAILER HAD UNDERGONE A SHIFT OF LADING DURING THE TRANSPORTATION OF LSA EXCLUSIVE USE SHIPMENT NO. 0486-296-A FROM D. C. COOK TO BARNWELL.
(8600 4)

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

CALVERT CLIFFS 1

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT

NO INPUT PROVIDED.

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1. Docket: 50-318 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: R. PORTER (301) 260-4868
4. Licensed Thermal Power (MWt): 2700
5. Nameplate Rating (Gross MWe): 1012 X 0.9 = 911
6. Design Electrical Rating (Net MWe): 845
7. Maximum Dependable Capacity (Gross MWe): 860
8. Maximum Dependable Capacity (Net MWe): 825
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	81,815.0
13. Hours Reactor Critical	592.3	4,853.2	68,295.4
14. Rx Reserve Shtdwn Hrs	.0	24.3	1,285.2
15. Hrs Generator On-Line	591.6	4,841.7	67,252.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,557,132	12,449,838	168,931,873
18. Gross Elec Ener (MWH)	515,538	4,202,500	55,724,059
19. Net Elec Ener (MWH)	493,574	4,026,032	53,176,287
20. Unit Service Factor	79.5	95.2	82.2
21. Unit Avail Factor	79.5	95.2	82.2
22. Unit Cap Factor (MDC Net)	80.4	95.9	78.9*
23. Unit Cap Factor (DER Net)	78.5	93.7	76.9
24. Unit Forced Outage Rate	20.5	4.8	5.9
25. Forced Outage Hours	152.4	245.3	4,247.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

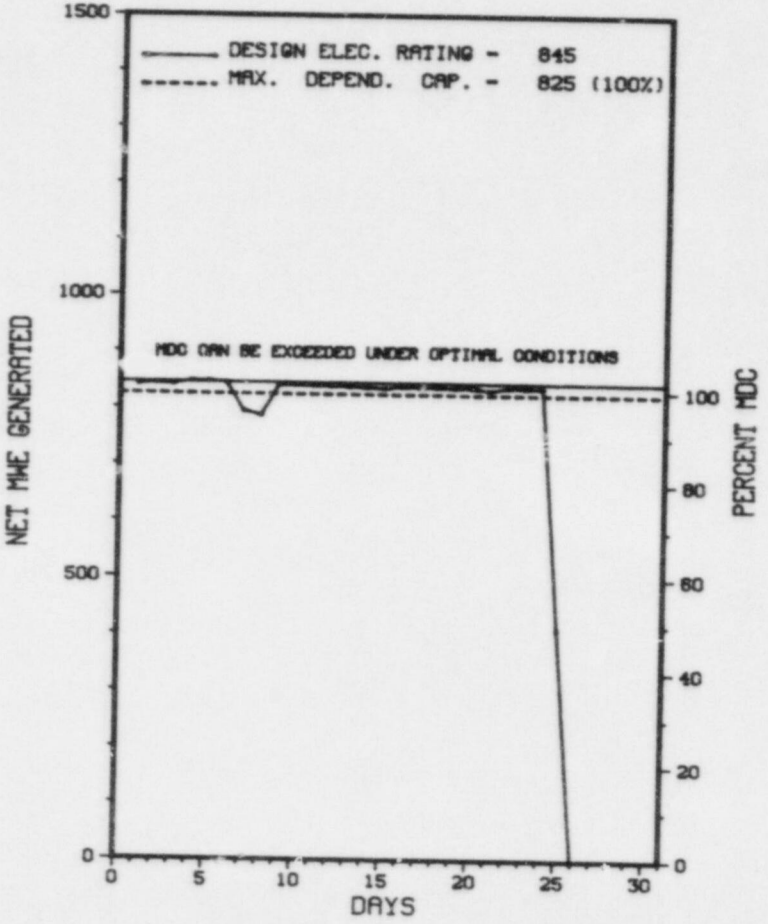
NONE

27. If Currently Shutdown Estimated Startup Date: 08/02/86

****
* CALVERT CLIFFS 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

CALVERT CLIFFS 2



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

U N I T S H U T D O W N S / R E D U C T I O N S

CALVERT CLIFFS 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-08	07/07/86	F	0.0	H	5				POWER WAS REDUCED WHEN THE LOAD DISPATCHER REQUESTED A REDUCTION DUE TO A PROBLEM ON THE GRID.
86-09	07/25/86	F	152.4	A	1	86-05	CB	INSTRU	THE UNIT WAS SHUTDOWN TO REPAIR A FAILED FLEXIBLE HOSE ON A SEAL PRESSURE TRANSMITTER FOR 21B REACTOR COOLANT PUMP.

SUMMARY

CALVERT CLIFFS 2 OPERATED WITH 1 REDUCTION AND SHUTDOWN FOR REPAIRS ON JULY 25TH.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* CALVERT CLIFFS 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....MARYLAND

COUNTY.....CALVERT

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...40 MI S OF
ANNAPOLIS, MD

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...NOVEMBER 30, 1976

DATE ELEC ENER 1ST GENER...DECEMBER 7, 1976

DATE COMMERCIAL OPERATE....APRIL 1, 1977

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CHESAPEAKE BAY

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....BALTIMORE GAS & ELEC

CORPORATE ADDRESS.....P.O. BOX 1475
BALTIMORE, MARYLAND 21203

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....T. FOLEY

LICENSING PROJ MANAGER.....S. MCNEIL
DOCKET NUMBER.....50-318

LICENSE & DATE ISSUANCE....DPR-69, NOVEMBER 30, 1976

PUBLIC DOCUMENT ROOM.....CALVERT COUNTY LIBRARY
FOURTH STREET
PRINCE FREDERICK, MARYLAND 20678

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-413 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: J. A. REAVIS (704) 373-7567
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1205
6. Design Electrical Rating (Net MWe): 1145
7. Maximum Dependable Capacity (Gross MWe): 1145
8. Maximum Dependable Capacity (Net MWe): 1145
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	9,552.0
13. Hours Reactor Critical	532.2	4,173.4	7,785.8
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	517.6	4,030.4	7,546.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,454,365	12,632,503	23,322,683
18. Gross Elec Ener (MWH)	502,127	4,451,740	8,136,009
19. Net Elec Ener (MWH)	460,250	4,151,818	7,592,332
20. Unit Service Factor	69.6	79.2	79.0
21. Unit Avail Factor	69.6	79.2	79.0
22. Unit Cap Factor (MDC Net)	54.0	71.3	69.4
23. Unit Cap Factor (DER Net)	54.0	71.3	69.4
24. Unit Forced Outage Rate	30.4	20.8	21.0
25. Forced Outage Hours	226.4	1,056.6	2,005.9

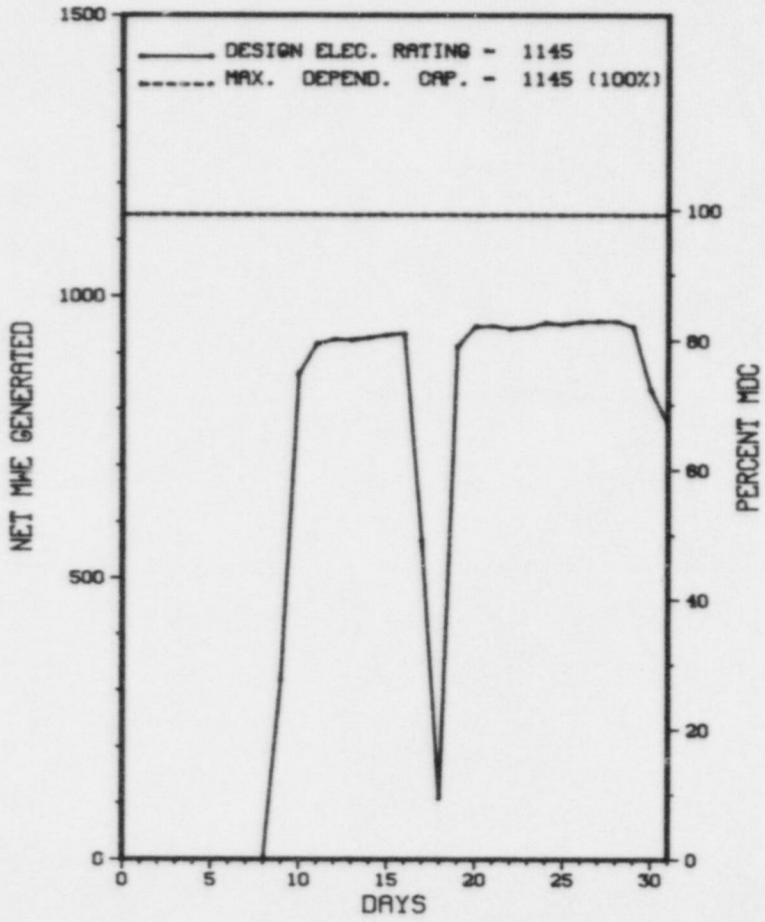
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
REFUELING - AUGUST 9, 1986 - 8 WEEKS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* CATAWBA 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

CATAWBA 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* CATAWBA 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
11	06/13/86	F	201.1	A	4		CJ	PIPEXX	REACTOR COOLANT LEAKAGE OUT OF SPEC DUE TO RUPTURED SOCKET-WELD ON LETDOWN.
60-P	07/09/86	S	0.0	B	5		CC	VALVEX	POWER INCREASE DELAY DUE TO CONTROL VALVE MOVEMENT TEST.
61-P	07/09/86	F	0.0	H	5		CG	ACCUMU	POWER INCREASE DELAY DUE TO INSUFFICIENT RECYCLE HOLDUP TANK VOLUME.
62-P	07/10/86	S	0.0	F	5		RC	ZZZZZZ	CORE COASTDOWN.
12	07/17/86	F	25.3	A	3		ED	INSTRU	REACTOR TRIP DUE TO RELAY LIGHT SOCKET SHORT RESULTING IN TURBINE LOAD IMBALANCE TRIP.
63-P	07/18/86	F	0.0	D	5		RC	XXXXXX	POWER INCREASE DELAY DUE TO AXIAL FLUX DIFFERENCE TIME ACQUIRED.
64-P	07/18/86	S	0.0	F	5		RC	ZZZZZZ	CORE COASTDOWN.
65-P	07/29/86	S	0.0	F	5		RC	ZZZZZZ	CORE COASTDOWN.

****
* SUMMARY *
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CATAWBA 1 OPERATED WITH 2 OUTAGES AND 6 REDUCTIONS LISTED IN DETAIL ABOVE.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training	5-Reduced Load	Licensee Event Report
	& License Examination	9-Other	(LER) File (NUREG-0161)

****
* CATAWBA 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....SOUTH CAROLINA

COUNTY.....YORK

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...6 MI NNW OF
ROCK HILL, SC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JANUARY 7, 1985

DATE ELEC ENER 1ST GENER...JANUARY 22, 1985

DATE COMMERCIAL OPERATE....JUNE 29, 1985

CONDENSER COOLING METHOD...MDCT

CONDENSER COOLING WATER....LAKE WYLIE

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUKE POWER

CORPORATE ADDRESS.....422 SOUTH CHURCH STREET
CHARLOTTE, NORTH CAROLINA 28242

CONTRACTOR
ARCHITECT/ENGINEER.....DUKE POWER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....DUKE POWER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....P. SKINNER

LICENSING PROJ MANAGER.....K. JABBOUR
DOCKET NUMBER.....50-413

LICENSE & DATE ISSUANCE....NPF-35, JANUARY 17, 1985

PUBLIC DOCUMENT ROOM.....YORK COUNTY LIBRARY
138 E. BLACK STREET
ROCK HILL, SOUTH CAROLINA 29730

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION APRIL 26 - MAY 25 (86-19): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED ON SITE IN THE AREAS OF REVIEW OF PLANT OPERATIONS; SURVEILLANCE OBSERVATION; MAINTENANCE OBSERVATION; REVIEW OF LICENSEE NONROUTINE EVENT REPORTS; AND FOLLOWUP OF PREVIOUSLY IDENTIFIED ITEMS. OF THE SIX (6) AREAS INSPECTED, NO APPARENT VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MAY 13-16 AND JUNE 2 (86-20): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED ONSITE INSPECTION IN THE AREA OF RADIATION PROTECTION INCLUDING: STARTUP SHIELDING SURVEYS; INTERNAL EXPOSURE CONTROL AND ASSESSMENT; EXTERNAL EXPOSURE CONTROL AND PERSONAL DOSIMETRY; RADIOACTIVE MATERIALS CONTROL, POSTING AND LABELING; SOLID RADIOACTIVE WASTE CLASSIFICATION AND CHARACTERIZATION; TRANSPORTATION OF RADIOACTIVE MATERIALS; AND PROGRAM FOR MAINTAINING EXPOSURE AS LOW AS REASONABLY ACHIEVABLE (ALARA). ONE VIOLATION - FAILURE TO MAINTAIN RADIATION EXPOSURE RECORDS IN ACCORDANCE WITH INSTRUCTIONS CONTAINED ON FORM NRC-5.

INSPECTION MAY 8-13, 19-23 AND 27-29 (86-21): THIS ROUTINE, UNANNOUNCED INSPECTION ADDRESSED THE AREAS OF REVIEW OF PRECRITICAL DATA, WITNESSING OF INITIAL CRITICALITY, WITNESSING OF STARTUP TESTS AND REVIEW OF PROPOSED TESTS FOR UNIT 2, AND REVIEW OF SURVEILLANCE TESTS FOR UNIT 1. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MAY 26 - JUNE 25 (86-24): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED ON SITE INSPECTING IN THE AREAS OF REVIEW OF PLANT OPERATIONS; SURVEILLANCE OBSERVATION; MAINTENANCE OBSERVATION; REVIEW OF LICENSEE NONROUTINE EVENT REPORTS; PROCEDURES; SURVEY OF LICENSEE RESPONSE TO SAFETY ISSUES; FOLLOWUP OF INFORMATION NOTICES; AND PREPARATIONS FOR REFUELING; OF THE 8 AREAS INSPECTED, 2 APPARENT VIOLATIONS WERE IDENTIFIED, (FAILURE TO FOLLOW PROCEDURES; AND FAILURE TO PROVIDE ADEQUATE PROCEDURES).

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NONE.

LAST IE SITE INSPECTION DATE: MAY 26 - JUNE 25, 1986 +

INSPECTION REPORT NO: 50-413/86-24 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-021	04/17/86	05/16/86	INADVERTENT START OF TURBINE DRIVEN AUXILIARY FEEDWATER PUMP DUE TO DEFECTIVE PROCEDURE.
86-022	04/19/86	05/19/86	REACTOR TRIPPED WHEN ALL 3 FLOW TRANSMITTERS ON LOOP C OF REACTOR COOLANT SYSTEM INDICATED A LOW FLOW; CAUSE - DEFECTIVE PROCEDURE.
86-028	06/06/86	07/07/86	UNIT SHUTDOWN DUE TO UNIDENTIFIED REACTOR COOLANT LEAKAGE; CAUSE - UNKNOWN.
86-029	06/06/86	07/07/86	ESF ACTUATION OCCURRED DURING TESTING OF THE 2B SEQUENCER; CAUSE - UNKNOWN.
86-030	06/11/86	07/11/86	REACTOR TRIP ON LOW-LOW STEAM GENERATOR LEVEL DUE TO PERSONNEL ERROR.
86-031	06/13/86	07/11/86	WELD BREAK ON LETDOWN PIPING RESULTED IN EXCESSIVE REACTOR COOLANT SYSTEM LEAKAGE CAUSING A FORCED SHUTDOWN; CAUSE - DESIGN, MANUFACTURING, CONSTRUCTION/INSTALLATION DEFICIENCY.

1. Docket: 50-315 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: C. ZAJKA (616) 465-5901
4. Licensed Thermal Power (MWt): 3250
5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152
6. Design Electrical Rating (Net MWe): 1030
7. Maximum Dependable Capacity (Gross MWe): 1056
8. Maximum Dependable Capacity (Net MWe): 1020
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	101,519.0
13. Hours Reactor Critical	373.2	3,900.5	72,200.2
14. Rx Reserve Shtdwn Hrs	.0	.0	463.0
15. Hrs Generator On-Line	337.4	3,864.7	70,717.3
16. Unit Reserve Shtdwn Hrs	.0	.0	321.0
17. Gross Therm Ener (MWH)	819,019	11,160,991	206,156,644
18. Gross Elec Ener (MWH)	248,760	3,612,600	67,590,140
19. Net Elec Ener (MWH)	236,794	3,473,282	65,020,439
20. Unit Service Factor	45.3	76.0	71.0
21. Unit Avail Factor	45.3	76.0	71.0
22. Unit Cap Factor (MDC Net)	31.2	66.9	64.0
23. Unit Cap Factor (DER Net)	30.9	66.3	61.7
24. Unit Forced Outage Rate	54.6	24.0	8.9
25. Forced Outage Hours	405.6	1,221.3	6,164.2

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

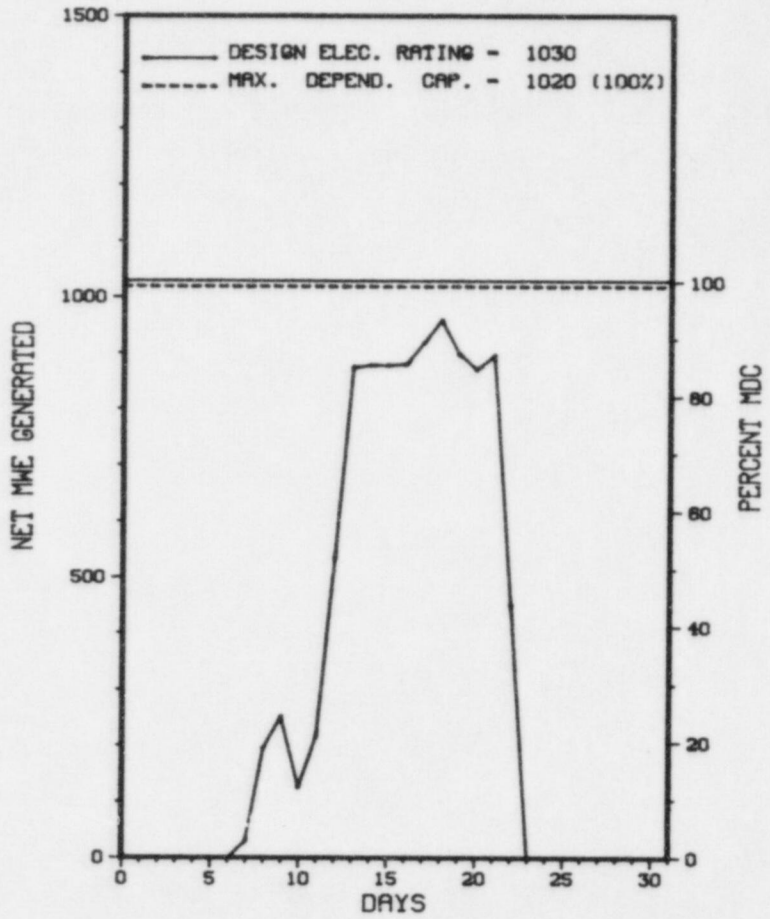
NONE

27. If Currently Shutdown Estimated Startup Date: 08/15/86

****
* COOK 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

COOK 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* COOK 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
250	05/28/86	F	163.6	A	4	86-012-0	HA	INSTRU	THE OUTAGE WHICH BEGAN ON 05/28/86 FOLLOWING A TURBINE TRIP FROM 90% RTP HAS BEEN COMPLETED. THE UNIT WAS RETURNED TO SERVICE AT 1937 HOURS ON 07/07/86 AND WAS HELD AT 25% FOR TURBINE ROTOR WARMING IN PREPARATION FOR TURBINE OVERSPEED TESTING.
251	07/07/86	S	1.0	B	1		ZZ	ZZZZZZ	THE TURBINE GENERATOR WAS REMOVED FROM PARALLEL AT 0307 HOURS FOR TURBINE OVERSPEED TESTING. THE UNIT WAS RETURNED TO SERVICE AT 0405 HOURS THE SAME DAY.
252	07/10/86	F	15.1	B	1	86-007	ZZ	ZZZZZZ	THE UNIT WAS REMOVED FROM SERVICE WHEN IT WAS DETERMINED THAT A NON-COMPLIANCE EXISTED ON TWO TECHNICAL SPECIFICATION REQUIRED SURVEILLANCE TESTS. THE TESTING WAS COMPLETED AND THE UNIT RETURNED TO SERVICE ON 07/11/86.
253	07/22/86	F	226.9	G	3		ZZ	ZZZZZZ	A TURBINE/REACTOR TRIP OCCURRED WHILE PERFORMING A ROUTINE CALIBRATION OF THE TURBINE GENERATOR STATOR COOLING WATER LOW PRESSURE TRIP. THE TRIP OCCURRED DUE TO INADEQUATE BLOCKING OF THE TRIP CIRCUIT.

****
* SUMMARY *
****

COOK 1 OPERATED WITH 4 OUTAGES IN JULY, AND SHUTDOWN FOR REPAIRS ON THE 27TH.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* COOK 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....MICHIGAN

COUNTY.....BERRIEN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...11 MI S OF
BENTON HARBOR, MI

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JANUARY 18, 1975

DATE ELEC ENER 1ST GENER...FEBRUARY 10, 1975

DATE COMMERCIAL OPERATE....AUGUST 27, 1975

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....EAST CENTRAL AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....INDIANA & MICHIGAN ELECTRIC

CORPORATE ADDRESS.....1 RIVERSIDE PLAZA
COLUMBUS, OHIO 43216

CONTRACTOR
ARCHITECT/ENGINEER.....AMERICAN ELEC. POWER SERVICE CORP.

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....AMERICAN ELEC. POWER SERVICE CORP.

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....B. JORGENSEN

LICENSING PROJ MANAGER.....D. WIGGINTON
DOCKET NUMBER.....50-315

LICENSE & DATE ISSUANCE....DPR-58, OCTOBER 25, 1974

PUBLIC DOCUMENT ROOM.....MAUDE PRESTON PALENSKE MEMORIAL LIBRARY
500 MARKET STREET
ST. JOSEPH, MICHIGAN 49085

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION DURING THE PERIOD APRIL 7 THROUGH MAY 29 (86001; 86001): UNANNOUNCED INSPECTION OF THE RADIATION PROTECTION PROGRAM DURING A REFUELING AND MAINTENANCE OUTAGE INCLUDING: STATUS OF THE RADIATION PROTECTION IMPROVEMENT PROGRAM; CHANGES IN ORGANIZATION, PERSONNEL, FACILITIES, EQUIPMENT, PROGRAMS, AND PROCEDURES; AUDITS AND APPRAISALS; PLANNING AND PREPARATION; TRAINING AND QUALIFICATIONS OF NEW PERSONNEL; INTERNAL AND EXTERNAL EXPOSURE CONTROL; CONTROL OF RADIOACTIVE MATERIALS AND CONTAMINATION, SURVEYS, AND MONITORING; AND THE ALARA PROGRAM. ALSO, CERTAIN TMI ACTION PLAN ITEMS, OPEN ITEMS, RADIATION PROTECTION STAFF STABILITY, USE OF THE HNS WASTE SORTER, IE INFORMATION NOTICES NO. 85-61 AND 86-22, AND CERTAIN RADIOACTIVE MATERIAL SHIPMENT CONCERNS IDENTIFIED AT THE BARNWELL WASTE BURIAL FACILITY BY STATE OF SOUTH CAROLINA INSPECTORS WERE REVIEWED. THREE VIOLATIONS WERE IDENTIFIED (FAILURE TO COMPLY WITH RADIOACTIVE WASTE BURIAL SITE REGULATIONS; FAILURE TO PREVENT SHIFTING OF RADIOACTIVE WASTE SHIPMENT LADING; AND FAILURE TO POST CONTAMINATED AREAS).

INSPECTION ON JUNE 9-11 (86016; 86016): ROUTINE, ANNOUNCED INSPECTION OF THE ANNUAL D. C. COOK EMERGENCY PREPAREDNESS EXERCISE INVOLVING OBSERVATIONS BY FIVE NRC REPRESENTATIVES OF KEY FUNCTIONS AND LOCATIONS DURING THE EXERCISE. THE INSPECTION WAS CONDUCTED BY THREE NRC INSPECTORS AND TWO CONSULTANTS. ALTHOUGH NO ITEMS OF NONCOMPLIANCE, DEFICIENCIES, OR DEVIATIONS WERE IDENTIFIED, FIVE EXERCISE WEAKNESSES WERE IDENTIFIED AND SUMMARIZED IN THE ATTACHMENT TO THE REPORT'S TRANSMITTAL LETTER.

INSPECTION ON MAY 13 THROUGH JUNE 16 (86022; 86022): ROUTINE, UNANNOUNCED INSPECTION BY THE RESIDENT INSPECTORS OF LICENSEE ACTIONS ON PREVIOUSLY IDENTIFIED ITEMS; OPERATIONAL SAFETY VERIFICATION; REACTOR TRIP; MAINTENANCE; SURVEILLANCE; REPORTABLE EVENTS; REGIONAL REQUESTS; AND MISCELLANEOUS INSPECTION. OF THE EIGHT AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN SEVEN AREAS. ONE VIOLATION (PROCEDURE FAILED TO PROPERLY RESTORE SYSTEM AFTER TEST) WAS IDENTIFIED IN THE REMAINING

INSPECTION SUMMARY

AREA.

INSPECTION ON JUNE 26 (86026): ROUTINE, UNANNOUNCED INSPECTION OF THE RESOLUTION OF AN IEIE BULLETIN. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

UNIT 1 TECHNICAL SPECIFICATION 6.8.1.A REQUIRES WRITTEN PROCEDURES BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING THE APPLICABLE PROCEDURES RECOMMENDED IN REGULATORY GUIDE 1.33 APPENDIX A, NOVEMBER 1972, WHICH INCLUDES ADMINISTRATIVE PROCEDURES FOR EQUIPMENT CONTROL AND FOR PROCEDURE REVIEW AND APPROVAL. PLANT MANAGER'S INSTRUCTION PMI-2010, "PLANT MANAGER AND DEPARTMENT HEAD INSTRUCTIONS, PROCEDURES, AND ASSOCIATED INDEXES," AN ADMINISTRATIVE PROCEDURE FOR EQUIPMENT CONTROL AND PROCEDURE REVIEW AND APPROVAL REQUIRES IN PARAGRAPH 3.7.1 THAT PROCEDURES BE PREPARED UTILIZING THE FORMAT SPECIFIED IN THE ASSOCIATED ATTACHMENTS 1 AND 2. ATTACHMENT 2, "FORMAT OF PROCEDURES" AT PARAGRAPH 4.I ENTITLED, "RESTORATION" REQUIRES STEPS BE PROVIDED TO ASSURE NECESSARY RESTORATION OF ALL CONTROLS AND EQUIPMENT WHOSE STATUS WAS CHANGED DURING THE PROCEDURE. CONTRARY TO THE ABOVE, LICENSEE PROCEDURE 1 THP 6030 IMP.076, "INCORE THERMOCOUPLE AND REACTOR COOLANT SYSTEM RTD CROSS CALIBRATION PROCEDURE" PROVIDES FOR INSTALLATION OF CALIBRATED PRESSURE GAUGES AT VARIOUS POINTS IN THE MAIN STEAM SYSTEM, BUT DOES NOT CONTAIN STEPS TO REMOVE THE INSTALLED GAUGES ON COMPLETION OF THE TEST. AS A CONSEQUENCE, FOLLOWING COMPLETION OF THE SUBJECT TEST ON OCTOBER 31, 1985 WITH THE PLANT IN MODE 3, FOUR PRESSURE GAUGES, ONE UPSTREAM OF EACH OF THE FOUR MAIN STEAM ISOLATION VALVES, WERE NOT REMOVED. THE PLANT SUBSEQUENTLY OPERATED IN MODE 1 FOR AN EXTENDED PERIOD WITH THE GAUGES STILL INSTALLED. SOME ITEMS WERE NOT SEARCHED OR SECURED AT REQUIRED BY THE SECURITY PLAN.
(8602 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

SHUTDOWN FOR REPAIRS.

LAST IE SITE INSPECTION DATE: 08/22/86

INSPECTION REPORT NO: 86031

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
85-48	09/17/85	07/01/86	INCORRECT REPLACEMENT PART INSTALLED ON THE TURBINE DRIVEN FEEDWATER PUMP, CAUSED BY INADEQUATE PROGRAM CONTROLS
85-64	11/23/85	07/15/86	NON-CONSERVATIVE FLUX RATE TRIP SETPOINTS DUE TO INADEQUATE CALIBRATION PROCEDURES
85-69	11/17/85	07/07/86	OPERATION WITH INOPERABLE INTERMEDIATE RANGE NEUTRON FLUX DETECTOR, CAUSED BY INSTRUMENT DRIFT
86-13	05/13/86	07/14/86	FAILURE TO LITERALLY COMPLY WITH TECHNICAL SPECIFICATION REQUIREMENTS DUE TO PROCEDURAL DEFICIENCIES
86-14	06/28/86	07/28/86	FAILURE TO VERIFY RESIDUAL HEAT REMOVAL SYSTEM INTERLOCK ACTION DUE TO PROCEDURAL DEFICIENCY

INSPECTION SUMMARY

AREA.

INSPECTION ON JUNE 26 (86026): ROUTINE, UNANNOUNCED INSPECTION OF THE RESOLUTION OF AN IEIE BULLETIN. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

UNIT 1 TECHNICAL SPECIFICATION 6.8.1.A REQUIRES WRITTEN PROCEDURES BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING THE APPLICABLE PROCEDURES RECOMMENDED IN REGULATORY GUIDE 1.33 APPENDIX A, NOVEMBER 1972, WHICH INCLUDES ADMINISTRATIVE PROCEDURES FOR EQUIPMENT CONTROL AND FOR PROCEDURE REVIEW AND APPROVAL. PLANT MANAGER'S INSTRUCTION PMI-2010, "PLANT MANAGER AND DEPARTMENT HEAD INSTRUCTIONS, PROCEDURES, AND ASSOCIATED INDEXES," AN ADMINISTRATIVE PROCEDURE FOR EQUIPMENT CONTROL AND PROCEDURE REVIEW AND APPROVAL REQUIRES IN PARAGRAPH 3.7.1 THAT PROCEDURES BE PREPARED UTILIZING THE FORMAT SPECIFIED IN THE ASSOCIATED ATTACHMENTS 1 AND 2. ATTACHMENT 2, "FORMAT OF PROCEDURES" AT PARAGRAPH 4.I ENTITLED, "RESTORATION" REQUIRES STEPS BE PROVIDED TO ASSURE NECESSARY RESTORATION OF ALL CONTROLS AND EQUIPMENT WHOSE STATUS WAS CHANGED DURING THE PROCEDURE. CONTRARY TO THE ABOVE, LICENSEE PROCEDURE 1 THP 6030 IMP.076, "INCORE THERMOCOUPLE AND REACTOR COOLANT SYSTEM RTD CROSS CALIBRATION PROCEDURE" PROVIDES FOR INSTALLATION OF CALIBRATED PRESSURE GAUGES AT VARIOUS POINTS IN THE MAIN STEAM SYSTEM, BUT DOES NOT CONTAIN STEPS TO REMOVE THE INSTALLED GAUGES ON COMPLETION OF THE TEST. AS A CONSEQUENCE, FOLLOWING COMPLETION OF THE SUBJECT TEST ON OCTOBER 31, 1985 WITH THE PLANT IN MODE 3, FOUR PRESSURE GAUGES, ONE UPSTREAM OF EACH OF THE FOUR MAIN STEAM ISOLATION VALVES, WERE NOT REMOVED. THE PLANT SUBSEQUENTLY OPERATED IN MODE 1 FOR AN EXTENDED PERIOD WITH THE GAUGES STILL INSTALLED. SOME ITEMS WERE NOT SEARCHED OR SECURED AT REQUIRED BY THE SECURITY PLAN.
(8602 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

SHUTDOWN FOR REPAIRS.

LAST IE SITE INSPECTION DATE: 08/22/86

INSPECTION REPORT NO: 86031

REPORTS FROM LICENSEE

COOK 1

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
85-48	09/17/85	07/01/86	INCORRECT REPLACEMENT PART INSTALLED ON THE TURBINE DRIVEN FEEDWATER PUMP, CAUSED BY INADEQUATE PROGRAM CONTROLS
85-64	11/23/85	07/15/86	NON-CONSERVATIVE FLUX RATE TRIP SETPOINTS DUE TO INADEQUATE CALIBRATION PROCEDURES
85-69	11/17/85	07/07/86	OPERATION WITH INOPERABLE INTERMEDIATE RANGE NEUTRON FLUX DETECTOR, CAUSED BY INSTRUMENT DRIFT
86-13	05/13/86	07/14/86	FAILURE TO LITERALLY COMPLY WITH TECHNICAL SPECIFICATION REQUIREMENTS DUE TO PROCEDURAL DEFICIENCIES
86-14	06/28/86	07/28/86	FAILURE TO VERIFY RESIDUAL HEAT REMOVAL SYSTEM INTERLOCK ACTION DUE TO PROCEDURAL DEFICIENCY

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1. Docket: 50-316 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: C. ZAJKA (616) 465-5901

4. Licensed Thermal Power (MWt): 3411

5. Nameplate Rating (Gross MWe): 1333 X 0.85 = 1133

6. Design Electrical Rating (Net MWe): 1100

7. Maximum Dependable Capacity (Gross MWe): 1100

8. Maximum Dependable Capacity (Net MWe): 1060

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	75,215.0
13. Hours Reactor Critical	534.9	1,887.5	50,916.3
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	444.2	1,716.7	49,570.6
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	980,980	4,385,577	158,060,029
18. Gross Elec Ener (MWH)	294,480	1,392,260	51,078,670
19. Net Elec Ener (MWH)	278,587	1,332,059	49,233,409
20. Unit Service Factor	59.7	33.7	68.1
21. Unit Avail Factor	59.7	33.7	68.1
22. Unit Cap Factor (MDC Net)	35.3	24.7	63.8
23. Unit Cap Factor (DER Net)	34.0	23.8	62.4
24. Unit Forced Outage Rate	10.3	10.0	15.6
25. Forced Outage Hours	51.2	190.4	9,155.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

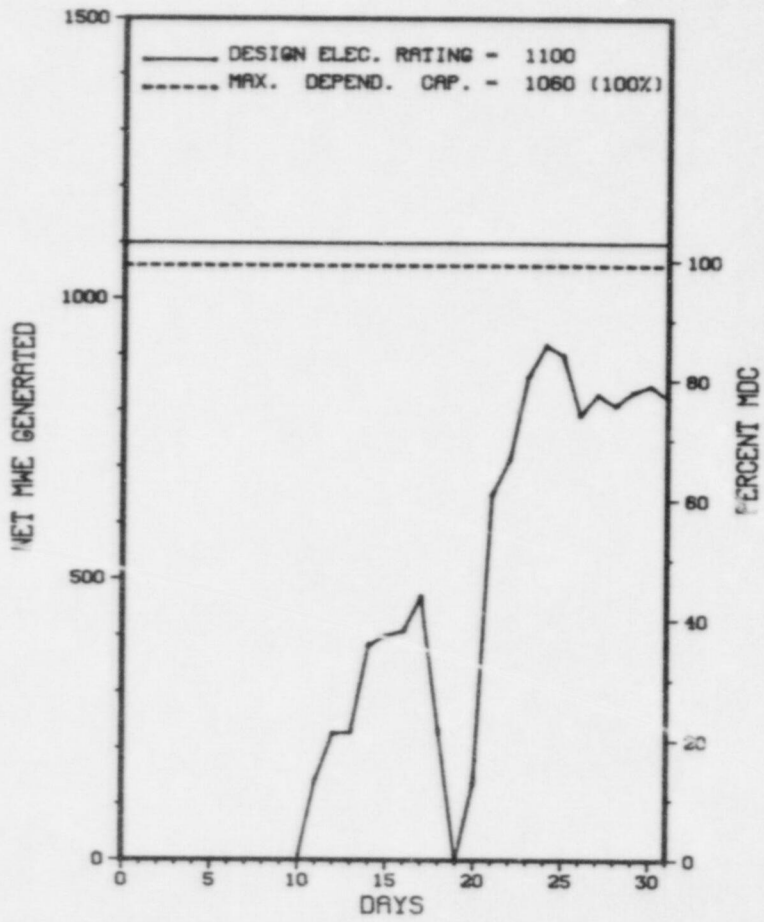
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* COOK 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

COOK 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

COOK 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
172	02/28/86	S	248.6	C	4		RC	FUELXX	THE UNIT WAS REMOVED FROM SERVICE ON 02/28/86 FOR THE SCHEDULED CYCLE 5 OR 6 REFUELING/MAINTENANCE OUTAGE. THE OUTAGE WORK HAS BEEN COMPLETED AND INITIAL CRITICALITY FOR CYCLE 6 OCCURRED ON 07/07/86.
173	07/18/86	F	51.2	A	3	LATER	CH	INSTRU	WHILE HOLDING REACTOR POWER AT 69% FOR THE POWER TESTING PROGRAM, A TURBINE/REACTOR TRIP OCCURRED DUE TO HIGH-HIGH LEVEL IN NO. 22 STEAM GENERATOR. THE HIGH LEVEL RESULTED FROM A FAILURE OF A FEED WATER FLOW TRANSMITTER.

SUMMARY

COOK 2 RETURNED ONLINE FROM REFUELING AND MAINTENANCE ON JULY 11TH, AND OPERATED WITH 1 ADDITIONAL OUTAGE FOR EQUIPMENT REPAIR.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* COOK 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....MICHIGAN

COUNTY.....BERRIEN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...11 MI S OF
BENTON HARBOR, MI

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MARCH 10, 1978

DATE ELEC ENER 1ST GENER...MARCH 22, 1978

DATE COMMERCIAL OPERATE....JULY 1, 1978

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....EAST CENTRAL AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....INDIANA & MICHIGAN ELECTRIC

CORPORATE ADDRESS.....1 RIVERSIDE PLAZA
COLUMBUS, OHIO 43216

CONTRACTOR
ARCHITECT/ENGINEER.....AMERICAN ELEC. POWER SERVICE CORP.

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....J. A. JONES CONSTRUCTION

TURBINE SUPPLIER.....BROWN BOVERI

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....B. JORGENSEN

LICENSING PROJ MANAGER.....D. WIGGINTON
DOCKET NUMBER.....50-316

LICENSE & DATE ISSUANCE....DPR-74, DECEMBER 23, 1977

PUBLIC DOCUMENT ROOM.....MAUDE PRESTON PALENSKE MEMORIAL LIBRARY
500 MARKET STREET
ST. JOSEPH, MICHIGAN 49085

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION DURING THE PERIOD APRIL 7 THROUGH MAY 29 (86001; 86001): UNANNOUNCED INSPECTION OF THE RADIATION PROTECTION PROGRAM DURING A REFUELING AND MAINTENANCE OUTAGE INCLUDING: STATUS OF THE RADIATION PROTECTION IMPROVEMENT PROGRAM; CHANGES IN ORGANIZATION, PERSONNEL, FACILITIES, EQUIPMENT, PROGRAMS, AND PROCEDURES; AUDITS AND APPRAISALS; PLANNING AND PREPARATION; TRAINING AND QUALIFICATIONS OF NEW PERSONNEL; INTERNAL AND EXTERNAL EXPOSURE CONTROL; CONTROL OF RADIOACTIVE MATERIALS AND CONTAMINATION, SURVEYS, AND MONITORING; AND THE ALARA PROGRAM. ALSO, CERTAIN TMI ACTION PLAN ITEMS, OPEN ITEMS, RADIATION PROTECTION STAFF STABILITY, USE OF THE HNS WASTE SORTER, IE INFORMATION NOTICES NO. 85-61 AND 86-22, AND CERTAIN RADIOACTIVE MATERIAL SHIPMENT CONCERNS IDENTIFIED AT THE BARNWELL WASTE BURIAL FACILITY BY STATE OF SOUTH CAROLINA INSPECTORS WERE REVIEWED. THREE VIOLATIONS WERE IDENTIFIED (FAILURE TO COMPLY WITH RADIOACTIVE WASTE BURIAL SITE REGULATIONS; FAILURE TO PREVENT SHIFTING OF RADIOACTIVE WASTE SHIPMENT LADING; AND FAILURE TO POST CONTAMINATED AREAS).

INSPECTION ON JUNE 9-11 (86016; 86016): ROUTINE, ANNOUNCED INSPECTION OF THE ANNUAL D. C. COOK EMERGENCY PREPAREDNESS EXERCISE INVOLVING OBSERVATIONS BY FIVE NRC REPRESENTATIVES OF KEY FUNCTIONS AND LOCATIONS DURING THE EXERCISE. THE INSPECTION WAS CONDUCTED BY THREE NRC INSPECTORS AND TWO CONSULTANTS. ALTHOUGH NO ITEMS OF NONCOMPLIANCE, DEFICIENCIES, OR DEVIATIONS WERE IDENTIFIED, FIVE EXERCISE WEAKNESSES WERE IDENTIFIED AND SUMMARIZED IN THE ATTACHMENT TO THE REPORT'S TRANSMITTAL LETTER.

INSPECTION ON MAY 13 THROUGH JUNE 16 (86022; 86022): ROUTINE, UNANNOUNCED INSPECTION BY THE RESIDENT INSPECTORS OF LICENSEE ACTIONS ON PREVIOUSLY IDENTIFIED ITEMS; OPERATIONAL SAFETY VERIFICATION; REACTOR TRIP; MAINTENANCE; SURVEILLANCE; REPORTABLE EVENTS; REGIONAL REQUESTS; AND MISCELLANEOUS INSPECTION. OF THE EIGHT AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN SEVEN AREAS. ONE VIOLATION (PROCEDURE FAILED TO PROPERLY RESTORE SYSTEM AFTER TEST) WAS IDENTIFIED IN THE REMAINING

INSPECTION SUMMARY

AREA.

INSPECTION OF JUNE 26 (86026): ROUTINE, UNANNOUNCED INSPECTION OF THE RESOLUTION OF AN IEIE BULLETIN. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 6.11 REQUIRES ADHERENCE TO RADIATION PROTECTION PROCEDURES. PROCEDURE 12 THP 6010.RAD.404, ESTABLISHING POSTED AREAS, REQUIRES ANY AREA IN WHICH THE REMOVABLE CONTAMINATION ON ANY ACCESSIBLE SURFACE OR EQUIPMENT EXCEEDS 500 DPM/100 CM2 BETA-GAMMA TO BE POSTED AS A CONTAMINATION AREA. CONTRARY TO THE ABOVE, ON MAY 6, 1986, THE INSPECTORS FOUND EQUIPMENT WITH REMOVABLE CONTAMINATION GREATER THAN 500 DPM/100 CM2 BETA-GAMMA LOCATED IN UNPOSTED AREAS ON THE 633-FOOT ELEVATION LEVEL OF THE AUXILIARY BUILDING, IN THAT: (A) PIPE FITTINGS STORED IN AN OPEN CABINET ADJACENT TO THE FREON TOOL DECONTAMINATION AREA HAD REMOVABLE CONTAMINATION OF 4000 DPM/100 CM2 BETA-GAMMA; (B) A CART LOCATED ADJACENT TO THE HOT TOOL CRIB DECONTAMINATION AREA HAD REMOVABLE CONTAMINATION OF 5 000 DPM/100 CM2 BETA-GAMMA; (C) A SLAG HAMMER STORED ON A RACK AFFIXED TO THE OUTSIDE OF AN EQUIPMENT CABINET, LOCATED ADJACENT TO THE HOT TOOL CRIB, HAD REMOVABLE CONTAMINATION OF 220,000 DPM/100 CM2 BETA-GAMMA. TECHNICAL SPECIFICATION 6.8.1 REQUIRES ADHERENCE TO THE PROCEDURES COVERING PROCESS CONTROL PROGRAM IMPLEMENTATION. PROCEDURE 12 PMP 3150 PCP.001, RADIOACTIVE WASTE PROCESS CONTROL MANUAL, REQUIRES THAT ALL PLANT GENERATED RADIOACTIVE WASTES BE TRANSFERRED, PACKAGED, AND SHIPPED SUCH THAT RADIOACTIVE WASTE SHIPMENT AND BURIAL REGULATIONS ARE SATISFIED. CONTRARY TO THE ABOVE, ON SEPTEMBER 17, 1985, AT THE BARNWELL WASTE BURIAL FACILITY, A STATE OF SOUTH CAROLINA INSPECTOR FOUND THAT THE LICENSEE WAS IN VIOLATION OF SOUTH CAROLINA DEPARTMENT OF HEALTH AND ENVIRONMENTAL CONTROL REGULATION 61-83, SECTION 1.2, FOR RADIOACTIVE WASTE SHIPMENT NO. 0985-292-A FROM D. C. COOK TO BARNWELL, IN THAT: (A) THE DISPOSAL CONTAINER WAS FOUND TO HAVE ONE END OF THE LIFTING CABLE NOT PROPERLY ATTACHED, CONTRARY TO THE REQUIREMENTS OF CONDITION 64 OF SOUTH CAROLINA RADIOACTIVE MATERIAL LICENSE NO. 097; (B) THE SHIPPING CASK WAS FOUND TO CONTAIN LOOSE CONTAMINATED MATERIAL, CONTRARY TO REQUIREMENTS OF CONDITION 60 OF SOUTH CAROLINA RADIOACTIVE MATERIAL LICENSE NO. 097. 10 CFR 71.5 PROHIBITS TRANSPORT OF ANY LICENSED MATERIAL OUTSIDE THE CONFINES OF A PLANT OR OTHER PLACE OF USE OR DELIVERY OF LICENSED MATERIAL TO A CARRIER FOR TRANSPORT UNLESS THE LICENSEE COMPLIES WITH APPLICABLE REGULATIONS OF THE DEPARTMENT OF TRANSPORTATION IN 49 CFR PARTS 170-189. 49 CFR 173.425(B)(6) REQUIRES THAT EXCLUSIVE USE SHIPMENTS OF LOW SPECIFIC ACTIVITY (LSA) MATERIAL MUST BE BRACED SO AS TO PREVENT SHIFTING OF LADING. CONTRARY TO THE ABOVE, ON APRIL 25, 1986, AT THE BARNWELL WASTE BURIAL FACILITY, A STATE OF SOUTH CAROLINA INSPECTOR FOUND THAT, DUE TO INADEQUATE BLOCKING AND BRACING, ALL THREE PACKAGES ON THE FLATBED TRAILER HAD UNDERGONE A SHIFT OF LADING DURING THE TRANSPORTATION OF LSA EXCLUSIVE USE SHIPMENT NO. 0486-294-A FROM D. C. COOK TO BARNWELL.
(8600 4)

UNIT 1 TECHNICAL SPECIFICATION 6.8.1.A REQUIRES WRITTEN PROCEDURES BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING THE APPLICABLE PROCEDURES RECOMMENDED IN REGULATORY GUIDE 1.33 APPENDIX A, NOVEMBER 1972, WHICH INCLUDES ADMINISTRATIVE PROCEDURES FOR EQUIPMENT CONTROL AND FOR PROCEDURE REVIEW AND APPROVAL. PLANT MANAGER'S INSTRUCTION PMI-2010, "PLANT MANAGER AND DEPARTMENT HEAD INSTRUCTIONS, PROCEDURES, AND ASSOCIATED INDEXES," AN ADMINISTRATIVE PROCEDURE FOR EQUIPMENT CONTROL AND PROCEDURE REVIEW AND APPROVAL REQUIRES IN PARAGRAPH 3.7.1 THAT PROCEDURES BE PREPARED UTILIZING THE FORMAT SPECIFIED IN THE ASSOCIATED ATTACHMENTS 1 AND 2. ATTACHMENT 2, "FORMAT OF PROCEDURES" AT PARAGRAPH 4.I ENTITLED, "RESTORATION" REQUIRES STEPS BE PROVIDED TO ASSURE NECESSARY RESTORATION OF ALL CONTROLS AND EQUIPMENT WHOSE STATUS WAS CHANGED DURING THE PROCEDURE. CONTRARY TO THE ABOVE, LICENSEE PROCEDURE 1 THP 6030 IMP.076, "INCORE THERMOCOUPLE AND REACTOR COOLANT SYSTEM RTD CROSS CALIBRATION PROCEDURE" PROVIDES FOR INSTALLATION OF CALIBRATED PRESSURE GAUGES AT VARIOUS POINTS IN THE MAIN STEAM SYSTEM, BUT DOES NOT CONTAIN STEPS TO REMOVE THE INSTALLED GAUGES ON COMPLETION OF THE TEST. AS A CONSEQUENCE, FOLLOWING COMPLETION OF THE SUBJECT TEST ON OCTOBER 31, 1985 WITH THE PLANT IN MODE 3, FOUR PRESSURE GAUGES, ONE UPSTREAM OF EACH OF THE FOUR MAIN STEAM ISOLATION VALVES, WERE NOT REMOVED. THE PLANT SUBSEQUENTLY OPERATED IN MODE 1 FOR AN EXTENDED PERIOD WITH THE GAUGES STILL INSTALLED. SOME ITEMS WERE NOT SEARCHED OR SECURED AT REQUIRED BY THE SECURITY PLAN.
(8602 4)

OTHER ITEMS

INSPECTION STATUS - (CONTINUED)

COOK 2

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

AFTER STARTUP FROM REFUELING, THE UNIT OPERATED NORMALLY THROUGHOUT THE MONTH.

LAST IE SITE INSPECTION DATE: 08/22/86

INSPECTION REPORT NO: 86031

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-15	04/08/86	06/06/86	INOPERABLE FIRE SEALS, NOT IDENTIFIED DURING PRIOR INSPECTIONS
86-18	05/09/86	06/09/86	MISSED SURVEILLANCE, CAUSED BY MIS-INTERPRETATION OF TECHNICAL SPECIFICATIONS
86-19	06/05/86	07/03/86	MISSED FIRE INSPECTION OF INOPERABLE FIRE DOOR DUE TO PERSONNEL ERROR
86-20	06/23/86	07/22/86	MISSED T/S SURVEILLANCE DUE TO THE FAILURE TO IDENTIFY/PERFORM APPROPRIATE POST-DESIGN CHANGE TESTING

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1. Docket: 50-298 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: K. E. SUTTON (402) 825-3811
4. Licensed Thermal Power (MWt): 2381
5. Nameplate Rating (Gross MWe): 983 X 0.85 = 836
6. Design Electrical Rating (Net MWe): 778
7. Maximum Dependable Capacity (Gross MWe): 787
8. Maximum Dependable Capacity (Net MWe): 764
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	105,936.0
13. Hours Reactor Critical	744.0	5,059.0	80,072.1
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	744.0	5,047.0	78,752.8
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,599,720	9,994,570	154,857,236
18. Gross Elec Ener (MWH)	529,844	3,338,360	49,478,333
19. Net Elec Ener (MWH)	510,041	3,217,205	47,671,565
20. Unit Service Factor	100.0	99.2	74.3
21. Unit Avail Factor	100.0	99.2	74.3
22. Unit Cap Factor (MDC Net)	89.7	82.8	58.9
23. Unit Cap Factor (DER Net)	88.1	81.3	57.8
24. Unit Forced Outage Rate	.0	.8	4.8
25. Forced Outage Hours	.0	40.0	3,314.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

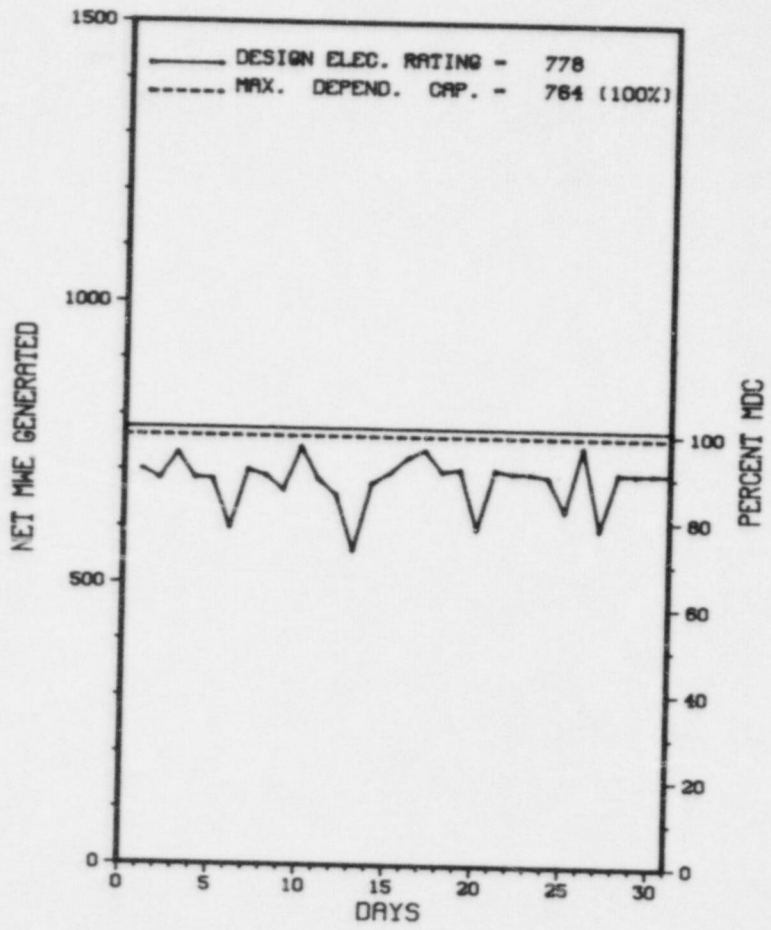
REFUELING & MAINTENANCE: 10/3/86 - 10 WEEKS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* COOPER STATION *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

COOPER STATION



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

COOPER STATION

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence

NONE

SUMMARY

COOPER STATION OPERATED ROUTINELY DURING THE JULY REPORT PERIOD.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* COOPER STATION *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....NEBRASKA

COUNTY.....NEMAHA

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...23 MI S OF
NEBRASKA CITY, NEB

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...FEBRUARY 21, 1974

DATE ELEC ENER 1ST GENER...MAY 10, 1974

DATE COMMERCIAL OPERATE....JULY 1, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MISSOURI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NEBRASKA PUBLIC POWER DISTRICT

CORPORATE ADDRESS.....P.O. BOX 499
COLUMBUS, NEBRASKA 68601

CONTRACTOR
ARCHITECT/ENGINEER.....BURNS & ROE

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BURNS & ROE

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....D. DUBOIS

LICENSING PROJ MANAGER.....W. LONG
DOCKET NUMBER.....50-298

LICENSE & DATE ISSUANCE....DPR-46, JANUARY 18, 1974

PUBLIC DOCUMENT ROOM.....AUBURN PUBLIC LIBRARY
1118 15TH STREET
AUBURN, NEBRASKA 68305

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION CONDUCTED APRIL 1-MAY 31, 1986 (86-14) ROUTINE, UNANNOUNCED INSPECTION OF PREVIOUS IDENTIFIED INSPECITON FINDINGS, LICENSEE EVENT REPORTS, SPENT FUEL SHIPMENTS, OPERATIONAL SAFETY VERIFICATION, AND MONTHLY SURVEILLANCE AND MAINTENANCE ACT-IVITIES. WITHIN THE SIX AREAS INSPECTED, THREE VIOLATIONS AND TWO DEVIATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED APRIL 21-25, 1986 (86-15) NONROUTINE, ANNOUNCED INSPECTION FOR IMPLEMENTATION OF AND COMPLIANCE TO THE SAFE SHUTDOWN REQUIREMENTS OF 10 CFR 50, APPENDIX R. WITHIN THE AREAS INSPECTED, THREE VIOLATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED APRIL 21-25, 1986 (86-16) ROUTINE, UNANNOUNCED INSPECTION OF THE VITAL AREA PHYSICAL BARRIERS AND DETECTION AIDS, PERSONNEL ACCESS CONTROL, ALARM STATION, SECURITY SYSTEM POWER SUPPLY, AND COMMUNICATIONS. WITHIN THE SIX AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED.

INSPECTION CONDUCTED JUNE 23-27, 1986 (86-20) ROUTINE, UNANNOUNCED INSPECTION OF THE CORRECTIVE ASTION SYSTEM. WITHIN THE AREA INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED JUNE 1-30, 1986 (86-21) ROUTINE, UNANNOUNCED INSPECTION OF PREVIOUSLY IDENTIFIED INSPECTION FINDINGS, LICENSEE EVENT REPORTS, SPENT FUEL SHIPMENTS, OPERATIONAL SAFETY VERIFICATION, AND MONTHLY SURVEILLANCE AND MAINTENANCE ACTIVITIES. WITHIN THE AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED.

ENFORCEMENT SUMMARY

10 CFR PART 50.55A REQUIRES THAT THE REACTOR COOLANT PRESSURE BOUNDARY BE CONSTRUCTED IN ACCORDANCE WITH ASME CODE III FOR CLASS I SYSTEMS. ASME CODE III FOR CLASS I SYSTEMS. ASME BOILER AND PRESSURE VESSEL CODE, SECTION III, NB 5320, REQUIRES THAT INDICATIONS OBSERVED IN INTERPRETING RADIOGRAPHIC FILM ARE RECORDED AND DISPOSITIONED. THE PROPER DISPOSITION OF LINEAR INDICATIONS, THAT EXCEED CODE, IS REPAIR OF THE AFFECTED AREA. CONTRARY TO THE ABOVE, AN NRC REVIEW OF SITE RADIOGRAPHS DURING MARCH 11-22, 1985, DISCLOSED THAT CODE INSPECTED AND ACCEPTED CLASS I WELD N2H CONTAINED UNACCEPTABLE LINEAR INDICATIONS THAT EXCEEDED CODE REQUIREMENTS AND WHICH HAD NOT BEEN RECORDED OR DISPOSITIONED.
(8501 4)

FAILURE TO POSITIVELY CONTROL ACCESS TO VITAL AREAS.

(8601 4)

CONTRARY TO CNS TECHNICAL SPECIFICATIONS, SECTION6.1.4 AND CNS PROCEDURE 0.17, THREE INDIVIDUALS EXCEEDED THE TIME FRAME FOR SECURITY REFRESHER TRAINING AND TWO INDIVIDUALS EXCEEDED THE TIME FRAME FOR EMERGENCY PLAN REFRESHER TRAINING.
(8601 5)

CONTRARY TO 10 CFR PART 50, APPENDIX B, CRITERION V, THE LICENSEE DID NOT HAVE A PROCEDURE FOR ENSURING THAT TECHNICAL SPECIFICATION AMENDMENT REQUIREMENTS RESULTED IN REVISIONS TO AFFECTED STATION PROCEDURES.
(8602 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

NORMAL POWER OPERATION

LAST IE SITE INSPECTION DATE: JUNE 30, 1986

INSPECTION REPORT NO: 50-298/86-21

REPORTS FROM LICENSEE

COOPER STATION

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-012	6/10/86	7/02/86	LOSS OF EMERGENCY TRANSFORMER SUPPLY
86-013	6/17/86	7/17/86	BREACH OF PRIMARY CONTAINMENT INTERGRITY

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1. Docket: 50-302 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: D. GRAHAM (904) 795-3802
4. Licensed Thermal Power (MWt): 2544
5. Nameplate Rating (Gross MWe): 989 X 0.9 = 890
6. Design Electrical Rating (Net MWe): 825
7. Maximum Dependable Capacity (Gross MWe): 860
8. Maximum Dependable Capacity (Net MWe): 821
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

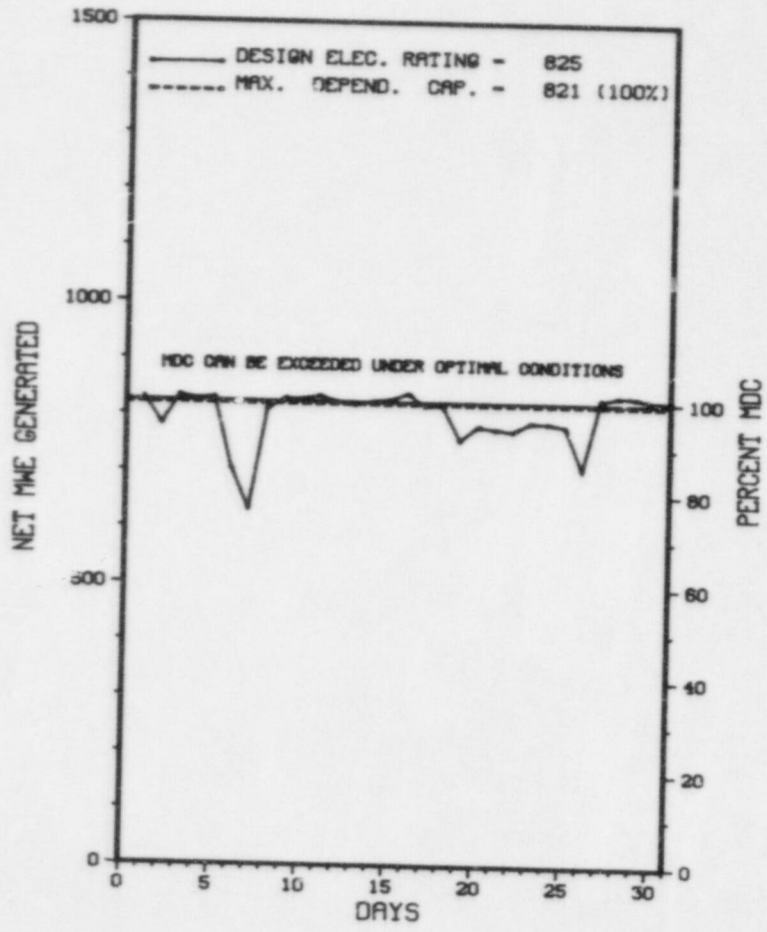
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	82,271.0
13. Hours Reactor Critical	744.0	1,033.9	51,335.7
14. Rx Reserve Shtdwn Hrs	.0	.0	1,275.5
15. Hrs Generator On-Line	744.0	1,013.2	50,104.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,807,807	2,424,534	112,893,399
18. Gross Elec Ener (MWH)	627,223	837,271	38,593,155
19. Net Elec Ener (MWH)	596,860	796,142	36,655,728
20. Unit Service Factor	100.0	19.9	60.9
21. Unit Avail Factor	100.0	19.9	60.9
22. Unit Cap Factor (MDC Net)	97.7	19.1	54.3
23. Unit Cap Factor (DER Net)	97.2	19.0	54.0
24. Unit Forced Outage Rate	.0	80.1	24.5
25. Forced Outage Hours	.0	4,073.8	16,234.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: N/A

****
* CRYSTAL RIVER 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

CRYSTAL RIVER 3



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

CRYSTAL RIVER 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-03	07/06/86	F	0.0	A	5		HC	HTEXCH	REDUCED POWER TO LOCATE AND REPAIR A CONDENSER SALT WATER LEAK IN THE "C" WATERBOX.
86-04	07/26/86	F	0.0	A	5		HH	MOTORX	LOST POWER TO THE 1A CONDENSATE PUMP MOTOR CONTROLLER. PLANT REDUCED POWER TO REPAIR.

SUMMARY

CRYSTAL RIVER 3 OPERATED WITH 2 REDUCTIONS FOR EQUIPMENT FAILURE IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* CRYSTAL RIVER 3 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....FLORIDA

COUNTY.....CITRUS

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...7 MI NW OF
CRYSTAL RIVER, FLA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JANUARY 14, 1977

DATE ELEC ENER 1ST GENER...JANUARY 30, 1977

DATE COMMERCIAL OPERATE....MARCH 13, 1977

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....GULF OF MEXICO

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....FLORIDA POWER CORPORATION

CORPORATE ADDRESS.....3201 34TH STREET, SOUTH
ST PETERSBURG, FLORIDA 33733

CONTRACTOR
ARCHITECT/ENGINEER.....GILBERT ASSOCIATES

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....J. A. JONES CONSTRUCTION

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....T. STETKA

LICENSING PROJ MANAGER.....H. SILVER
DOCKET NUMBER.....50-302

LICENSE & DATE ISSUANCE....DPR-72, JANUARY 28, 1977

PUBLIC DOCUMENT ROOM.....CRYSTAL RIVER PUBLIC LIBRARY
668 N.W. FIRST
CRYSTAL RIVER, FLORIDA 32639

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION APRIL 12 - JUNE 6 (86-14): THIS ROUTINE INSPECTION WAS CONDUCTED BY TWO RESIDENT INSPECTORS IN THE AREAS OF PLANT OPERATIONS, SECURITY, RADIOLOGICAL CONTROLS, LICENSEE EVENT REPORTS AND NONCONFORMING OPERATIONS REPORTS, FACILITY MODIFICATIONS, BIOFOULING OF COOLING WATER HEAT EXCHANGERS, AND LICENSEE ACTION ON PREVIOUS INSPECTION ITEMS. NUMEROUS FACILITY TOURS WERE CONDUCTED AND OPERATIONS OBSERVED. SOME OF THESE TOURS AND OBSERVATIONS WERE CONDUCTED ON BACKSHIFTS. TWO VIOLATIONS WERE IDENTIFIED: FAILURE TO TAKE ADEQUATE CORRECTIVE ACTION AND FAILURE TO HAVE AN ADEQUATE MAINTENANCE PROCEDURE.

INSPECTION MAY 19-23 (86-17): THIS ROUTINE, ANNOUNCED INSPECTION WAS CONDUCTED ON SITE IN THE AREAS OF LICENSEE ACTIONS ON PREVIOUS ENFORCEMENT MATTERS, TO ASSESS QUALITY ASSURANCE EFFECTIVENESS, AND LICENSEE ACTION ON PREVIOUSLY IDENTIFIED INSPECTION FINDINGS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 17-20 (86-18): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED EVALUATION OF THE ANNUAL RADIOLOGICAL EMERGENCY PREPAREDNESS EXERCISE. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 2-6 (86-19): THIS ROUTINE, UNANNOUNCED PHYSICAL SECURITY INSPECTION WAS ACCOMPLISHED INSPECTING: ACCESS CONTROLS - PERSONNEL; TRAINING AND QUALIFICATION - GENERAL REQUIREMENTS; RECORDS AND REPORTS; SECURITY SYSTEM POWER SUPPLY; AND SAFEGUARDS INFORMATION. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS IN THE FIVE AREAS INSPECTED EXCEPT FOR A VIOLATION IDENTIFIED IN THE TRAINING AND QUALIFICATION AREA.

****
* CRYSTAL RIVER 3 *
****

ENFORCEMENT SUMMARY

CONTRARY TO TS 6.8.1.A AND MAINTENANCE PROCEDURE MP-405, INSTALLING, REPAIRING AND TERMINATING CONTROLS, POWER AND INSTRUMENTATION CABLES, ON JUNE 3, 1986, PROCEDURE MP-405 WAS FOUND TO BE INADEQUATE, IN THAT NUCLEAR INSTRUMENTATION CABLE THAT WAS REPLACED AND TESTED WITH THIS PROCEDURE HAS A NORMAL OPERATING VOLTAGE OF APPROXIMATELY 2000 VOLTS DC, THEREFORE MAKING A 1000 VOLT DC TEST INADEQUATE TO VERIFY PROPER INSULATION RESISTANCE. CONTRARY TO 10 CFR PART 50, APPENDIX B, CRITERION V AND COMPLIANCE PROCEDURE CP-125, CORRECTIVE ACTION PROCEDURE, THAT WAS WRITTEN TO ACCOMPLISH CRITERION XVI, AS OF JUNE 4, 1986, A DEFICIENCY IN A SAFETY-RELATED MOTOR OPERATED VALVE (DHV-39), WHICH WAS IDENTIFIED BY THE LICENSEE ON MAY 2, 1986, DID NOT HAVE A CAA COMPLETED AS REQUIRED BY PROCEDURE CP-125. AS A RESULT THE DEFICIENCY IN DHV-39 WAS NOT CORRECTED. FAILURE TO SATISFY REQUIREMENTS OF LICENSEE'S TRAINING AND QUALIFICATION.
(8601 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATIONS.

LAST IE SITE INSPECTION DATE: JUNE 2-6, 1986 +

INSPECTION REPORT NO: 50-302/86-19 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-003	02/02/86	05/15/86	LOSS OF DECAY HEAT REMOVAL CAPABILITY; DUE TO A MOTOR OVERLOAD CAUSED BY A PUMP SHAFT FAILURE.
86-008	06/20/86	07/17/86	LOW LEVEL ALARM RECEIVED ON THE 'B' OTSG AND WAS FOLLOWED BY AN AUTOMATIC ACTUATION OF EMERGENCY FEEDWATER SYSTEM; CAUSE - UNKNOWN.

1. Docket: 50-346 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: MORTEZA KHARZRAI (419) 249-5000 X290
4. Licensed Thermal Power (MWt): 2772
5. Nameplate Rating (Gross MWe): 1069 X 0.9 = 962
6. Design Electrical Rating (Net MWe): 906
7. Maximum Dependable Capacity (Gross MWe): 904
8. Maximum Dependable Capacity (Net MWe): 860
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	70,152.0
13. Hours Reactor Critical	.0	.0	35,878.0
14. Rx Reserve Shtdwn Hrs	.0	.0	4,058.8
15. Hrs Generator On-Line	.0	.0	34,371.8
16. Unit Reserve Shtdwn Hrs	.0	.0	1,732.7
17. Gross Therm Ener (MWH)	0	0	81,297,598
18. Gross Elec Ener (MWH)	0	0	26,933,622
19. Net Elec Ener (MWH)	0	0	25,233,177
20. Unit Service Factor	.0	.0	49.0
21. Unit Avail Factor	.0	.0	51.5
22. Unit Cap Factor (MDC Net)	.0	.0	41.8
23. Unit Cap Factor (DER Net)	.0	.0	39.7
24. Unit Forced Outage Rate	100.0	100.0	32.8
25. Forced Outage Hours	744.0	5,087.0	17,378.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

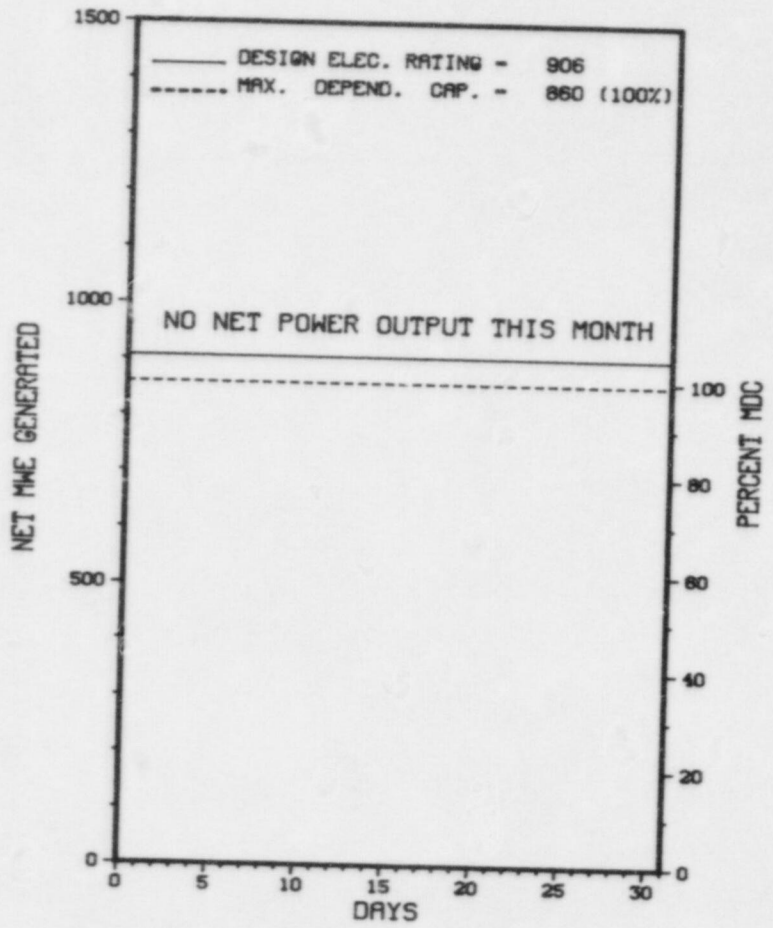
NONE

27. If Currently Shutdown Estimated Startup Date: 10/18/86

****
* DAVIS-BESSE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

DAVIS-BESSE 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* DAVIS-BESSE 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
7	06/09/85	F	744.0	A	4	85-013	JK	SC	THE UNIT REMAINED SHUTDOWN FOLLOWING THE REACTOR TRIP ON JUNE 9, 1985, DUE TO MAIN FEED CONTROL PUMP PROBLEMS.

****
* SUMMARY *
****

DAVIS BESSE 1 REMAINS SHUTDOWN FOR EQUIPMENT REPAIRS.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* DAVIS-BESSE 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....OHIO

COUNTY.....OTTAWA

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...21 MI E OF
TOLEDO, OH

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 12, 1977

DATE ELEC ENER 1ST GENER...AUGUST 28, 1977

DATE COMMERCIAL OPERATE....JULY 31, 1978

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....LAKE ERIE

ELECTRIC RELIABILITY
COUNCIL.....EAST CENTRAL AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....TOLEDO EDISON

CORPORATE ADDRESS.....300 MADISON AVENUE
TOLEDO, OHIO 43652

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....W. ROGERS

LICENSING PROJ MANAGER.....A. DEAGAZIO
DOCKET NUMBER.....50-346

LICENSE & DATE ISSUANCE....NPF-3, APRIL 22, 1977

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GOVERNMENT DOCUMENTS COLLECTION
2801 WEST BANCROFT AVENUE
TOLEDO, OHIO 43606

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON MAY 1 THROUGH JUNE 15 (86014): ROUTINE, UNANNOUNCED INSPECTION BY RESIDENT INSPECTORS OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, OPERATIONAL SAFETY, LICENSEE EVENT REPORTS, MAINTENANCE, SURVEILLANCE, EMERGENCY PLANNING, PERFORMANCE ENHANCEMENT PROGRAM, STRIKE ACTIVITIES AND MANAGEMENT MEETINGS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON JUNE 27 (86018): ROUTINE, UNANNOUNCED INSPECTION OF THE RESOLUTION OF AN IE BULLETIN. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

OTHER ITEMS

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS IN COLD SHUTDOWN FOLLOWING THE JUNE 9, 1985 TRIP.

LAST IE SITE INSPECTION DATE: 08/07/86

INSPECTION REPORT NO: 86024

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
84-15	10/25/86	07/02/86	VELAN CHECK VALVE ANTIROTATION BINDING
86-05	12/30/85	07/01/86	IMPROPER BOOT-SEAL INSTALLATION
86-06	01/02/86	08/01/86	ENVIRONMENTAL QUALIFICATION PROGRAM NOT ADEQUATELY ESTABLISHED
86-10	01/29/86	07/11/86	FIRE DAMPER INSTALLATION DEFICIENCIES VOIDING UL RATING
86-25	07/01/86	07/31/86	DISCREPANCIES BETWEEN TECHNICAL SPECIFICATIONS AND INSTALLED FIRE DETECTION
86-26	07/03/86	08/04/86	DESIGN AND TESTING DEFICIENCIES FOR THE ATMOSPHERIC VENT VALVES
86-27	06/25/86	07/31/86	FIRE DOORS INOPERABLE BY NFPA 80 STANDARDS

1. Docket: 50-275 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: A. T. KELLER (805) 595-7351

4. Licensed Thermal Power (MWt): 3338

5. Nameplate Rating (Gross MWe): 1137

6. Design Electrical Rating (Net MWe): 1086

7. Maximum Dependable Capacity (Gross MWe): 1125

8. Maximum Dependable Capacity (Net MWe): 1073

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	10,821.3
13. Hours Reactor Critical	744.0	5,055.5	10,351.1
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	744.0	5,030.0	10,237.3
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,842,336	15,561,760	32,127,213
18. Gross Elec Ener (MWH)	597,800	5,185,300	10,699,632
19. Net Elec Ener (MWH)	560,977	4,928,258	10,162,492
20. Unit Service Factor	100.0	98.9	94.6
21. Unit Avail Factor	100.0	98.9	94.6
22. Unit Cap Factor (MDC Net)	70.3	90.3	87.5
23. Unit Cap Factor (DER Net)	69.4	89.2	86.5
24. Unit Forced Outage Rate	.0	1.1	2.9
25. Forced Outage Hours	.0	57.0	304.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

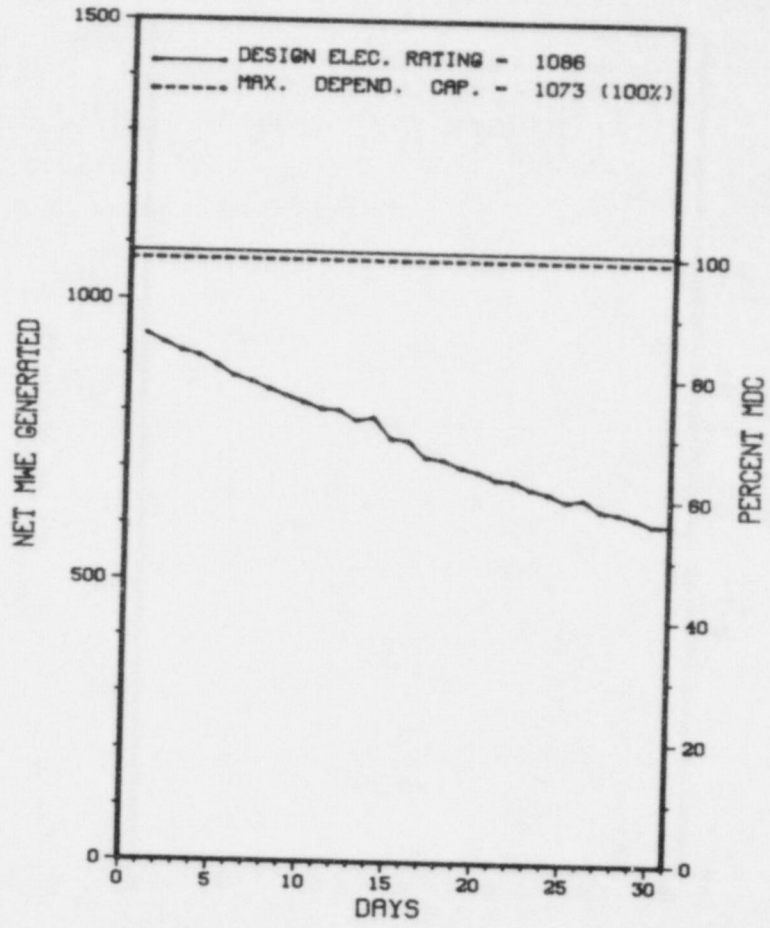
REFUELING OUTAGE: SEPTEMBER 1, 1986, 63 DAYS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* DIABLO CANYON 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

DIABLO CANYON 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

DIABLO CANYON 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

DIABLO CANYON 1 OPERATED ROUTINELY DURING JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* DIABLO CANYON 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....CALIFORNIA

COUNTY.....SAN LUIS OBISPO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...12 MI WSW OF
SAN LUIS OBISPO

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...APRIL 29, 1984

DATE ELEC ENER 1ST GENER...NOVEMBER 11, 1984

DATE COMMERCIAL OPERATE....MAY 7, 1985

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....PACIFIC OCEAN

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PACIFIC GAS & ELECTRIC

CORPORATE ADDRESS.....77 BEALE STREET
SAN FRANCISCO, CALIFORNIA 94106

CONTRACTOR
ARCHITECT/ENGINEER.....PACIFIC GAS & ELECTRIC

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....PACIFIC GAS & ELECTRIC

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....M. MENDONCA

LICENSING PROJ MANAGER.....H. SCHIERLING
DOCKET NUMBER.....50-275

LICENSE & DATE ISSUANCE....DPR-80, NOVEMBER 2, 1984

PUBLIC DOCUMENT ROOM.....ROBERT F. KENNEDY LIBRARY
CALIFORNIA POLYTECHNIC STATE UNIVERSITY
SAN LUIS OBISPO, CA. 93407

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON JUNE 9-12, 1986 (REPORT NO. 50-275/86-10) AREAS INSPECTED: ANNOUNCED INSPECTION OF THE EMERGENCY PREPAREDNESS EXERCISE AND ASSOCIATED CRITIQUE. DURING THIS INSPECTION, ONE INSPECTION PROCEDURE WAS UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON APRIL 28 - MAY 2, 1986 (REPORT NO. 50-275/86-15) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION BY HEADQUARTERS OF THE OPERATION, MAINTENANCE AND SURVEILLANCE TESTING OF THE AUXILIARY FEEDWATER (AFW) SYSTEM. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: DURING THIS INSPECTION, ONE UNRESOLVED ITEM WAS IDENTIFIED CONCERNING THE DETERMINATION OF THE AFW SYSTEM OPERABILITY DURING PUMP MAINTENANCE.

+ INSPECTION ON MAY 25 - JULY 5, 1986 (REPORT NO. 50-275/86-18) AREAS INSPECTED: THE INSPECTION INCLUDED ROUTINE INSPECTIONS OF PLANT OPERATIONS, MAINTENANCE AND SURVEILLANCE ACTIVITIES, FOLLOWUP OF ONSITE EVENTS, OPEN ITEMS, AND LERS, AS WELL AS SELECTED INDEPENDENT INSPECTION ACTIVITIES. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JULY 7 - AUGUST 1, 1986 (REPORT NO. 50-275/86-19) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

****
* DIABLO CANYON 1 *
****

INSPECTION SUMMARY

+ INSPECTION ON JULY 6 - AUGUST 9, 1986 (REPORT NO. 50-275/86-20) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JULY 21-25, 1986 (REPORT NO. 50-275/86-21) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON AUGUST 1, 1985 - JULY 31, 1986 (REPORT NO. 50-275/86-22) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE PLANT IS IN COMMERCIAL OPERATION.

LAST IE SITE INSPECTION DATE: 07/06-08/09/86

INSPECTION REPORT NO: 50-275/86-20

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-323 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: A. T. KELLER (805) 595-7351
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1164
6. Design Electrical Rating (Net MWe): 1119
7. Maximum Dependable Capacity (Gross MWe): 1124
8. Maximum Dependable Capacity (Net MWe): 1073
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	3,380.0	3,380.0
13. Hours Reactor Critical	693.5	3,220.5	3,220.5
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	637.5	3,117.5	3,117.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,782,564	9,519,687	9,519,687
18. Gross Elec Ener (MWH)	566,500	3,100,199	3,100,199
19. Net Elec Ener (MWH)	531,133	2,932,317	2,932,317
20. Unit Service Factor	85.7	92.2	92.2
21. Unit Avail Factor	85.7	92.2	92.2
22. Unit Cap Factor (MDC Net)	66.5	80.7	80.9
23. Unit Cap Factor (DER Net)	63.8	77.5	77.5
24. Unit Forced Outage Rate	10.8	7.0	7.0
25. Forced Outage Hours	77.2	233.2	233.2

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

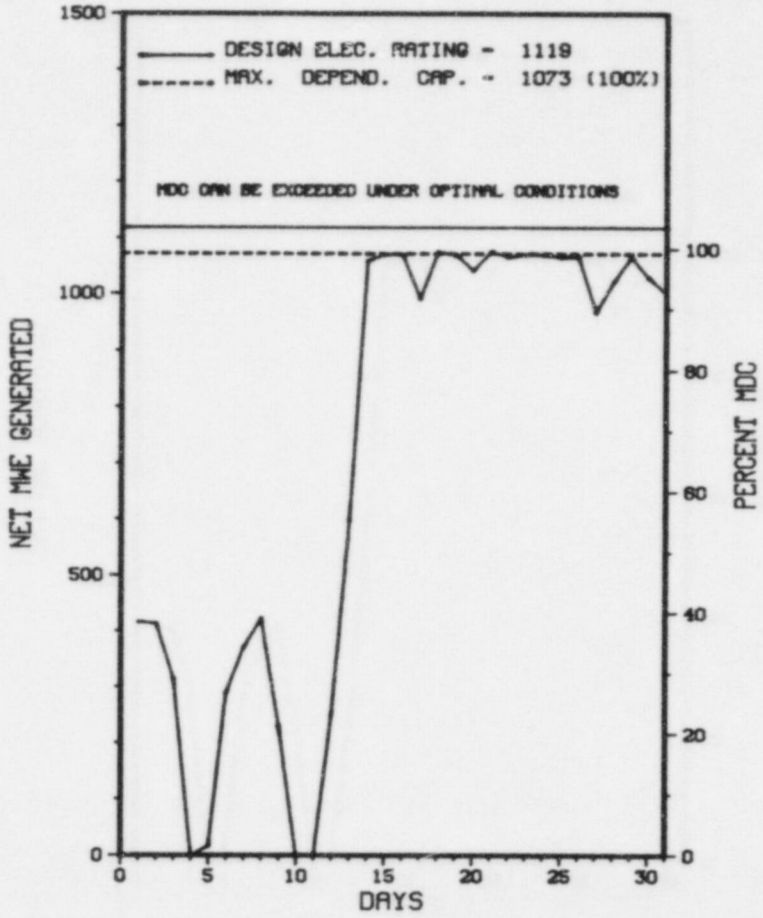
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* DIABLO CANYON 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

DIABLO CANYON 2



JULY 1986

Report Period JUL 1986 **UNIT SHUTDOWNS / REDUCTIONS**

DIABLO CANYON 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	07/03/86	S	29.3	B	1				THE GENERATOR WAS SEPARATED FROM THE PG&E SYSTEM IN PREPARATION FOR OVERSPEED TRIP TESTING ON THE MAIN TURBINE.
2	07/05/86	F	18.4	H	3	2-86-020	SJ	FCV	WHILE MANUALLY ADJUSTING THE FLOW RATE TO SG 2-2, THE AUTOMATIC FLOW CONTROL VALVE FOR SG 2-1 ALLOWED EXCESSIVE FEED CAUSING A HIGH HIGH SG WATER LEVEL TURBINE TRIP AND SUBSEQUENT REACTOR TRIP. NO CORRECTIVE ACTION WAS DEEMED NECESSARY SINCE OPERATORS ARE ALREADY TRAINED FOR THIS SENSITIVE TRANSITION FROM MANUAL TO AUTOMATIC SG FEED CONTROL.
3	07/06/86	F	3.4	A	1		SE	PSF	THE GENERATOR WAS SEPARATED FROM THE PG&E SYSTEM TO REPAIR A STEAM LEAK ON AN EXTRACTION STEAM LINE FROM THE MAIN TURBINE.
4	07/09/86	F	55.4	A	3	86-021	SJ	P	MAIN FEEDWATER PUMP (MFP) 2-1 TRIPPED ON OVERSPEED RESULTING IN A REACTOR TRIP. NO APPARENT CAUSE OF THE PUMP TRIP COULD BE FOUND.

SUMMARY

DIABLO CANYON 2 OPERATED WITH 4 OUTAGES DURING JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* DIABLO CANYON 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....CALIFORNIA

COUNTY.....SAN LUIS OBISPO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...12 MI WSW OF
SAN LUIS OBISPO

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 19, 1985

DATE ELEC ENER 1ST GENER...OCTOBER 20, 1985

DATE COMMERCIAL OPERATE....MARCH 13, 1986

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....PACIFIC OCEAN

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PACIFIC GAS & ELECTRIC

CORPORATE ADDRESS.....77 BEALE STREET
SAN FRANCISCO, CALIFORNIA 94106

CONTRACTOR
ARCHITECT/ENGINEER.....PACIFIC GAS & ELECTRIC

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....PACIFIC GAS & ELECTRIC

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....M. MENDOUCA

LICENSING PROJ MANAGER.....H. SCHIERLING
DOCKET NUMBER.....50-323

LICENSE & DATE ISSUANCE...DPR-82, AUGUST 26, 1985

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CALIFORNIA POLYTECHNIC STATE UNIVERSITY
SAN LUIS OBISPO, CA. 93407

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON JUNE 9-12, 1986 (REPORT NO. 50-323/86-11) AREAS INSPECTED: ANNOUNCED INSPECTION OF THE EMERGENCY PREPAREDNESS EXERCISE AND ASSOCIATED CRITIQUE. DURING THIS INSPECTION, ONE INSPECTION PROCEDURE WAS UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON MAY 25 - JULY 5, 1986 (REPORT NO. 50-323/86-18) AREAS INSPECTED: THE INSPECTION INCLUDED ROUTINE INSPECTIONS OF PLANT OPERATIONS, MAINTENANCE AND SURVEILLANCE ACTIVITIES, FOLLOWUP OF ONSITE EVENTS, OPEN ITEMS, AND LERS, AS WELL AS SELECTED INDEPENDENT INSPECTION ACTIVITIES. ADDITIONALLY, A/U TESTING PHASE OF THE LIGHT WATER REACTOR INSPECTION PROGRAM WAS CONCLUDED DURING THIS PERIOD. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: ONE VIOLATION AND NO DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JULY 7 - AUGUST 1, 1986 (REPORT NO. 50-323/86-19) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JULY 6 - AUGUST 9, 1986 (REPORT NO. 50-323/86-20) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JULY 21-25, 1986 (REPORT NO. 50-323/86-21) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON AUGUST 1, 1985 - JULY 31, 1986 (REPORT NO. 50-323/86-22) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

****
* DIABLO CANYON 2 *
****

INSPECTION SUMMARY

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE PLANT IS IN COMMERCIAL OPERATION.

LAST IE SITE INSPECTION DATE: 07/06-08/09/86

INSPECTION REPORT NO: 50-323/86-20

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-14-L0	05-06-86	06-05-86	DIESEL GENERATOR START AND LOADING DUE TO INCORRECTLY TERMINATED JUMPER
86-15-L0	02-14-86	05-30-86	MISSED SURVEILLANCE OF RADIATION MONITOR

1. Docket: 50-237 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: D. C. MAXWELL (815) 942-2920
4. Licensed Thermal Power (MWt): 2527
5. Nameplate Rating (Gross MWe): 920 X 0.9 = 828
6. Design Electrical Rating (Net MWe): 794
7. Maximum Dependable Capacity (Gross MWe): 812
8. Maximum Dependable Capacity (Net MWe): 772
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	142,151.0
13. Hours Reactor Critical	701.5	4,280.1	107,978.6
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	683.0	3,981.4	102,966.4
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,583,834	9,295,771	210,972,370
18. Gross Elec Ener (MWH)	504,650	3,013,678	67,496,854
19. Net Elec Ener (MWH)	480,972	2,869,382	63,814,674
20. Unit Service Factor	91.8	78.3	72.4
21. Unit Avail Factor	91.8	78.3	72.4
22. Unit Cap Factor (MDC Net)	83.7	73.1	58.2
23. Unit Cap Factor (DER Net)	81.4	71.0	56.5
24. Unit Forced Outage Rate	8.2	21.7	12.0
25. Forced Outage Hours	61.0	1,105.6	6,552.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

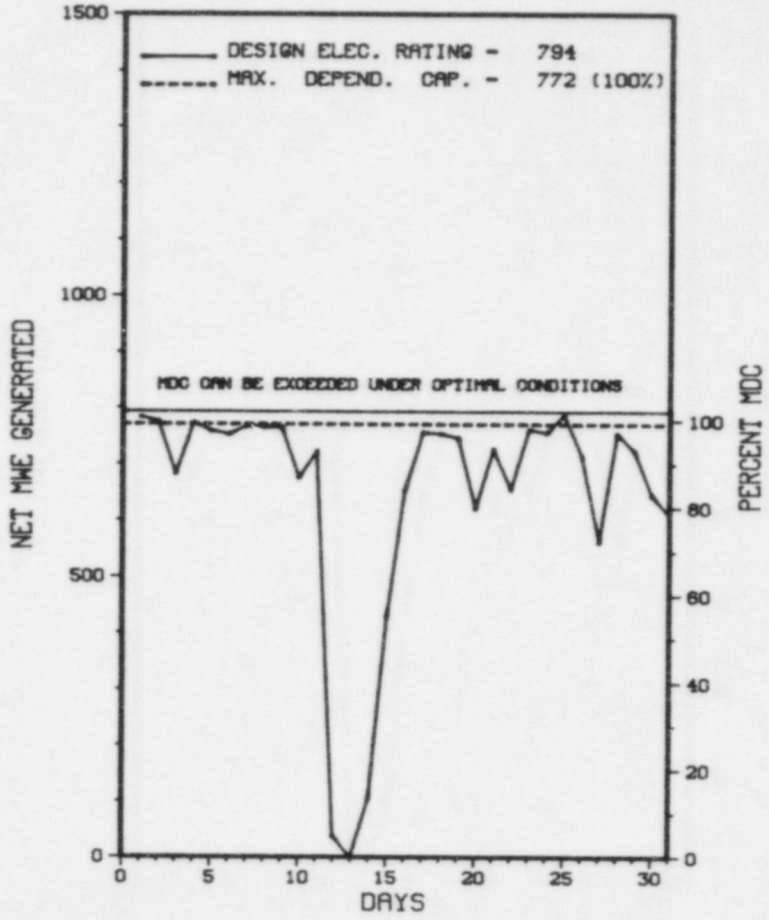
REFUELING OUTAGE: DECEMBER, 1986-FEBRUARY, 1987.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* DRESDEN 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

DRESDEN 2



JULY 1986

Report Period JUL 1986 UNIT SHUTDOWNS / REDUCTIONS

****
* DRESDEN 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
6	07/12/86	F	61.0	A	3	86-017-0			RX SCRAM, 1C MSIV FULL CLOSED DURING SURVEILLANCE TESTING, CAUSING A PRESSURE AND HIGH FLUX SPIKE.

****
* SUMMARY *
****

DRESDEN 2 OPERATED WITH 1 OUTAGE DUE TO EQUIPMENT FAILURE IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* DRESDEN 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....GRUNDY

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...9 MI E OF
MORRIS, ILL

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JANUARY 7, 1970

DATE ELEC ENER 1ST GENER...APRIL 13, 1970

DATE COMMERCIAL OPERATE....JUNE 9, 1970

CONDENSER COOLING METHOD...COOLING LAKE

CONDENSER COOLING WATER....KANKAKEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....L. MCGREGOR

LICENSING PROJ MANAGER.....J. STANG
DOCKET NUMBER.....50-237

LICENSE & DATE ISSUANCE....DPR-19, DECEMBER 22, 1969

PUBLIC DOCUMENT ROOM.....MORRIS PUBLIC LIBRARY
604 LIBERTY STREET
MORRIS, ILLINOIS 60450

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION DURING THE PERIOD OF APRIL 26 THROUGH JUNE 13 (86011; 86014): ROUTINE UNANNOUNCED RESIDENT INSPECTION OF OPERATIONAL SAFETY, FOLLOWUP OF EVENTS, REGIONAL REQUESTS, MAINTENANCE, SURVEILLANCE, LICENSEE EVENT REPORTS AND TMI MODIFICATIONS. OF THE SEVEN AREAS INSPECTED, NO VIOLATIONS OF NRC REQUIREMENTS WERE IDENTIFIED.

INSPECTION ON MAY 19 THROUGH 23 (86015; 86017): SPECIAL TEAM INSPECTION TO FOLLOWUP ON MODIFICATION PACKAGE PROBLEMS IDENTIFIED BY THE SAFETY SYSTEMS OUTAGE MODIFICATION INSPECTION TEAM FROM THE OFFICE OF INSPECTION AND ENFORCEMENT ON UNIT 3 TO DETERMINE IF SIMILAR PROBLEMS WERE PRESENT ON UNIT 2 AND, IF PROBLEMS WERE PRESENT, TO DETERMINE IF THEY JEOPARDIZED THE RETURN TO OPERATION OF UNIT 2. THE INSPECTION TEAM DETERMINED THAT IN THE THIRTEEN UNIT 2 MODIFICATION PACKAGES INSPECTED ABOUT HALF CONTAINED SIMILAR PROBLEMS AS HAD BEEN IDENTIFIED FOR UNIT 3. THE PROBLEMS IDENTIFIED WERE EVALUATED BY THE TEAM TO NOT JEOPARDIZE THE RETURN TO OPERATION OF UNIT 2. ONE VIOLATION WAS IDENTIFIED IN THE AREA OF AS-BUILT DRAWINGS VERSUS AS-FOUND ELECTRICAL CONFIGURATION (FAILURE TO ENSURE DRAWINGS ARE KEPT UPDATED AND INSTALLATION IS PERFORMED IN ACCORDANCE WITH DRAWINGS.

INSPECTION ON JUNE 30 THROUGH JULY 3 (86019; 86023): ROUTINE UNANNOUNCED INSPECTION OF THE RESOLUTION OF IE BULLETIN 79-02, "PIPE SUPPORT BASEPLATE DESIGN USING CONCRETE EXPANSION ANCHORS." NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

ON FEBRUARY 7 THROUGH 20, 1986, UNIT 2 EXCEEDED THE LICENSED THERMAL POWER LIMIT DUE TO INSTRUMENT DRIFT OF THE "C" REACTOR FEEDWATER FLOW TRANSMITTER. NO SAFETY LIMITS WERE EXCEEDED.

THE LICENSEE'S MODIFICATION PROCESS AND ASSOCIATED DRAWING CONTROL PROCESS WERE NOTED TO HAVE SEVERAL PROBLEMS SUCH THAT "AS-BUILT" DRAWINGS DID NOT EFFECTIVELY EXIST.

MANAGERIAL ITEMS:

E. O'CONNOR WILL REPLACE S. MCDONALD AS THE RAD CHEM SUPERVISOR THIS SUMMER.

PLANT STATUS:

UNIT IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: 08/18/86

INSPECTION REPORT NO: 86022

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-14	06/16/86	07/15/86	DFPP 4185-2 SMOKE DETECTOR SEMI-ANNUAL MAINTENANCE TEST EXCEEDED SCHEDULED COMPLETION DATE DUE TO PERSONNEL ERROR

1. Docket: 50-249 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: D. C. MAXWELL (815) 942-2920

4. Licensed Thermal Power (MWt): 2527

5. Nameplate Rating (Gross MWe): 920 X 0.9 = 828

6. Design Electrical Rating (Net MWe): 794

7. Maximum Dependable Capacity (Gross MWe): 812

8. Maximum Dependable Capacity (Net MWe): 773

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	131,736.0
13. Hours Reactor Critical	.0	.0	93,442.9
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	.0	89,794.8
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	181,765,542
18. Gross Elec Ener (MWH)	0	0	58,821,103
19. Net Elec Ener (MWH)	-5,590	-36,276	55,690,017
20. Unit Service Factor	.0	.0	68.2
21. Unit Avail Factor	.0	.0	68.2
22. Unit Cap Factor (MDC Net)	.0	.0	54.7
23. Unit Cap Factor (DER Net)	.0	.0	53.2
24. Unit Forced Outage Rate	.0	.0	12.2
25. Forced Outage Hours	.0	.0	7,365.0

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

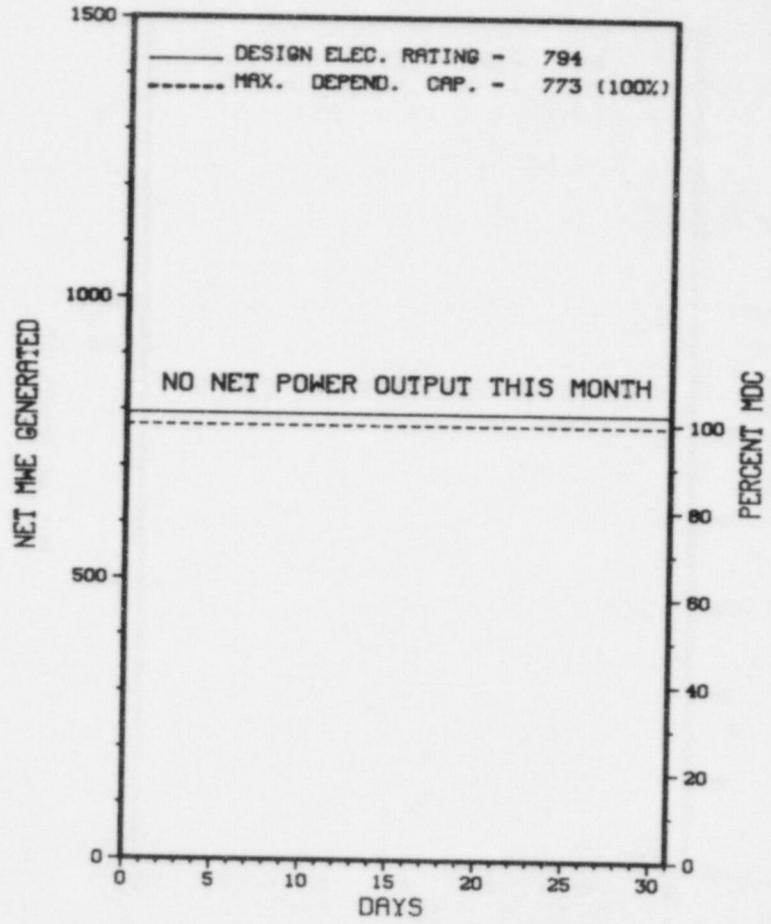
NONE

27. If Currently Shutdown Estimated Startup Date: 08/08/86

****
* DRESDEN 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

DRESDEN 3



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

DRESDEN 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
6	10/23/85	S	744.0	C	4				OFF-LINE MANUALLY TO 9TH REFUELING/RECIRCULATION PIPE REPLACEMENT OUTAGE.

SUMMARY

DRESDEN 3 REMAINS SHUTDOWN FOR REFUELING.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* DRESDEN 3 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....GRUNDY

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...9 MI E OF
MORRIS, ILL

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JANUARY 31, 1971

DATE ELEC ENER 1ST GENER...JULY 22, 1971

DATE COMMERCIAL OPERATE....NOVEMBER 16, 1971

CONDENSER COOLING METHOD...COOLING LAKE

CONDENSER COOLING WATER....KANKAKEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....L. MCGREGOR

LICENSING PROJ MANAGER.....J. STANG
DOCKET NUMBER.....50-249

LICENSE & DATE ISSUANCE....DPR-25, MARCH 2, 1971

PUBLIC DOCUMENT ROOM.....MORRIS PUBLIC LIBRARY
604 LIBERTY STREET
MORRIS, ILLINOIS 60450

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION DURING THE PERIOD OF APRIL 26 THROUGH JUNE 13 (86011; 86014): ROUTINE UNANNOUNCED RESIDENT INSPECTION OF OPERATIONAL SAFETY, FOLLOWUP OF EVENTS, REGIONAL REQUESTS, MAINTENANCE, SURVEILLANCE, LICENSEE EVENT REPORTS AND TMI MODIFICATIONS. OF THE SEVEN AREAS INSPECTED, NO VIOLATIONS OF NRC REQUIREMENTS WERE IDENTIFIED.

INSPECTION ON MAY 19 THROUGH 23 (86015; 86017): SPECIAL TEAM INSPECTION TO FOLLOWUP ON MODIFICATION PACKAGE PROBLEMS IDENTIFIED BY THE SAFETY SYSTEMS OUTAGE MODIFICATION INSPECTION TEAM FROM THE OFFICE OF INSPECTION AND ENFORCEMENT ON UNIT 3 TO DETERMINE IF SIMILAR PROBLEMS WERE PRESENT ON UNIT 2 AND, IF PROBLEMS WERE PRESENT, TO DETERMINE IF THEY JEOPARDIZED THE RETURN TO OPERATION OF UNIT 2. THE INSPECTION TEAM DETERMINED THAT IN THE THIRTEEN UNIT 2 MODIFICATION PACKAGES INSPECTED ABOUT HALF CONTAINED SIMILAR PROBLEMS AS HAD BEEN FOR UNIT 3. THE PROBLEMS IDENTIFIED WERE EVALUATED BY THE TEAM TO NOT JEOPARDIZE THE RETURN TO OPERATION OF UNIT 2. ONE VIOLATION WAS IDENTIFIED IN THE AREA OF AS-BUILT RSUS AS-FOUND ELECTRICAL CONFIGURATION (FAILURE TO ENSURE DRAWINGS ARE KEPT INSTALLATION IS PERFORMED IN ACCORDANCE WITH DRAWINGS.

INSPECTION ON JUNE 2-4 (86018): UNANNOUNCED, SPECIAL INSPECTION OF ALLEGATIONS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON JUNE 30 THROUGH JULY 3 (86019; 86023): ROUTINE UNANNOUNCED INSPECTION OF THE RESOLUTION OF IE BULLETIN 79-02, "PIPE SUPPORT BASEPLATE DESIGN USING CONCRETE EXPANSION ANCHORS." NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

DRESDEN 3 RECIRCULATING SYSTEM PIPING REPLACEMENT PROJECT IN PROGRESS

FACILITY ITEMS (PLANS AND PROCEDURES):

THE LICENSEE'S MODIFICATION PROCESS AND ASSOCIATED DRAWING CONTROL PROCESS WERE NOTED TO HAVE SEVERAL PROBLEMS SUCH THAT "AS-BUILT" DRAWINGS DID NOT EFFECTIVELY EXIST.

MANAGERIAL ITEMS:

E. O'CONNOR WILL REPLACE S. MCDONALD AS THE RAD CHEM SUPERVISOR THIS COMMING SUMMER

PLANT STATUS:

PLANT IS NOW TOTALLY REFUELED AND IS PERFORMING THE REQUIRED TESTING PRIOR TO RESTART. RESTART IS SCHEDULED FOR 08/20/86.

LAST IE SITE INSPECTION DATE: 08/18/86

INSPECTION REPORT NO: 86027

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-07	06/12/86	07/11/86	AUTOMATIC INITIATION OF STANDBY GAS TREATMENT AND ISOLATION OF REACTOR BUILDING VENTILATION SYSTEM DUE TO PERSONNEL ERROR
86-08	06/18/86	07/17/86	DRYWELL RADIATION MONITOR FAILURE CAUSED BY PERSONNEL ERROR RESULTING IN A GROUP II PRIMARY CONTAINMENT ISOLATION
86-09	07/08/86	08/06/86	AUTOMATIC START OF THE UNIT 2/3 DIESEL GENERATOR DUE TO PERSONNEL ERROR

1. Docket: 50-331 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: BRADFORD THOMAS (319) 851-7339
4. Licensed Thermal Power (MWt): 1658
5. Nameplate Rating (Gross MWe): 663 X 0.9 = 597
6. Design Electrical Rating (Net MWe): 538
7. Maximum Dependable Capacity (Gross MWe): 545
8. Maximum Dependable Capacity (Net MWe): 515
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	100,775.0
13. Hours Reactor Critical	744.0	4,413.5	71,708.8
14. Rx Reserve Shtdwn Hrs	.0	42.5	172.8
15. Hrs Generator On-Line	744.0	4,316.2	69,875.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,127,340	5,574,487	88,180,690
18. Gross Elec Ener (MWH)	376,524	1,880,606	29,536,667
19. Net Elec Ener (MWH)	354,717	1,766,770	27,661,188
20. Unit Service Factor	100.0	84.8	69.3
21. Unit Avail Factor	100.0	84.8	69.3
22. Unit Cap Factor (MDC Net)	92.6	67.4	53.3
23. Unit Cap Factor (DER Net)	88.6	64.6	51.0
24. Unit Forced Outage Rate	.0	2.7	15.2
25. Forced Outage Hours	.0	121.1	12,505.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

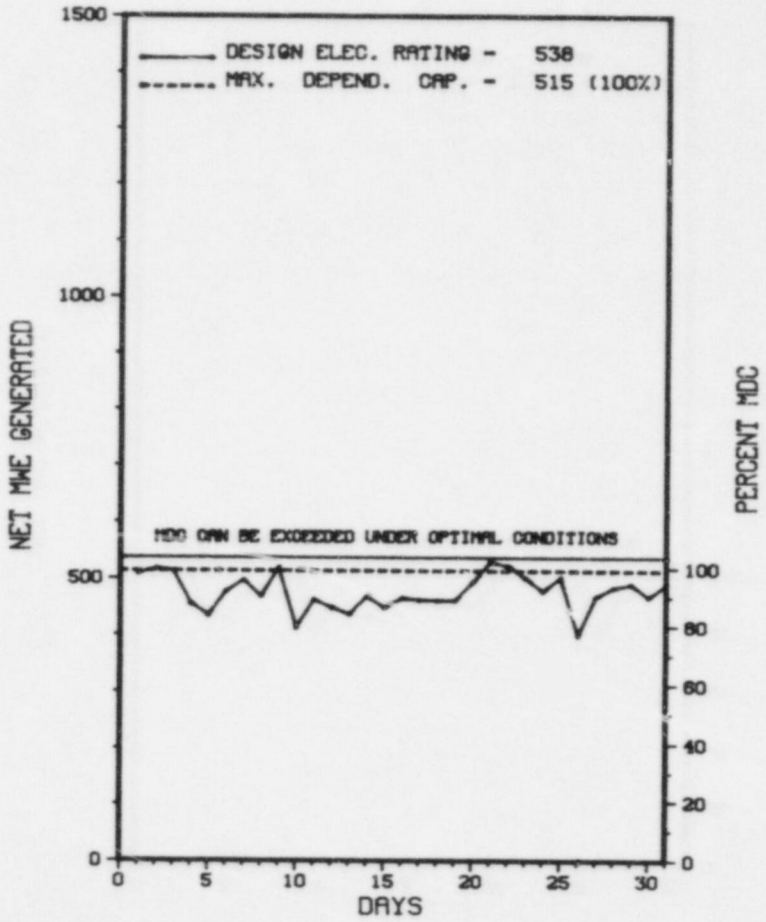
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* DUANE ARNOLD *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

DUANE ARNOLD



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* DUANE ARNOLD *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
7	07/10/86	S	0.0	F	5		ZZ	ZZZZZZ	POWER REDUCED APPROXIMATELY 21% FOR 12 HOURS AT THE REQUEST OF THE LOAD DISPATCHER (LOAD FOLLOWING).
8	07/26/86	S	0.0	F	5		ZZ	ZZZZZZ	POWER REDUCED APPROXIMATELY 21% FOR 24 HOURS AT THE REQUEST OF THE LOAD DISPATCHER (LOAD FOLLOWING).

****
* SUMMARY *
****

DUANE ARNOLD OPERATED WITH 2 REDUCTIONS DURING JULY.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* DUANE ARNOLD *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....IOWA

COUNTY.....LINN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...8 MI NW OF
CEDAR RAPIDS, IA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...MARCH 23, 1974

DATE ELEC ENER 1ST GENER...MAY 19, 1974

DATE COMMERCIAL OPERATE....FEBRUARY 1, 1975

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....CEDAR RAPIDS RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....IOWA ELECTRIC LIGHT & POWER

CORPORATE ADDRESS.....I E TOWERS, P.O. BOX 351
CEDAR RAPIDS, IOWA 52406

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....J. WEIBE

LICENSING PROJ MANAGER.....R. GILBERT
DOCKET NUMBER.....50-331

LICENSE & DATE ISSUANCE....DPR-49, FEBRUARY 22, 1974

PUBLIC DOCUMENT ROOM.....CEDAR RAPIDS PUBLIC LIBRARY
500 FIRST STREET, S.E.
CEDAR RAPIDS, IOWA 52401

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON MAY 20 THROUGH JULY 14 (86009): ROUTINE, UNANNOUNCED INSPECTION BY THE RESIDENT INSPECTORS OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, OPERATIONAL SAFETY, MAINTENANCE, SURVEILLANCE, LICENSEE EVENT REPORTS, OUTAGE ACTIVITIES, PLANT TRIPS, LOW-LEVEL RADIOACTIVE WASTE STORAGE FACILITY, RESOLUTION OF REGULATORY EFFECTIVENESS REVIEW TEAM COMMENTS, GENERIC LETTERS, TMI ACTION ITEMS, AND LICENSEE RESPONSE TO SELECTED SAFETY ISSUES. OF THE 12 AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON MAY 27-30 (86011): INCLUDED A REVIEW OF ALLEGATIONS RECEIVED BY NRC REGION III OF PERCEIVED SECURITY DEFICIENCIES IN THE LICENSEE'S SECURITY PROGRAM. THE INSPECTION BEGAN DURING THE DAY SHIFT AND SOME OFFSHIFT INSPECTION ACTIVITIES WERE PERFORMED. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THIS INSPECTION.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

****
* DUANE ARNOLD *
****

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

OPERATING ROUTINELY

LAST IE SITE INSPECTION DATE: 08/05/86

INSPECTION REPORT NO: 86013

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-17	06/13/86	07/11/86	MANUAL SCRAM IN RESPONSE TO FEEDWATER LEVEL CONTROL PROBLEMS DURING REACTOR STARTUP

1. Docket: 50-348 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: J. D. WOODARD (205) 899-5156

4. Licensed Thermal Power (MWt): 2652

5. Nameplate Rating (Gross MWe): 1045 X 0.85 = 888

6. Design Electrical Rating (Net MWe): 829

7. Maximum Dependable Capacity (Gross MWe): 873

8. Maximum Dependable Capacity (Net MWe): 827

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	75,959.0
13. Hours Reactor Critical	720.6	5,015.2	54,648.3
14. Rx Reserve Shtdwn Hrs	.0	.0	3,650.7
15. Hrs Generator On-Line	711.6	4,995.3	53,400.9
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,846,629	12,892,512	135,764,052
18. Gross Elec Ener (MWH)	597,758	4,248,844	43,451,028
19. Net Elec Ener (MWH)	565,850	4,030,184	41,027,902
20. Unit Service Factor	95.6	98.2	70.3
21. Unit Avail Factor	95.6	98.2	70.3
22. Unit Cap Factor (MDC Net)	92.0	95.8	67.4*
23. Unit Cap Factor (DER Net)	91.7	95.6	65.2
24. Unit Forced Outage Rate	4.4	1.8	10.8
25. Forced Outage Hours	32.4	91.7	6,474.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

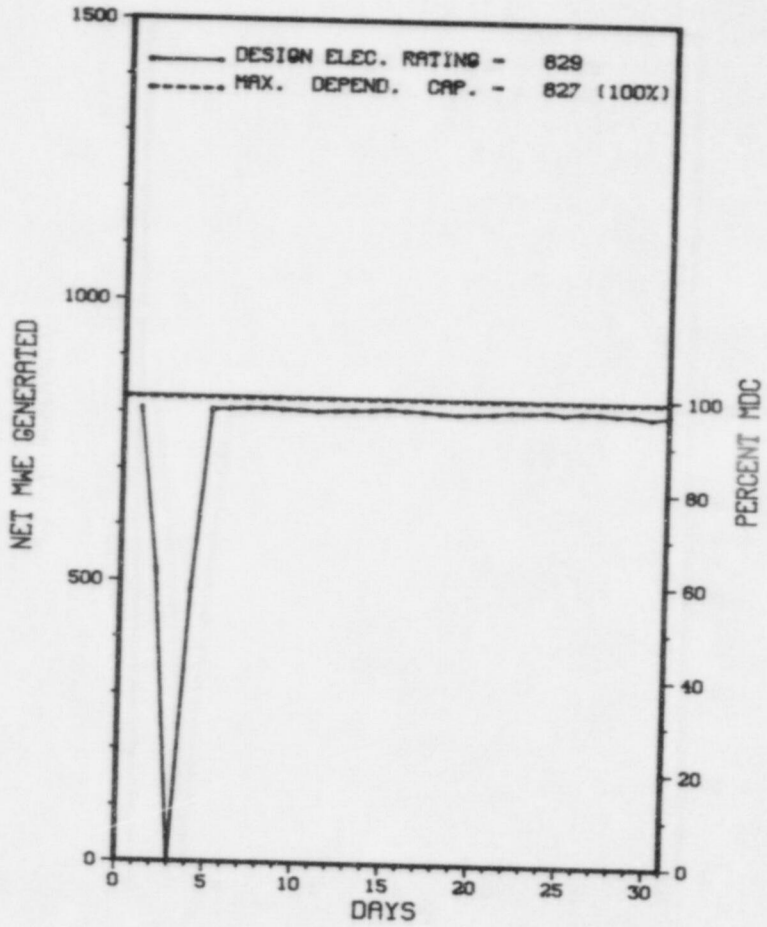
REFUELING/MAINTENANCE OUTAGE: 10/4/86, 44 DAYS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* FARLEY 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

FARLEY 1



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* FARLEY 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
004	07/02/86	F	32.4	A	3	86-008-00	AA	PEN	A REACTOR TRIP OCCURRED AS A RESULT OF A DROPPED CONTROL ROD. A SHORT OCCURRED IN THE CONTAINMENT ELECTRICAL PENETRATION MODULE BETWEEN THE TWO CONDUCTORS SUPPLYING POWER TO THE STATIONARY GRIPPER COIL OF THE ROD. THE ASSOCIATED FUSES FOR THE CONDUCTORS WERE ALSO BLOWN. THE AFFECTED CIRCUITS WERE REROUTED THROUGH SPARE TERMINALS IN THE PENETRATION. THE BLOWN FUSES WERE REPLACED AND ALL OTHER CONTROL RODS DRIVE SYSTEM CABLES WERE CHECKED FOR GROUNDS OR SHORTS. NO OTHER SHORTED OR GROUNDED CONDUCTORS WERE FOUND.

****
* SUMMARY *
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FARLEY 1 OPERATED WITH 1 OUTAGE FOR EQUIPMENT REPAIRS DURING JULY.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* FARLEY 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....ALABAMA

COUNTY.....HOUSTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...18 MI SE OF
DOTHAN, ALA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 9, 1977

DATE ELEC ENER 1ST GENER...AUGUST 18, 1977

DATE COMMERCIAL OPERATE....DECEMBER 1, 1977

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....CHATAHOOCHEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....ALABAMA POWER CO.

CORPORATE ADDRESS.....600 NORTH 18TH STREET
BIRMINGHAM, ALABAMA 35203

CONTRACTOR
ARCHITECT/ENGINEER.....SOUTHERN SERVICES INCORPORATED

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....DANIEL INTERNATIONAL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....W. BRADFORD

LICENSING PROJ MANAGER.....E. REEVES
DOCKET NUMBER.....50-348

LICENSE & DATE ISSUANCE....NPF-2, JUNE 25, 1977

PUBLIC DOCUMENT ROOM.....HOUSTON/LOVE MEMORIAL LIBRARY
212 W. BURDESHAW STREET
DOTHAN, ALABAMA 36301

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION APRIL 11 - MAY 10 AND JUNE 3 (86-10): THIS ROUTINE AND REACTIVE INSPECTION AND ENFORCEMENT CONFERENCE INCLUDED ONSITE INSPECTION IN THE AREAS OF MONTHLY SURVEILLANCE OBSERVATION, MONTHLY MAINTENANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, INOPERABLE ECCS SUBSYSTEM, INOPERABLE FIRE DOOR, ENGINEERED SAFETY SYSTEM INSPECTION, PART 21 REPORTS, AND REFUELING ACTIVITIES. TWO VIOLATIONS WERE IDENTIFIED: (1) VIOLATION OF TECHNICAL SPECIFICATION 3.5.2.D - EXCEEDING LIMITING CONDITION FOR OPERATION AND (2) VIOLATION OF TECHNICAL SPECIFICATION 3.7.12 - INOPERABLE FIRE BARRIER.

INSPECTION MAY 14 - JUNE 10 (86-11): THIS ROUTINE INSPECTION WAS CONDUCTED IN THE AREAS OF MONTHLY SURVEILLANCE OBSERVATION, MONTHLY MAINTENANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, LICENSEE EVENT REPORTS, ONSITE FOLLOWUP OF EVENTS, FOLLOWUP SURVEY OF LICENSEE RESPONSE TO SELECTED SAFETY ISSUES AND PLANT STARTUP FROM REFUELING OUTAGE. TWO VIOLATIONS WERE IDENTIFIED: VIOLATION OF TECHNICAL SPECIFICATION 3.7.12 - INOPERABLE FIRE DAMPER; VIOLATION OF TECHNICAL SPECIFICATION 6.8.1 - FAILURE TO FOLLOW PROCEDURES.

INSPECTION JUNE 16-20 (86-12): THIS ROUTINE, UNANNOUNCED INSPECTION INSPECTED: SECURITY SYSTEM POWER SUPPLY; LIGHTING, ACCESS CONTROLS-PERSONNEL; ACCESS CONTROLS-PACKAGES, VEHICLES AND PERSONNEL TRAINING AND QUALIFICATIONS. THERE WERE NO VIOLATIONS OR DEVIATIONS IDENTIFIED.

INSPECTION JUNE 11 - JULY 10 (86-13): THIS ROUTINE INSPECTION IN THE AREAS OF MONTHLY SURVEILLANCE OBSERVATION, MONTHLY MAINTENANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, ENGINEERED SAFETY FEATURE WALKDOWN, AND ONSITE FOLLOWUP OF EVENTS. ONE VIOLATION WAS IDENTIFIED: FAILURE TO UPDATE PROCEDURES AND DRAWINGS.

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* FARLEY 1 *
****

INSPECTION SUMMARY

ENFORCEMENT SUMMARY

CONTRARY TO TS 3.5.2D AND TS 6.8.1, FROM 10:00 A.M. ON APRIL 25, 1986 TO 9:45 A.M. ON APRIL 29, 1986, A PERIOD OF APPROXIMATELY 96 HOURS, WITH THE PLANT IN MODE 1, UNIT 1 ECCS SUBSYSTEM "B" TRAIN RHR PUMP SUCTION WAS NOT CAPABLE OF BEING TRANSFERRED TO THE CONTAINMENT SUMP. THE INOPERABLE SUBSYSTEM WAS NOT RESTORED TO OPERABLE STATUS WITHIN 72 HOURS, AND THE UNIT WAS NOT PLACED IN HOT STANDBY WITHIN THE NEXT 6 HOURS OR HOT SHUTDOWN WITHIN THE FOLLOWING 6 HOURS. IN ADDITION, ON APRIL 25, 1986, A SHIFT FOREMAN INSPECTOR (SFI) WHO WAS NOT THE DESIGNATED OPERATOR ATTEMPTED TO REHANG A TAG ON UNIT 2'S MOTOR OPERATED VALVE (MOV) 8811-B'S BREAKER FV-B5. THE SFI MISTAKENLY HUNG THE TAG ON THE UNIT 1 MOV 8811-B'S BREAKER FV-B5, NOTICED THE BREAKER WAS NOT OPEN AS THE TAB INDICATED, OPENED THE BREAKER WITHOUT PROPER TAGGING ORDERS, AND RENDERED CERTAIN FUNCTIONS OF THE UNIT 1 EMERGENCY CORE COOLING SYSTEM'S (ECCS) SUBSYSTEM INOPERABLE. IN ADDITION, FROM APRIL 25, 1986 AT 10:00 A.M. UNTIL APRIL 29, 1986 AT 9:45 A.M., DURING WHICH TIME 12 SHIFT RELIEFS OCCURRED, UNIT 1 CONTROL ROOM PERSONNEL DID NOT OBSERVE THE ABSENCE OF INDICATING LIGHTS FOR MOV 8811-B DURING SHIFT CHANGE WALKDOWNS.
(8601 3)

CONTRARY TO TS 6.8.1 AND ADMINISTRATIVE PROCEDURE 35, ON MAY 16, 1986, COMBUSTIBLE LIQUIDS WERE FOUND UNATTENDED IN THE HOT MACHINE SHOP LOCATED IN THE AUXILIARY BUILDING.
(8601 5)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATIONS.

LAST IE SITE INSPECTION DATE: JUNE 11 - JULY 10, 1986 +

INSPECTION REPORT NO: 50-348/86-13 +

REPORTS FROM LICENSEE

FARLEY 1

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

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1. Docket: 50-364 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: J. D. WOODARD (205) 899-5156

4. Licensed Thermal Power (MWt): 2652

5. Nameplate Rating (Gross MWe): 860

6. Design Electrical Rating (Net MWe): 829

7. Maximum Dependable Capacity (Gross MWe): 872

8. Maximum Dependable Capacity (Net MWe): 829

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	43,872.0
13. Hours Reactor Critical	651.9	3,892.3	37,692.9
14. Rx Reserve Shtdwn Hrs	.0	.0	138.4
15. Hrs Generator On-Line	640.7	3,804.8	37,197.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,625,000	9,503,899	95,347,297
18. Gross Elec Ener (MWH)	529,128	3,141,914	30,864,524
19. Net Elec Ener (MWH)	499,398	2,964,882	29,254,122
20. Unit Service Factor	86.1	74.8	84.8
21. Unit Avail Factor	86.1	74.8	84.8
22. Unit Cap Factor (MDC Net)	81.0	70.3	80.4
23. Unit Cap Factor (DER Net)	81.0	70.3	80.4
24. Unit Forced Outage Rate	13.9	4.2	4.7
25. Forced Outage Hours	103.3	166.0	1,852.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

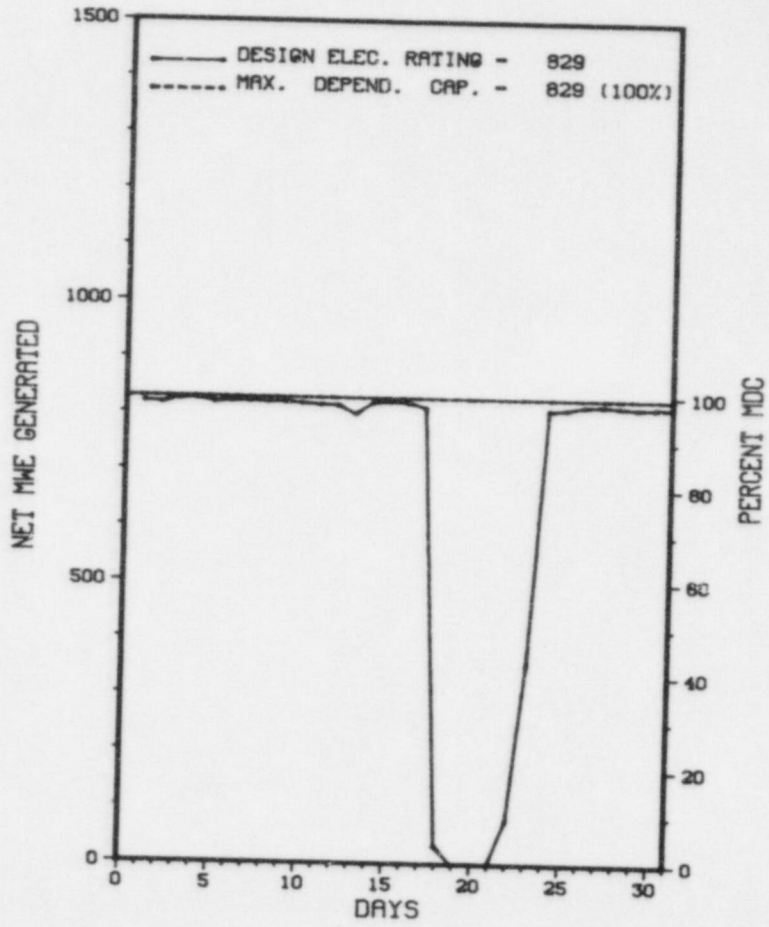
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* FARLEY 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

FARLEY 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

FARLEY 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
007	07/18/86	F	103.3	A	1	86-009-00	BQ	MO	THE 2B CHARGING PUMP TRIPPED ON OVERLOAD. THE MOTOR WAS FOUND TO HAVE FAILED DUE TO AN ARC IN THE WINDING. NO SPARES WERE AVAILABLE.

SUMMARY

FARLEY 2 OPERATED WITH 1 OUTAGES FOR EQUIPMENT REPAIRS DURING JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* FARLEY 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....ALABAMA

COUNTY.....HOUSTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...28 MI SE OF
DOTHAN, ALA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 5, 1981

DATE ELEC ENER 1ST GENER...MAY 25, 1981

DATE COMMERCIAL OPERATE....JULY 30, 1981

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....CHATAHOOCHEE RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....ALABAMA POWER CO.

CORPORATE ADDRESS.....600 NORTH 18TH STREET
BIRMINGHAM, ALABAMA 35203

CONTRACTOR
ARCHITECT/ENGINEER.....SOUTHERN SERVICES INCORPORATED

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....W. BRADFORD

LICENSING PROJ MANAGER.....E. REEVES
DOCKET NUMBER.....50-364

LICENSE & DATE ISSUANCE....NPF-8, MARCH 31, 1981

PUBLIC DOCUMENT ROOM.....HOUSTON/LOVE MEMORIAL LIBRARY
212 W. BURDESHAW STREET
DOTHAN, ALABAMA 36301

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION APRIL 11 - MAY 10 AND JUNE 3 (86-10): THIS ROUTINE AND REACTIVE INSPECTION AND ENFORCEMENT CONFERENCE INCLUDED ONSITE INSPECTION IN THE AREAS OF MONTHLY SURVEILLANCE OBSERVATION, MONTHLY MAINTENANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, INOPERABLE ECCS SUBSYSTEM, INOPERABLE FIRE DOOR, ENGINEERED SAFETY SYSTEM INSPECTION, PART 21 REPORTS, AND REFUELING ACTIVITIES. TWO VIOLATIONS WERE IDENTIFIED: (1) VIOLATION OF TECHNICAL SPECIFICATION 3.5.2.D - EXCEEDING LIMITING CONDITION FOR OPERATION AND (2) VIOLATION OF TECHNICAL SPECIFICATION 3.7.12 - INOPERABLE FIRE BARRIER.

INSPECTION MAY 14 - JUNE 10 (86-11): THIS ROUTINE INSPECTION WAS CONDUCTED IN THE AREAS OF MONTHLY SURVEILLANCE OBSERVATION, MONTHLY MAINTENANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, LICENSEE EVENT REPORTS, ONSITE FOLLOWUP OF EVENTS, FOLLOWUP SURVEY OF LICENSEE RESPONSE TO SELECTED SAFETY ISSUES AND PLANT STARTUP FROM REFUELING OUTAGE. TWO VIOLATIONS WERE IDENTIFIED: VIOLATION OF TECHNICAL SPECIFICATION 3.7.12 - INOPERABLE FIRE DAMPER; VIOLATION OF TECHNICAL SPECIFICATION 6.8.1 - FAILURE TO FOLLOW PROCEDURES.

INSPECTION JUNE 16-20 (86-12): THIS ROUTINE, UNANNOUNCED INSPECTION INSPECTED: SECURITY SYSTEM POWER SUPPLY; LIGHTING, ACCESS CONTROLS-PERSONNEL; ACCESS CONTROLS-PACKAGES, VEHICLES AND PERSONNEL TRAINING AND QUALIFICATIONS. THERE WERE NO VIOLATIONS OR DEVIATIONS IDENTIFIED.

INSPECTION JUNE 11 - JULY 10 (86-13): THIS ROUTINE INSPECTION IN THE AREAS OF MONTHLY SURVEILLANCE OBSERVATION, MONTHLY MAINTENANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, ENGINEERED SAFETY FEATURE WALKDOWN, AND ONSITE FOLLOWUP OF EVENTS. ONE VIOLATION WAS IDENTIFIED: FAILURE TO UPDATE PROCEDURES AND DRAWINGS.

INSPECTION SUMMARY

ENFORCEMENT SUMMARY

CONTRARY TO TS 3.7.12, ON APRIL 29, 1986, AT 9:30 A.M., FIRE DOOR NO. 2406, THE HOT MACHINE SHOP ENTRANCE TO UNIT 2, WAS FOUND BLOCKED OPEN BY A RUBBER HOSE WITHOUT A CONTINUOUS FIRE WATCH POSTED NOR WAS THE OPERABILITY OF FIRE DETECTORS ON AT LEAST ONE SIDE OF THE DOOR VERIFIED OPERABLE OR AN HOURLY FIRE WATCH PATROL INITIATED.
(8601 4)

CONTRARY TO TS 6.8.1 AND ADMINISTRATIVE PROCEDURE 35, ON MAY 16, 1986, COMBUSTIBLE LIQUIDS WERE FOUND UNATTENDED IN THE HOT MACHINE SHOP LOCATED IN THE AUXILIARY BUILDING. CONTRARY TO TS 3.7.12, AT 8:55 A.M. ON MAY 21, 1986, UNIT 2 FIRE DAMPER PENETRATION 05-139-06, LOCATED BETWEEN THE ELECTRICAL PENETRATION ROOM AND THE PIPING PENETRATION ROOM WAS NOT FUNCTIONAL DUE TO A SOUND POWERED TELEPHONE CORD BLOCKING THE CLOSURE OF THE DAMPER.
(8601 5)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

LICENSEE CONTINUES TENDON FIELD ANCHORS INSPECTION.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 11 - JULY 10, 1986 +

INSPECTION REPORT NO: 50-364/86-13 +

REPORTS FROM LICENSEE

FARLEY 2

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-006	05/21/86	06/17/86	FIRE DAMPER IN PENETRATION 05-139-06 INOPERABLE BECAUSE A CABLE FOR SOUND POWERED HEADPHONES ROUTED THROUGH FIRE DAMPER; CAUSE - PERSONNEL ERROR.
86-007	06/08/86	07/03/86	REACTOR TRIP DUE TO MALFUNCTION OF BOTH CONTROL ROD DRIVE MOTOR GENERATOR SETS.

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1. Docket: 50-333 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: J. COOK (315) 342-3840

4. Licensed Thermal Power (MWt): 2436

5. Nameplate Rating (Gross MWe): 981 X 0.9 = 883

6. Design Electrical Rating (Net MWe): 816

7. Maximum Dependable Capacity (Gross MWe): 823

8. Maximum Dependable Capacity (Net MWe): 796

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	96,528.0
13. Hours Reactor Critical	719.0	4,656.2	70,071.8
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	707.2	4,532.8	68,059.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,680,120	10,761,504	146,474,218
18. Gross Elec Ener (MWH)	560,760	3,656,610	49,687,950
19. Net Elec Ener (MWH)	541,760	3,532,515	48,097,040
20. Unit Service Factor	95.1	89.1	70.5
21. Unit Avail Factor	95.1	89.1	70.5
22. Unit Cap Factor (MDC Net)	91.5	87.0	64.4*
23. Unit Cap Factor (DER Net)	89.2	85.0	61.1
24. Unit Forced Outage Rate	4.9	3.0	12.6
25. Forced Outage Hours	36.8	142.4	9,988.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

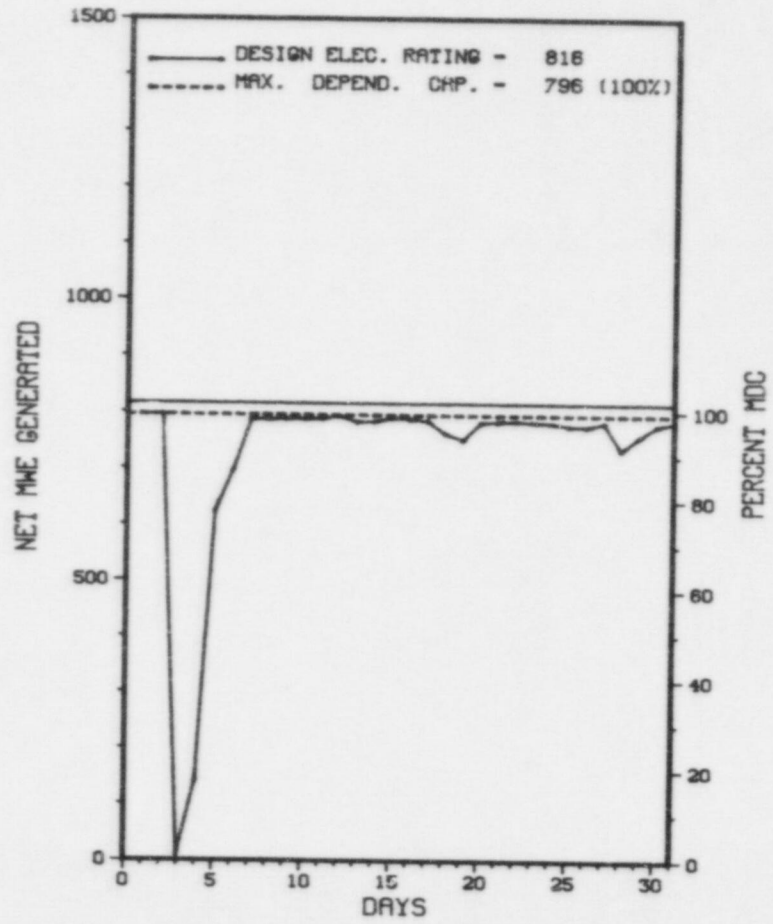
MAINTENANCE: 09/86 - 10 DAYS.

27. If Currently Shutdown Estimated Startup Date: N/A

FITZPATRICK

AVERAGE DAILY POWER LEVEL (MWe) PLOT

FITZPATRICK



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

FITZPATRICK

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
4	07/03/86	F	36.8	A	3		EL	IC	RX SCRAM CAUSED BY GENERATOR/TURBINE TRIP ON LOAD REJECT, THE RESULT OF A FAILED PK TEST BLOCK IN THE 345KV BUS CURRENT TRANSFORMER DIFFERENTIAL CIRCUITRY.

SUMMARY

THE FITZPATRICK PLANT OPERATED AT NEAR FULL THERMAL POWER FOR THIS REPORTING PERIOD WITH ONE TRIP ON 860703 FOR EQUIPMENT REPAIRS.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* FITZPATRICK *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....NEW YORK

COUNTY.....OSWEGO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...8 MI NE OF
OSWEGO, NY

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...NOVEMBER 17, 1974

DATE ELEC ENER 1ST GENER...FEBRUARY 1, 1975

DATE COMMERCIAL OPERATE....JULY 28, 1975

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE ONTARIO

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NEW YORK POWER AUTHORITY

CORPORATE ADDRESS.....10 COLUMBUS CIRCLE
NEW YORK, NEW YORK 10019

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....A. LUPTAK

LICENSING PROJ MANAGER.....H. ABELSON
DOCKET NUMBER.....50-333

LICENSE & DATE ISSUANCE....DPR-59, OCTOBER 17, 1974

PUBLIC DOCUMENT ROOM.....STATE UNIVERSITY COLLEGE OF OSWEGO
PENFIELD LIBRARY - GOVERNMENT DOCUMENTS COL
OSWEGO, NY 13126
(315) 341-2323

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

****
* FITZPATRICK *
****

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-285 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: T. P. MATTHEWS (402) 536-4733

4. Licensed Thermal Power (MWt): 1500

5. Nameplate Rating (Gross MWe): 591 X 0.85 = 502

6. Design Electrical Rating (Net MWe): 478

7. Maximum Dependable Capacity (Gross MWe): 502

8. Maximum Dependable Capacity (Net MWe): 478

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	112,632.0
13. Hours Reactor Critical	711.2	4,858.7	87,325.0
14. Rx Reserve Shtdwn Hrs	.0	.0	1,309.5
15. Hrs Generator On-Line	703.1	4,674.8	85,747.8
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,003,268	6,206,805	109,957,850
18. Gross Elec Ener (MWH)	327,442	2,052,799	36,349,223
19. Net Elec Ener (MWH)	311,118	1,955,737	34,433,622
20. Unit Service Factor	94.5	91.9	76.1
21. Unit Avail Factor	94.5	91.9	76.1
22. Unit Cap Factor (MDC Net)	87.5	80.4	66.2*
23. Unit Cap Factor (DER Net)	87.5	80.4	64.0
24. Unit Forced Outage Rate	5.5	.9	3.4
25. Forced Outage Hours	40.9	40.9	1,791.2

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

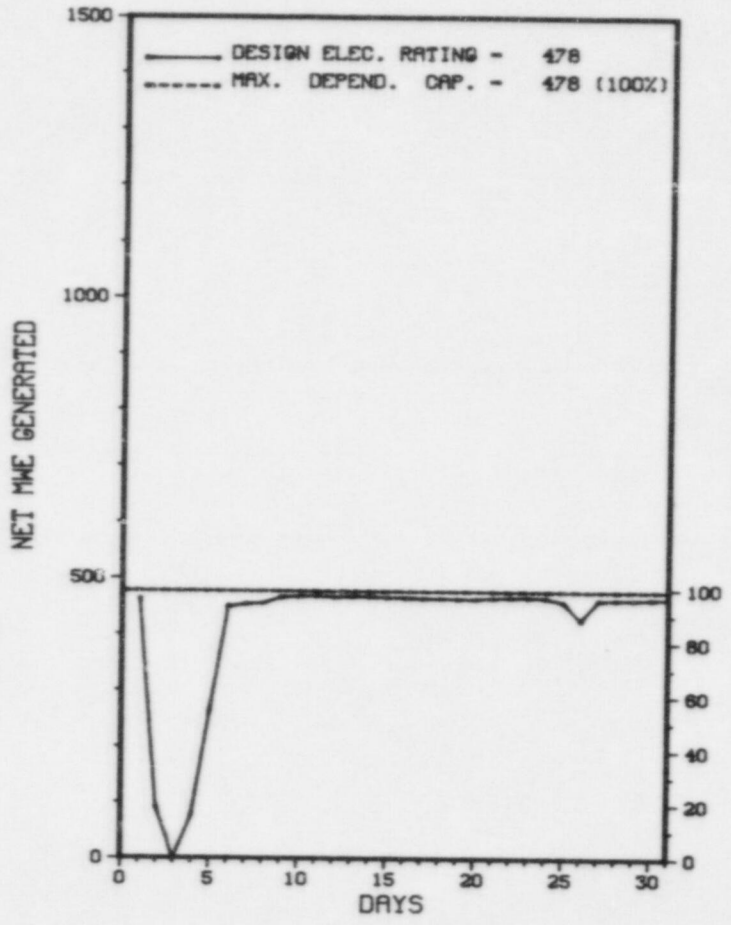
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* FORT CALHOUN 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

FORT CALHOUN 1



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* FORT CALHOUN 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-03	07/02/86	F	39.3	A	3	8601	EF	INUT	ON JULY 2, 1986, AT 0534, THE REACTOR AND TURBINE-GENERATOR WERE AUTOMATICALLY TRIPPED ON LOW STEAM GENERATOR LEVEL AFTER THE FAILURE OF A SAFETY RELATED INSTRUMENT INVERTERS. THE UNIT RETURNED TO SERVICE JULY 3, AT 2052. SEE LER-86-01 FOR DETAILS OF THE CORRECTIVE ACTIONS TAKEN.
86-04	07/04/86	F	1.6	H	3		XX	XXXX	ON JULY 4, AT 0138, THE TURBINE-GENERATOR WAS TAKEN OFF-LINE TO FIX AN EHC LEAK. THE UNIT RETURNED TO SERVICE THE SAME DAY AT 0316.

****
* SUMMARY *
****

FORT CALHOUN OPERATED WITH 2 OUTAGES DURING JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* FORT CALHOUN 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....NEBRASKA

COUNTY.....WASHINGTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...19 MI N OF
OMAHA, NEB

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 6, 1973

DATE ELEC ENER 1ST GENER...AUGUST 25, 1973

DATE COMMERCIAL OPERATE....JUNE 20, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MISSOURI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....OMAHA PUBLIC POWER DISTRICT

CORPORATE ADDRESS.....1623 HARNEY STREET
OMAHA,, NEBRASKA 68102

CONTRACTOR
ARCHITECT/ENGINEER.....GIBBS, HILL, DURHAM & RICHARDSON

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....GIBBS, HILL, DURHAM & RICHARDSON

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....P. HARRELL

LICENSING PROJ MANAGER.....D. SELLS
DOCKET NUMBER.....50-285

LICENSE & DATE ISSUANCE....DPR-40, AUGUST 9, 1973

PUBLIC DOCUMENT ROOM.....W. DALE CLARK LIBRARY
215 S. 15TH STREET
OMAHA, NEBRASKA 68102

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION CONDUCTED MAY 1-31, 1986 (86-14) ROUTINE, UNANNOUNCED INSPECTION OF OPERATIONAL SAFETY VERIFICATION, MAINTENANCE, SURVEILLANCE, PLANT TOURS, SAFETY-RELATED SYSTEM WALKDOWNS, FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS, FOLLOWUP ON A LICENSEE EVENT REPORT (LER), AND FOLLOWUP ON A 10 CFR PART 21 REPORT ON VALCOR VALVES. WITHIN THE AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED.

INSPECTION CONDUCTED MAY 19-23, 1986 (86-15) ROUTINE, UNANNOUNCED INSPECTION CONSISTING OF SECURITY ORGANZATION, MANAGEMENT, RECORDS AND REPORTS, AND PERSONNEL TRAINING AND QUALIFICATIONS. WITHIN THE AREAS INSPECTED, THREE APPARENT VIOLATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED JUNE 9-13, 1986 (86-17) ROUTINE, UNANNOUNCED INSPECTION CONSISTING OF PHYSICAL BARRIERS - VITAL AREA (VA), DETECTION AIDS - VA, ACCESS CONTROL - PERSONNEL, SECURITY SYSTEM POWER SUPPLY, ALARM STATIONS, AND COMMUNICATIONS. WITHIN THE AREAS INSPECTED AN APPANENT VIOLATION WAS IDENTIFIED.

INSPECTION CONDUCTED JUNE 1-30, 1986 (86-18) ROUTINE, UNANNOUNCED INSPECTION INCLUDING OPERATIONAL SAFETY VERIFICATION; MAINTENANCE; SURVEILLANCE; PLANT TOURS; SAFETY-RELATED SYSTEM WALKDOWN; FOLLOWUP ON A PREVIOUSLY IDENTIFIED ITEM; FOLLOWUP ON IE INFORMATION NOTICES AND IE BULLETINS ISSUED FOR INFORMATION ONLY; FOLLOWUP ON A LICENSEE EVENT REPORT; AND FOLLOWUP ON NUREG-0737 (TMI) I.D.2, IMPLEMFNTATION OF THE SPDS. WITHIN THE NINE AREAS INSPECTED, ONE DEVIATION WAS IDENTIFIED.

****
* FORT CALHOUN 1 *
****

ENFORCEMENT SUMMARY

CONTRARY TO PART 50, CRITERION V; DRAWINGS 11405-E-60 AND 11405-E-151, CABLE AND CABLE TRAYS ARE NOT BEING MAINTAINED IN ACCORDANCE WITH BASIC DESIGN DOCUMENTATION.
(8601 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

LAST IE SITE INSPECTION DATE: JUNE 1-30, 1986

INSPECTION REPORT NO: 50-285/86-18

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-267 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: FRANK NOVACHEK (303) 785-2224
4. Licensed Thermal Power (MWt): 842
5. Nameplate Rating (Gross MWe): 403 X 0.85 = 343
6. Design Electrical Rating (Net MWe): 330
7. Maximum Dependable Capacity (Gross MWe): 342
8. Maximum Dependable Capacity (Net MWe): 330
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	62,112.0
13. Hours Reactor Critical	.0	2,456.6	30,492.8
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	1,087.1	19,550.6
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	370,315	10,113,482
18. Gross Elec Ener (MWH)	0	85,402	3,334,290
19. Net Elec Ener (MWH)	-2,215	61,174	2,957,220
20. Unit Service Factor	.0	21.4	31.5
21. Unit Avail Factor	.0	21.4	31.5
22. Unit Cap Factor (MDC Net)	.0	3.6	14.4
23. Unit Cap Factor (DER Net)	.0	3.6	14.4
24. Unit Forced Outage Rate	100.0	78.6	59.8
25. Forced Outage Hours	744.0	3,999.9	29,089.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

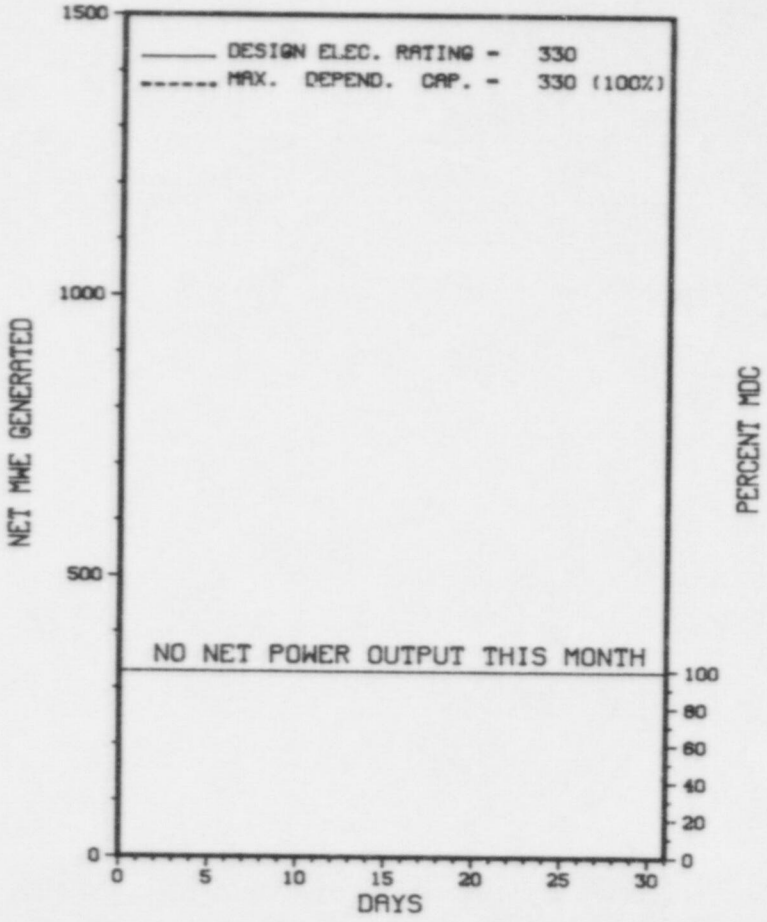
NONE

27. If Currently Shutdown Estimated Startup Date: 10/09/86

****
* FORT ST VRAIN *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

FORT ST VRAIN



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* FORT ST VRAIN *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrance
86-03	05/30/86	F	744.0	D	1	4	ZZ	ZZZZZ	ENVIRONMENTAL QUALIFICATION MODIFICATIONS.

****
* SUMMARY *
****

FORT ST. VRAIN REMAINS SHUTDOWN FOR ENVIRONMENTAL QUALIFICATION MODIFICATIONS.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* FORT ST VRAIN *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....COLORADO

COUNTY.....WELD

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...35 MI N OF
DENVER, COL

TYPE OF REACTOR.....HTGR

DATE INITIAL CRITICALITY...JANUARY 31, 1974

DATE ELEC ENER 1ST GENER...DECEMBER 11, 1976

DATE COMMERCIAL OPERATE....JULY 1, 1979

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....S. PLATTE RIVER

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PUBLIC SERVICE OF COLORADO

CORPORATE ADDRESS.....P.O. BOX 840
DENVER, COLORADO 80201

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ATOMIC CORP.

CONSTRUCTOR.....EBASCO

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....R. FARRELL

LICENSING PROJ MANAGER.....C. HINSON
DOCKET NUMBER.....50-267

LICENSE & DATE ISSUANCE....DPR-34, DECEMBER 21, 1973

PUBLIC DOCUMENT ROOM.....GREELEY PUBLIC LIBRARY
CITY COMPLEX BUILDING
GREELEY, COLORADO 80631

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION CONDUCTED MARCH 31 - APRIL 4, 1986 (86-09) ROUTINE, ANNOUNCED INSPECTION OF LICENSEE MAINTENANCE ACTIVITIES, COVERING THE MAINTENANCE PROGRAM, MAINTENANCE PROGRAM IMPLEMENTATION, INSTRUMENTATION AND CONTROL (I&C) MAINTENANCE, AND ELECTRICAL MAINTENANCE. WITHIN THE AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED.

INSPECTION CONDUCTED MAY 1 - JUNE 6, 1986 (86-13) ROUTINE, UNANNOUNCED INSPECTION OF OPERATIONAL SAFETY VERIFICATION, SURVEILLANCES, ENGINEERED SAFETY FEATURES, MAINTENANCE, AND SECURITY. WITHIN THE FIVE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED MAY 12-16, 1986 (86-14) ROUTINE, UNANNOUNCED INSPECTION OF PHYSICAL BARRIERS - VITAL AREA (VA), ACCESS CONTROL - PERSONNEL, DETECTION AIDS - VA, ALARM STATIONS, SECURITY SYSTEMS POWER SUPPLY, AND COMMUNICATIONS. WITHIN THE SIX AREAS INSPECTED, THREE VIOLATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED MAY 19-22, 1986 (86-15) ROUTINE, UNANNOUNCED EMERGENCY PREPAREDNESS INSPECTION IN THE AREAS OF EMERGENCY DETECTION AND CLASSIFICATION, PROTECTIVE ACTION DECISIONMAKING, DOSE CALCULATION AND ASSESSMENT, NOTIFICATION AND COMMUNICATION, PUBLIC INFORMATION PROGRAM, AND SHIFT STAFFING AND AUGMENTATION. WITHIN THE EMERGENCY RESPONSE AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED.

INSPECTION CONDUCTED JUNE 16-20, 1986 (86-18) ROUTINE, UNANNOUNCED INSPECTION OF ACCESS CONTROL - VEHICLES, ACCESS CONTROL - PACKAGES, ACCESS CONTROL - PERSONNEL, AND COMPENSATORY MEASURES. WITHIN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE

****
* FORT ST VRAIN *
****

INSPECTION SUMMARY

IDENTIFIED.

INSPECTION CONDUTED JUNE 23-27, 1986 (86-19) ROUTINE, UNANNOUNCED INSPECTION OF THE LICENSEE'S QUALITY ASSURANCE PROGRAM EQUIPMENT QUALIFICATION (EQ) OUTAGE PLANNING, AND REVIEW OF 10 CFR 50, APPENDIX R. WITHIN THE AREAS INSPECTED, ONE APPARENT VIOLATION WAS IDENTIFIED.

ENFORCEMENT SUMMARY

CONTRARY TO 10CFR 50, CRITERION VI, THE LICENSEE WAS USING SCHERNATIC DRAWINGS THAT WERE NOT UP TO DATE. (8600 4)

SUITIBILITY, TRAINING AND QUALIFICATIONS. SAFEGUARDS INFO. INADEQUATE ACCESS - CONTROL- FAILURE TO MAINTAIN POSITIVE ACCESS CONTROL OF PERSONNEL IAW 10CFR 73.55(D)(7) UNBADGED INDIVIDUALS IN PROTECTED AREA ACCESS - CONTROL PERSONNEL - FAILURE TO ESCORT 10 CFR 50.47(B)(15) REQUIRES THAT ADEQUATE PROVISIONS EXIST IN THE EMERGENCY PLAN TO ENSURE THAT RADIOLOGICAL EMERGENCY RESPONSE TRAINING IS PROVIDED TO THOSE WHO MAT BE CALLED ON TO ASSIST IN AN EMERGENCY. SECTION 8.1.1 OF THE EMERGENCY PLAN STATES, IN PART, "A TRAINING AND ANNUAL RETRAINING PROGRAM IS IN EFFECT TO ENSURE THAT STATION PERSONNEL WHO ACTIVELY PARTICIPATE IN EMERGENCY SITUATIONS ARE FAMILIAR WITH THE CONTENTS AND RESPONSES SET FORTH IN THIS RERP. THE FT.ST.VRAIN TRAINING PROGRAMS ADMINISTRATIVE MANUAL (TPAM), SITUATION RESPONSE TRAINING PROGRAM - RERP, DELINEATES SPECIFIC TRAINING...." CONTRARY TO THE ABOVE, ON MAY22, 1986, THE NRC INSPECTORS DETERMINED THAT TWO KEY PERSONS ASSIGNED EMERGENCY RESPONSE FUNCIONS (CORPORATE EMERGENCY DIRECTOR) HAD NOT RECEIVED ALL OF THE EMERGENCY RESPONSE TRAINING DELINEATED IN THE TPAM PRIOR TO HAVING BEEN ASSIGNED TO THE EMERGENCY RESPONSE ORGANIZATION, AS EVIDENCED BY THE LACK OF RECORDS DOCUMENTING THE COMPLETION OF REQUIRED TRAINING. (8601 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

R.O.WILLIAMS NAMED VICE PRESIDENT, NUCLEAR OPERATIONS

PLANT STATUS:

SHUTDOWN FOR EQUIPMENT QUALIFICATION WORK

LAST IE SITE INSPECTION DATE: JUNE 23-27, 1986

INSPECTION REPORT NO: 50-267/86-19

REPORTS FROM LICENSEE

FORT ST VRAIN

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

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1. Docket: 50-244 O P E R A T I N G S T A T U S

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: ANDREW MC NAMARA (315) 524-4446

4. Licensed Thermal Power (MWt): 1520

5. Nameplate Rating (Gross MWe): 608 X 0.85 = 517

6. Design Electrical Rating (Net MWe): 470

7. Maximum Dependable Capacity (Gross MWe): 490

8. Maximum Dependable Capacity (Net MWe): 470

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	146,207.0
13. Hours Reactor Critical	711.9	4,064.6	112,351.2
14. Rx Reserve Shtdwn Hrs	.0	.0	1,687.7
15. Hrs Generator On-Line	692.0	4,015.5	110,009.3
16. Unit Reserve Shtdwn Hrs	.0	.0	8.5
17. Gross Therm Ener (MWH)	1,035,216	5,873,136	153,595,361
18. Gross Elec Ener (MWH)	344,174	1,976,496	50,287,353
19. Net Elec Ener (MWH)	327,280	1,879,116	47,682,434
20. Unit Service Factor	93.0	78.9	75.2
21. Unit Avail Factor	93.0	78.9	75.2
22. Unit Cap Factor (MDC Net)	93.6	78.6	70.9*
23. Unit Cap Factor (DER Net)	93.6	78.6	70.9*
24. Unit Forced Outage Rate	7.0	1.3	6.9
25. Forced Outage Hours	52.0	52.0	4,295.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

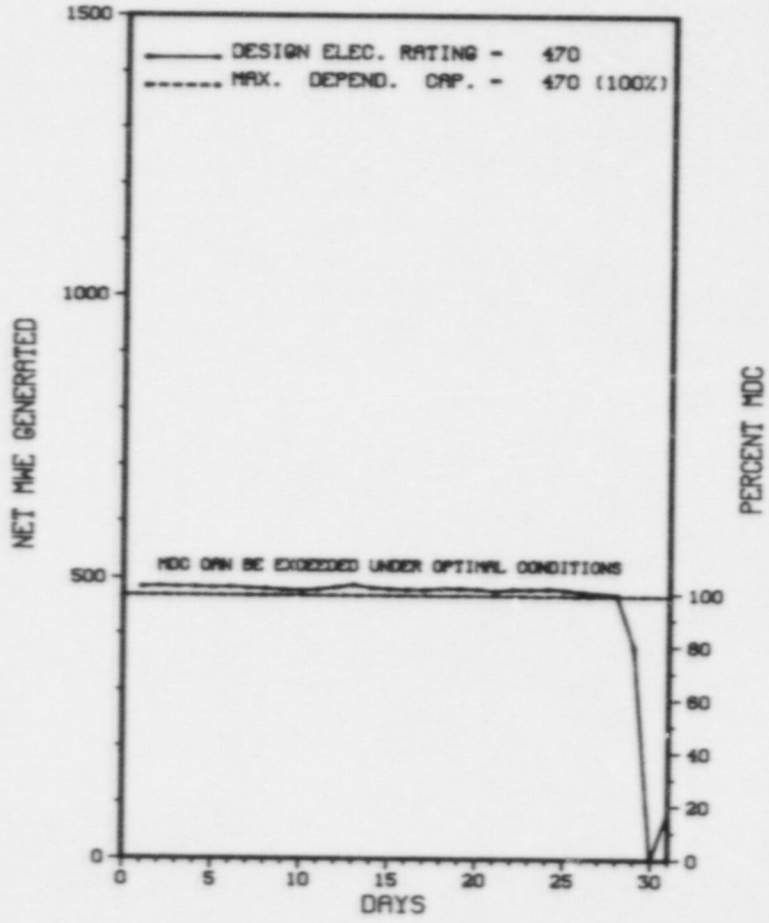
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* GINNA *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

GINNA



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

U N I T S H U T D O W N S / R E D U C T I O N S

****
* GINNA *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-2	07/29/86	F	39.0	A	1	86-4	HJ	PIPEXX	2A MSR DRAIN LINE TO 5B HEATER - ELBOW RUPTURE.
86-3	07/30/86	F	13.0	A	3	86-5	IA	RELAYX	AUTOMATIC REACTOR TRIP DUE TO TWO FAULTY RELAYS IN THE "A" TRAIN OF THE INTERMEDIATE RANGE BLOCKING CIRCUIT.

****
* SUMMARY *
****

GINNA OPERATED WITH 2 OUTAGES FOR EQUIPMENT REPAIRS IN JULY.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* GINNA *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....NEW YORK

COUNTY.....WAYNE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...15 MI NE OF
ROCHESTER, NY

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...NOVEMBER 8, 1969

DATE ELEC ENER 1ST GENER...DECEMBER 2, 1969

DATE COMMERCIAL OPERATE....JULY 1, 1970

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE ONTARIO

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....ROCHESTER GAS & ELECTRIC

CORPORATE ADDRESS.....89 EAST AVENUE
ROCHESTER, NEW YORK 14604

CONTRACTOR
ARCHITECT/ENGINEER.....GILBERT ASSOCIATES

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....W. COOK

LICENSING PROJ MANAGER.....M. FAIRTILE
DOCKET NUMBER.....50-244

LICENSE & DATE ISSUANCE....DPR-18, DECEMBER 10, 1984

PUBLIC DOCUMENT ROOM.....ROCHESTER PUBLIC LIBRARY
BUSINESS AND SOCIAL SCIENCE DIVISION
115 SOUTH AVENUE
ROCHESTER, NEW YORK 14604

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-416 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: S. H. HOBBS (601) 969-2458
4. Licensed Thermal Power (MWt): 3833
5. Nameplate Rating (Gross MWe): 1373
6. Design Electrical Rating (Net MWe): 1250
7. Maximum Dependable Capacity (Gross MWe): 1157
8. Maximum Dependable Capacity (Net MWe): 1108
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	9,504.0
13. Hours Reactor Critical	715.2	4,735.4	7,618.8
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	700.2	4,614.8	7,307.3
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,040,720	12,582,529	21,663,360
18. Gross Elec Ener (MWH)	598,250	3,745,780	6,538,670
19. Net Elec Ener (MWH)	567,106	3,544,276	6,198,425
20. Unit Service Factor	94.1	90.7	76.9
21. Unit Avail Factor	94.1	90.7	76.9
22. Unit Cap Factor (MDC Net)	68.8	62.9	58.9
23. Unit Cap Factor (DER Net)	61.0	55.7	52.2
24. Unit Forced Outage Rate	5.9	9.3	9.7
25. Forced Outage Hours	43.8	472.2	781.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

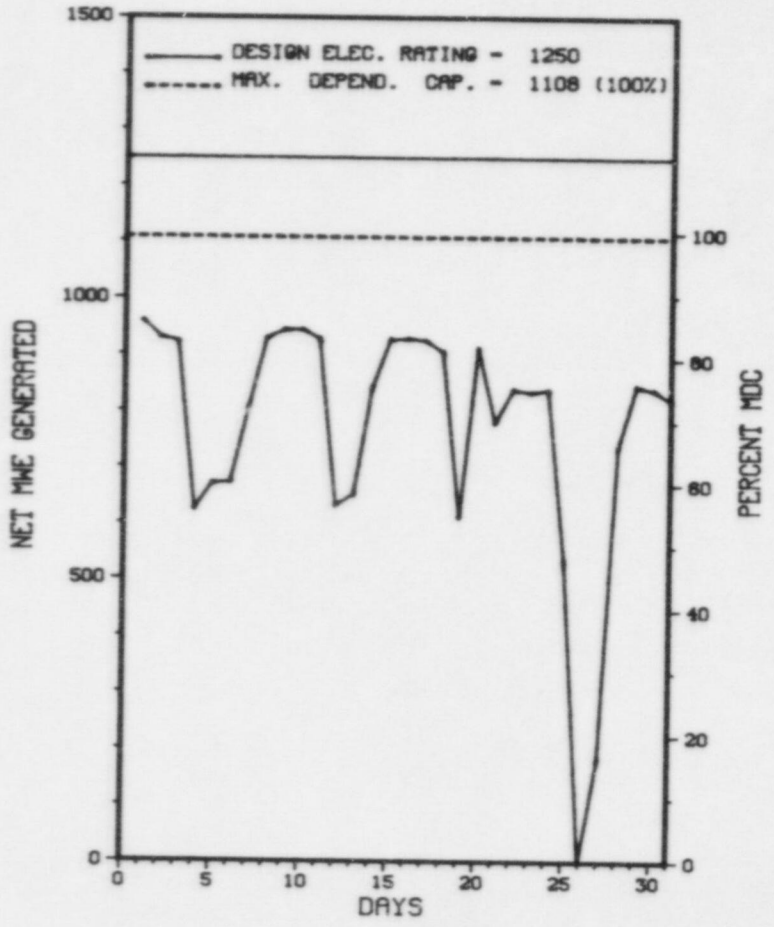
REFUELING OUTAGE: 09/5/86; APPROX. 60 DAYS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* GRAND GULF 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

GRAND GULF 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

GRAND GULF 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-09	07/25/86	F	43.8	H	3	86-025	ZZ	ZZZZZZ	A POWER SUPPLY BREAKER WAS INADVERTENTLY BUMPED OPEN CAUSING A LOSS OF AIR TO SCRAM VALVES. THE REACTOR SCRAMMED ON THE SCRAM DISCHARGE VOLUME HIGH HIGH LEVEL THAT OCCURRED AS THE SCRAM VALVES DRIFTED OPEN.

SUMMARY

GRAND GULF 1 OPERATED WITH 1 OUTAGE AND SEVERAL REDUCTIONS FOR ECONOMIC AND SYSTEM LOAD DEMAND CONSIDERATIONS.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* GRAND GULF 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....MISSISSIPPI

COUNTY.....CLAIBORNE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI S OF
VICKSBURG, MISS

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...AUGUST 18, 1982

DATE ELEC ENER 1ST GENER...OCTOBER 20, 1984

DATE COMMERCIAL OPERATE....JULY 1, 1985

CONDENSER COOLING METHOD...CCHNDCT

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHWEST POWER POOL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....MISSISSIPPI POWER & LIGHT COMPANY

CORPORATE ADDRESS.....P.O. BOX 1640
JACKSON, MISSISSIPPI 39205

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....ALLIS-CHALMERS

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....R. BUTCHER

LICENSING PROJ MANAGER.....L. KINTNER
DOCKET NUMBER.....50-416

LICENSE & DATE ISSUANCE....NPF-29, NOVEMBER 1, 1984

PUBLIC DOCUMENT ROOM.....HINDS JUNIOR COLLEGE
MC LENDON LIBRARY
RAYMOND, MISSISSIPPI 39154

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MAY 12-16 AND 21 (86-12): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED IN THE AREA OF EMERGENCY PREPAREDNESS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MAY 20 - JUNE 12 (86-17): THIS ROUTINE INSPECTION WAS CONDUCTED IN THE AREAS OF LICENSEE ACTION ON PREVIOUS ENFORCEMENT MATTERS, OPERATIONAL SAFETY VERIFICATION, MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, ESF SYSTEM WALKDOWN, REPORTABLE OCCURRENCES, OPERATING REACTOR EVENTS, INSPECTOR FOLLOWUP AND UNRESOLVED ITEMS, AND IE BULLETIN, CONFIRMATORY ACTION LETTER AND GENERIC LETTER FOLLOWUP. ONE VIOLATION - FAILURE TO HAVE PROCEDURAL CONTROLS REQUIRING PERSONNEL REVISING PROCEDURES WHICH AFFECT NUCLEAR SAFETY TO MAKE A DETERMINATION THAT AN UNREVIEWED SAFETY QUESTION IS OR IS NOT INVOLVED.

INSPECTION JUNE 9-13 (86-18): THIS ROUTINE, ANNOUNCED INSPECTION WAS CONDUCTED AT THE CORPORATE OFFICE IN THE AREAS OF OFFSITE SUPPORT STAFF AND OFFSITE REVIEW COMMITTEE. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 13 - JULY 14 (86-20): THIS ROUTINE INSPECTION WAS CONDUCTED BY THE RESIDENT INSPECTORS AT THE SITE IN THE AREAS OF LICENSEE ACTION ON PREVIOUS ENFORCEMENT MATTERS, OPERATIONAL SAFETY VERIFICATION, MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, ESF SYSTEM WALKDOWN, REPORTABLE OCCURRENCES, INSPECTOR FOLLOWUP AND UNRESOLVED ITEMS, INFORMATION MEETINGS WITH LOCAL OFFICIALS, DESIGN CHANGES AND MODIFICATIONS, AND SPENT FUEL STORAGE RACKS. ONE VIOLATION WITH TWO EXAMPLES WERE IDENTIFIED: 1) INADEQUATE SURVEILLANCE PROCEDURE IN THAT ISOLATION VALVES WERE NOT REQUIRED TO BE RESTORED TO THE LOCKED OPEN POSITION; AND 2) FAILURE TO FOLLOW PROCEDURES WHEN INITIATING TEMPORARY ALTERATIONS.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ FIRST REFUELING OUTAGE TO BEGIN 09/05/86.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

OPERATING AT 100% POWER.

LAST IE SITE INSPECTION DATE: JUNE 13 - JULY 14, 1986 +

INSPECTION REPORT NO: 50-416/86-20 +

REPORTS FROM LICENSEE

GRAND GULF 1

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
85-043	11/07/85	12/09/85	WORK ON THE HYDROGEN ANALYZERS FOUND TERMINAL STRIPS IN ALL FOUR HYDROGEN ANALYZERS BRITTLE, CRACKED OR BROKEN; CAUSE - EXCESSIVE HEAT.
86-012	04/10/86	05/12/86	CONTAINMENT ISOLATION VALVE E12-F008 AUTOMATICALLY CLOSED DUE TO A FAILED COIL IN A LOGIC RELAY.
86-013	04/21/86	05/02/86	PROCEDURAL AND DESIGN DEFICIENCIES; CAUSE - UNMONITORED CONTROL ROOM AIR INFLUENT.
86-014	04/11/86	05/16/86	FAILURE TO PERFORM A NONROUTINE SURVEILLANCE RETEST FOR A MOTOR OPERATED ISOLATION VALVE'S THERMAL OVERLOAD BYPASS CIRCUITRY; DUE TO PROCEDURAL ERROR.
86-016	04/17/86	05/16/86	PROCEDURAL ERROR CAUSES MISSED ISOLATION TIME MEASUREMENT; CAUSE - REQUIREMENT TO MEASURE INADVERTENTLY DELETED FROM PROCEDURE DATA TABLE.
86-019	12/15/84	06/23/86	TWO UNDESIGNATED AND IMPROPERLY SEALED PENETRATIONS IN FIRE RATED WALL BETWEEN CONTROL BUILDING COMPUTER ROOM AND LOWER CABLE SPREADING ROOM; CAUSE - IMPROPER INSTALLATION DURING CONSTRUCTION PHASE.
86-020	06/03/86	07/03/86	NONQUALIFIED RELAY COULD CAUSE LOSS OF SGTS SAFETY FUNCTION.
86-021	06/07/86	07/07/86	UNPLANNED REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION OCCURRED DUE TO A FALSE HIGH STEAM FLOW TRIP SIGNAL.

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1. Docket: 50-213 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: J. P. DRAGO (203) 267-2556 X452

4. Licensed Thermal Power (MWt): 1825

5. Nameplate Rating (Gross MWe): 667 X 0.9 = 600

6. Design Electrical Rating (Net MWe): 582

7. Maximum Dependable Capacity (Gross MWe): 596

8. Maximum Dependable Capacity (Net MWe): 569

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	162,887.0
13. Hours Reactor Critical	249.3	1,633.0	138,032.4
14. Rx Reserve Shtdwn Hrs	.0	.0	1,221.5
15. Hrs Generator On-Line	246.3	1,418.7	132,219.4
16. Unit Reserve Shtdwn Hrs	.0	.0	398.0
17. Gross Therm Ener (MWH)	445,484	1,935,473	229,051,385
18. Gross Elec Ener (MWH)	137,608	900,261	75,430,699
19. Net Elec Ener (MWH)	127,327	542,287	71,443,378
20. Unit Service Factor	33.1	27.9	81.2
21. Unit Avail Factor	33.1	27.9	81.4
22. Unit Cap Factor (MDC Net)	30.1	18.7	80.4*
23. Unit Cap Factor (DER Net)	29.4	18.3	75.4*
24. Unit Forced Outage Rate	66.9	31.3	6.0
25. Forced Outage Hours	497.7	646.6	1,972.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

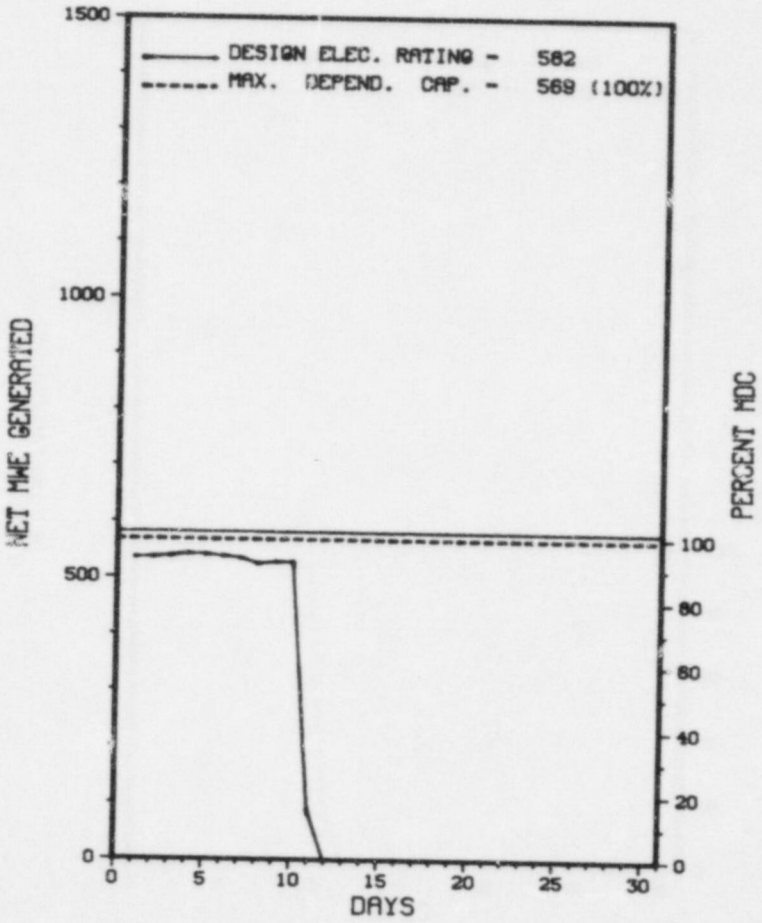
NONE

27. If Currently Shutdown Estimated Startup Date: 08/25/86

****
* HADDAM NECK *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

HADDAM NECK



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

HADDAM NECK

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-07	07/11/86	F	497.7	A	1	86-036	SF	PUMPXX	METAL FATIGUE OF CHARGING PUMP SHAFT. IMPROVED INSPECTION, PERIODIC MAINTENANCE, AND ENGINEERING EVALUATION OF PRESENT PUMP DESIGN. ALSO, PRIMARY WATER STRESS CORROSION CRACKING OF STEAM GENERATOR TUBES.

SUMMARY

CONNECTICUT YANKEE HADDAM NECK SHUTDOWN ON JULY 11TH FOR EQUIPMENT REPAIRS.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* HADDAM NECK *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....CONNECTICUT

COUNTY.....MIDDLESEX

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...13 MI E OF
MERIDEN, CONN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JULY 24, 1967

DATE ELEC ENER 1ST GENER...AUGUST 7, 1967

DATE COMMERCIAL OPERATE....JANUARY 1, 1968

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CONNECTICUT RIVER

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CONNECTICUT YANKEE ATOMIC POWER

CORPORATE ADDRESS.....P.O. BOX 270
HARTFORD, CONNECTICUT 06101

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....P. SWETLAND

LICENSING PROJ MANAGER.....F. AKSTULEWICZ
DOCKET NUMBER.....50-213

LICENSE & DATE ISSUANCE....DPR-61, DECEMBER 27, 1974

PUBLIC DOCUMENT ROOM.....RUSSELL LIBRARY
123 BROAD STREET
MIDDLETOWN, CONNECTITCUT 06457

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 4.4 STATES THAT A LLRT WILL BE PERFORMED BY A METHOD THAT IS EQUIVALENT SENSITIVE. THE LICENSEE IS USING A METHOD (WATER COLLECTION METHOD) WHICH HAS BEEN DISAPPROVED BY THE NRC. SA-V-413 IS CLASSIFIED AS A MANUAL CONTAINMENT ISOLATION VALVE WHICH DURING OPERATION OF THE PLANT SHOULD BE IN THE CLOSED POSITION AND LOCKED ACCORDING TO TECHNICAL SPECIFICATION 1.8. CONTRARY TO THE ABOVE THE LICENSEE UNLOCKED AND OPENED THE VALVE 3 DIFFERENT TIMES IN 1985 TO PERFORM SURVEILLANCE TESTING. THE LICENSEE HAD TO REDESIGNATE TWO VALVES AS CONTAINMENT ISOLATION VALVES BECAUSE OF THE FOLLOWING: (1) PENETRATION 80 - UNABLE TO PERFORM LLRT OF THE CHECK VALVE WHICH WAS DESIGNATED AS THE CIV BECAUSE OF THE CONFIGURATION (COULD NOT PRESSURIZE) COULD ONLY BE PERFORMED DURING ILRT. (2) PENETRATION 62 - DOUBLE CHECK VALVES ONE INSIDE CONTAINMENT AND ONE OUTSIDE SHOWED EXCESSIVE LEAKAGE SO A MOTOR OPERATED VALVE DOWNSTREAM WAS REDESIGNATED AS THE ISOLATION VALVE. THE LICENSEE FAILED TO PERFORM REQUIRED 50.59 SAFETY EVALUATION REPORT TO DEMONSTRATE THAT AN UNREVIEWED SAFETY ISSUE DOES NOT EXIST. (8600 4)

FAILURE TO FOLLOW PROCEDURE 1.2-3.1, DESIGN CHANGES SUCH THAT OPERATING PROCEDURES WERE NOT IMPLEMENTED UPON TURNOVER OF A MODIFIED STEAM GENERATOR CHEMICAL CONTROL SYSTEM.

ENFORCEMENT SUMMARY

(8600 5)

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-321 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: T. KILROY (912) 367-7781 X2882
4. Licensed Thermal Power (MWt): 2436
5. Nameplate Rating (Gross MWe): 1000 X 0.85 = 850
6. Design Electrical Rating (Net MWe): 777
7. Maximum Dependable Capacity (Gross MWe): 801
8. Maximum Dependable Capacity (Net MWe): 768
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	92,759.0
13. Hours Reactor Critical	744.0	2,055.1	64,107.1
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	744.0	1,770.4	60,336.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,782,168	3,952,824	128,385,079
18. Gross Elec Ener (MWH)	574,690	1,264,890	41,497,730
19. Net Elec Ener (MWH)	549,811	1,200,910	39,410,088
20. Unit Service Factor	100.0	34.8	65.0
21. Unit Avail Factor	100.0	34.8	65.0
22. Unit Cap Factor (MDC Net)	96.2	30.7	55.3
23. Unit Cap Factor (DER Net)	95.1	30.4	54.7
24. Unit Forced Outage Rate	.0	7.6	15.3
25. Forced Outage Hours	.0	144.8	10,664.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

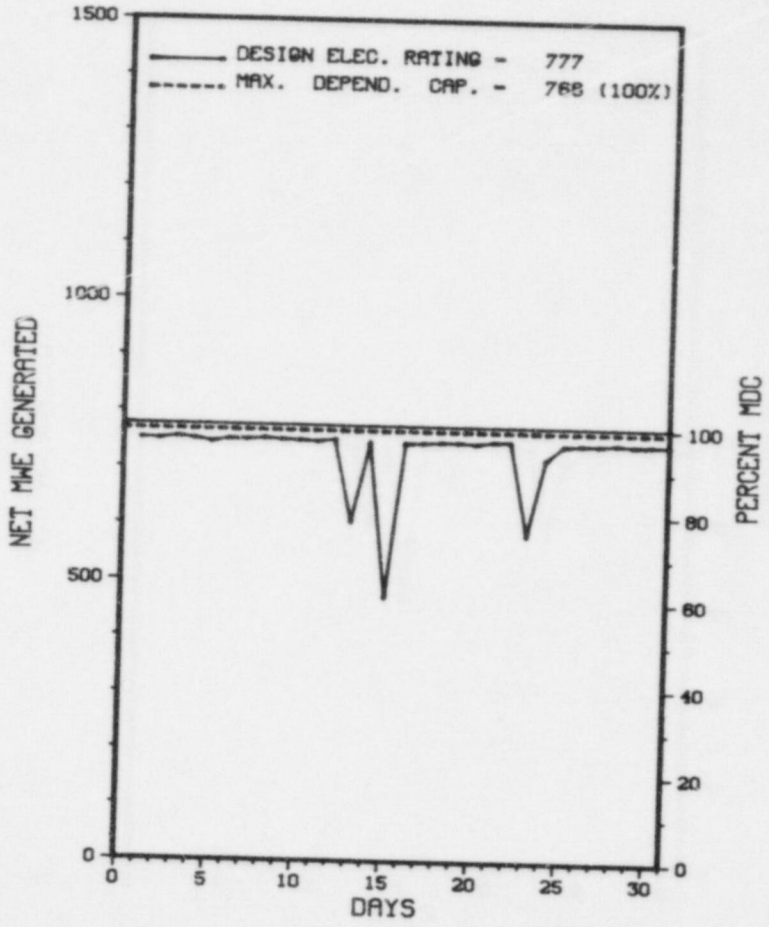
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* HATCH 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

HATCH 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

HATCH 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-023	07/02/86	F	0.0	A	5		CH	VALVEX	DECREASED RECIRC. FLOW DUE TO N21-F200A HYDRAULIC MALFUNCTION.
86-024	07/05/86	S	0.0	B	5		HA	TURBIN	WEEKLY TURBINE TESTING.
86-025	07/13/86	F	0.0	A	5		CB	PUMPXX	"1B" RECIRC. PUMP TRIP.
86-026	07/19/86	S	0.0	B	5		HA	TURBIN	WEEKLY TURBINE TESTING.
86-027	07/23/86	F	0.0	A	5		CH	TURBIN	"1A" REACTOR FEED PUMP TURBINE (RFPT) TRIP.
86-028	07/27/86	S	0.0	B	5		HA	TURBIN	WEEKLY TURBINE TESTING.

SUMMARY

HATCH 1 OPERATED WITH 6 REDUCTIONS LISTED IN DETAIL ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* HATCH 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....GEORGIA

COUNTY.....APPLING

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...11 MI N OF
BAXLEY, GA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...SEPTEMBER 12, 1974

DATE ELEC ENER 1ST GENER...NOVEMBER 11, 1974

DATE COMMERCIAL OPERATE....DECEMBER 31, 1975

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....ALTAMAHA RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....GEORGIA POWER

CORPORATE ADDRESS.....333 PIEDMONT AVENUE
ATLANTA, GEORGIA 30308

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....GEORGIA POWER CO.

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....P. HOLMES RAY

LICENSING PROJ MANAGER.....G. RIVENBARK
DOCKET NUMBER.....50-321

LICENSE & DATE ISSUANCE....DPR-57, OCTOBER 13, 1974

PUBLIC DOCUMENT ROOM.....APPLING COUNTY PUBLIC LIBRARY
301 CITY HALL DRIVE
BAXLEY, GEORGIA 31563

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MAY 31 - JUNE 30 (86-17): THIS ROUTINE INSPECTION WAS CONDUCTED AT THE SITE IN THE AREAS OF LICENSEE ACTION ON PREVIOUS ENFORCEMENT MATTERS, OPERATIONAL SAFETY VERIFICATION, MAINTENANCE OBSERVATION, PLANT MODIFICATION AND SURVEILLANCE OBSERVATION, AND REPORTABLE OCCURRENCES. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 23-27 (86-18): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED A REVIEW OF THE LICENSEE'S HEALTH PHYSICS PROGRAM, INCLUDING ORGANIZATION AND MANAGEMENT CONTROLS, INTERNAL EXPOSURE, EXTERNAL EXPOSURE CONTROL, CONTROL AND PERSONAL DOSIMETRY, THE LICENSEE'S PROGRAM TO MAINTAIN EXPOSURES AS LOW AS REASONABLY ACHIEVABLE (ALARA), AND THE CONTROL OF MATERIALS RELEASED FROM THE RCA. ONE VIOLATION - INADEQUATE RELEASE SURVEYS OF MATERIALS FOR UNRESTRICTED USE.

ENFORCEMENT SUMMARY

CONTRARY TO TS 3.5.B.1, ON APRIL 15, 1986 FROM APPROXIMATELY 1:00 A.M. UNTIL APPROXIMATELY 1:45 A.M., SHUTDOWN COOLING WAS NOT OPERABLE SINCE THE IN SERVICE RHR PUMP DISCHARGE VALVE (1E11-F015B) WAS, UNKNOWN TO THE REACTOR OPERATORS, SHUT DUE TO A BLOWN FUSE. THE FUSE WAS REPLACED ABOUT 1:45 A.M. THE RESET OF AND REOPENING OF VALVE 1E11-F015B AND THEREBY RESUMPTION OF SHUTDOWN COOLING DID NOT OCCUR UNTIL APPROXIMATELY 4:00 A.M. WHEN A PLANT EQUIPMENT OPERATOR CALLED THE CONTROL ROOM TO REPORT THAT THE RUNNING RHR PUMP DID NOT SOUND NORMAL. NO INCREASE IN REACTOR VESSEL TEMPERATURE WAS NOTED DURING THE THREE HOURS OF THIS OCCURRENCE. CONTRARY TO TS 4.7.4.B, ON FEBRUARY 26, 1986, HYDRAULIC SNUBBER 2E11-RHR-R90, WAS FOUND LEAKING AND NOT DECLARED

ENFORCEMENT SUMMARY

INOPERABLE AS REQUIRED BY TS 4.7.4.B, AMENDMENT NO. 51. THIS CONDITION WAS ATTRIBUTED TO AN INADEQUATE SNUBBER, INSPECTION AND FUNCTIONAL TEST PROCEDURE 52 SV-SUV-001-2 WHICH THE LICENSEE FAILED TO REVISE FOLLOWING IMPLEMENTATION OF TS AMENDMENT NO. 51. TS AMENDMENT NO. 51 WAS IMPLEMENTED BY THE LICENSEE ON SEPTEMBER 4, 1985, AND CHANGED THE SNUBBER VISUAL INSPECTION CRITERIA. PREVIOUS LICENSEE CORRECTIVE ACTIONS DISCUSSED IN LICENSEE CORRECTIVE ACTIONS DISCUSSED IN LICENSEE EVENT REPORT NUMBERS 50-321/85-28 AND 50-366/85-28 FAILED TO PREVENT THIS TYPE OF RECURRENCE.
(8601 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

+ NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 23-27, 1986 +

INSPECTION REPORT NO: 50-321/86-18 +

REPORTS FROM LICENSEE

HATCH 1

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-017	04/15/86		PERSONNEL ERROR CAUSES LOSS OF SHUTDOWN COOLING.
86-018	05/21/86	06/20/86	HIGH TURBINE BUILDING TEMPERATURE CAUSES ESF ACTUATIONS; CAUSE - STEAM LEAK AROUND PACKING OF MAIN STEAM LINE DRAIN VALVE.
86-019	04/13/86	05/23/86	DEFECTIVE PROCEDURE COULD PREVENT DIESEL GENERATOR FROM BEING OPERABLE.
86-020	04/25/86	05/23/86	REACTOR WATER CLEANUP SYSTEM'S INBOARD AND OUTBOARD PRIMARY CONTAINMENT ISOLATION VALVES ISOLATED ON A "HIGH LEAKAGE FLOW" SIGNAL; CAUSE - JUMPER DISCONNECTED.
86-025	05/22/86	06/20/86	PERSONNEL ERROR DURING TROUBLESHOOTING, BLOWS FUSE RESULTING IN MULTIPLE ESF ACTUATIONS.
86-026	06/05/86	07/07/86	DEFECTIVE PROCEDURE CAUSES NON-COMPLIANCE WITH TECHNICAL SPECIFICATION; CAUSE - DELETED THE STEP REQUIRING PERTURBATION OF THE REACTOR WATER LEVEL.
86-027	06/12/86	07/14/86	TWO DRYWELL VENT VALVES FAILED TO PRESSURIZE DURING PERFORMANCE OF SPECIAL PURPOSE LLRT PROCEDURE 42SP-061286-0U-1S, 'T' RINGS ON BOTH VALVES REPLACED AND DISCS WERE ADJUSTED.

ENFORCEMENT SUMMARY

INOPERABLE AS REQUIRED BY TS 4.7.4.B, AMENDMENT NO. 51. THIS CONDITION WAS ATTRIBUTED TO AN INADEQUATE SNUBBER, INSPECTION AND FUNCTIONAL TEST PROCEDURE 52 SV-SUV-001-2 WHICH THE LICENSEE FAILED TO REVISE FOLLOWING IMPLEMENTATION OF TS AMENDMENT NO. 51. TS AMENDMENT NO. 51 WAS IMPLEMENTED BY THE LICENSEE ON SEPTEMBER 4, 1985, AND CHANGED THE SNUBBER VISUAL INSPECTION CRITERIA. PREVIOUS LICENSEE CORRECTIVE ACTIONS DISCUSSED IN LICENSEE CORRECTIVE ACTIONS DISCUSSED IN LICENSEE EVENT REPORT NUMBERS 50-321/85-28 AND 50-366/85-28 FAILED TO PREVENT THIS TYPE OF RECURRENCE.
(8601 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

+ NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 23-27, 1986 +

INSPECTION REPORT NO: 50-321/86-18 +

REPORTS FROM LICENSEE

HATCH 1

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-017	04/15/86		PERSONNEL ERROR CAUSES LOSS OF SHUTDOWN COOLING.
86-018	05/21/86	06/20/86	HIGH TURBINE BUILDING TEMPERATURE CAUSES ESF ACTUATIONS; CAUSE - STEAM LEAK AROUND PACKING OF MAIN STEAM LINE DRAIN VALVE.
86-019	04/13/86	05/23/86	DEFECTIVE PROCEDURE COULD PREVENT DIESEL GENERATOR FROM BEING OPERABLE.
86-020	04/25/86	05/23/86	REACTOR WATER CLEANUP SYSTEM'S INBOARD AND OUTBOARD PRIMARY CONTAINMENT ISOLATION VALVES ISOLATED ON A "HIGH LEAKAGE FLOW" SIGNAL; CAUSE - JUMPER DISCONNECTED.
86-025	05/22/86	06/20/86	PERSONNEL ERROR DURING TROUBLESHOOTING, BLOWS FUSE RESULTING IN MULTIPLE ESF ACTUATIONS.
86-026	06/05/86	07/07/86	DEFECTIVE PROCEDURE CAUSES NON-COMPLIANCE WITH TECHNICAL SPECIFICATION; CAUSE - DELETED THE STEP REQUIRING PERTURBATION OF THE REACTOR WATER LEVEL.
86-027	06/12/86	07/14/86	TWO DRYWELL VENT VALVES FAILED TO PRESSURIZE DURING PERFORMANCE OF SPECIAL PURPOSE LLRT PROCEDURE 42SP-061286-OU-1S, 'T' RINGS ON BOTH VALVES REPLACED AND DISCS WERE ADJUSTED.

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1. Docket: 50-366 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: T. KILROY (912) 367-7781 X2882
4. Licensed Thermal Power (MWt): 2436
5. Nameplate Rating (Gross MWe): 1000 X 0.85 = 850
6. Design Electrical Rating (Net MWe): 784
7. Maximum Dependable Capacity (Gross MWe): 804
8. Maximum Dependable Capacity (Net MWe): 777
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	60,528.0
13. Hours Reactor Critical	672.3	4,799.9	42,520.6
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	651.0	4,703.4	40,714.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,142,516	9,430,716	87,963,963
18. Gross Elec Ener (MWH)	353,870	3,046,180	28,961,170
19. Net Elec Ener (MWH)	334,433	2,897,218	27,567,411
20. Unit Service Factor	87.5	92.5	67.3
21. Unit Avail Factor	87.5	92.5	67.3
22. Unit Cap Factor (MDC Net)	57.9	73.3	58.6
23. Unit Cap Factor (DER Net)	57.3	72.6	58.1
24. Unit Forced Outage Rate	.0	4.6	9.2
25. Forced Outage Hours	.0	228.3	4,114.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

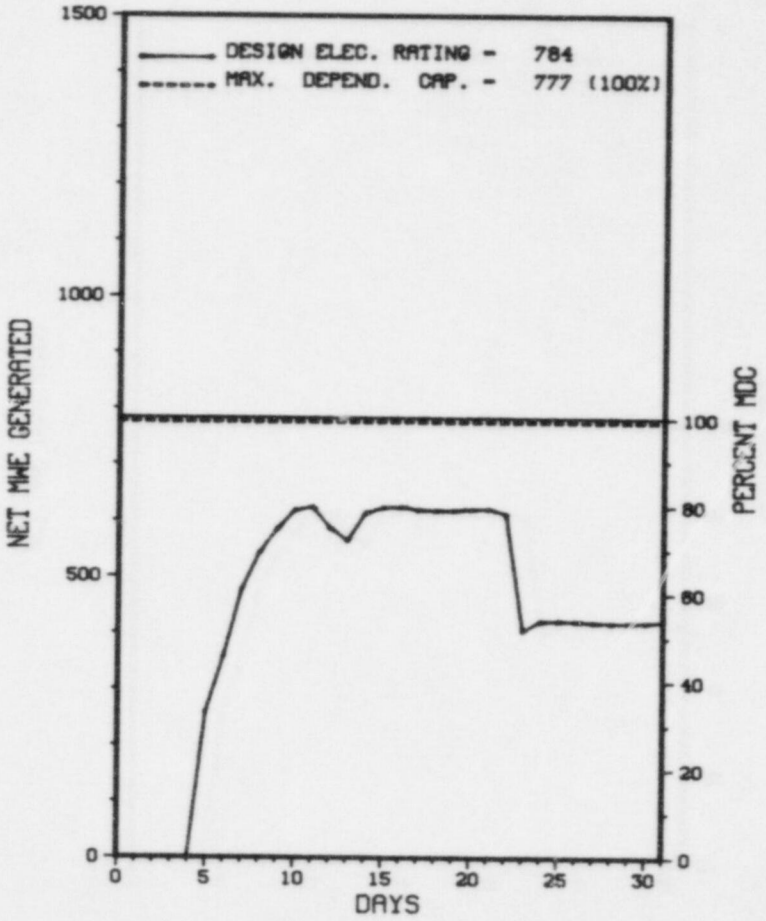
REFUELING OUTAGE: SEPTEMBER 8, 1986 - 8 WEEKS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* HATCH 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

HATCH 2



JULY 1986

Report Period JUL 1986 U N I T S H U T D O W N S / R E D U C T I O N S

****
* HATCH 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-045	06/23/86	S	93.0	A	4	028612	CH	XXXXXX	SHUTDOWN TO REPAIR CONDENSER BAY STEAM LEAKS CONTINUES.
86-046	07/04/86	S	0.0	A	5		CH	XXXXX	RAMPING TO 85% POWER FOLLOWING REPAIRS.
86-047	07/10/86	S	0.0	H	5		ZZ	ZZZZZZ	MANAGEMENT DECISION TO HOLD AT 85% POWER DUE TO HIGH OFFGAS ACTIVITY.
86-048	07/12/86	F	0.0	A	5		ZZ	ZZZZZ	RECIRC. FLOW DECREASED MANUALLY AFTER PROCESS COMPUTER PROBLEMS.
86-049	07/13/86	S	0.0	B	5		HA	VALVEX	REDUCED LOAD FOR STOP VALVE TESTING.
86-050	07/14/86	S	0.0	H	5		ZZ	ZZZZZZ	MANAGEMENT DECISION TO HOLD AT 85% POWER DUE TO HIGH OFFGAS ACTIVITY.
86-051	07/18/86	F	0.0	A	5		HH	PUMPXX	"2B" CONDENSATE PUMP TRIP.
86-052	07/19/86	S	0.0	H	5		ZZ	ZZZZZZ	MANAGEMENT DECISION TO HOLD AT 85% POWER DUE TO HIGH OFFGAS ACTIVITY.
86-053	07/22/86	F	0.0	A	5		CB	PUMPXX	"2B" CIRCULATING WATER PUMP BEARING FAILURE.

****
* SUMMARY *
****

HATCH 2 RETURNED ONLINE JULY 5TH FROM AN EQUIPMENT REPAIR OUTAGE AND OPERATED WITH 8 REDUCTIONS LISTED IN DETAIL ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* HATCH 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....GEORGIA

COUNTY.....APPLING

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...11 MI N OF
BAXLEY, GA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JULY 4, 1978

DATE ELEC ENER 1ST GENER...SEPTEMBER 22, 1978

DATE COMMERCIAL OPERATE....SEPTEMBER 5, 1979

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....ALTAMAHA RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....GEORGIA POWER

CORPORATE ADDRESS.....333 PIEDMONT AVENUE
ATLANTA, GEORGIA 30308

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....GEORGIA POWER CO.

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....P. HOLMES RAY

LICENSING PROJ MANAGER.....G. RIVENBARK
DOCKET NUMBER.....50-366

LICENSE & DATE ISSUANCE....NPF-5, JUNE 13, 1978

PUBLIC DOCUMENT ROOM.....APPLING COUNTY PUBLIC LIBRARY
301 CITY HALL DRIVE
BAXLEY, GEORGIA 31563

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MAY 31 - JUNE 30 (86-17): THIS ROUTINE INSPECTION WAS CONDUCTED AT THE SITE IN THE AREAS OF LICENSEE ACTION ON PREVIOUS ENFORCEMENT MATTERS, OPERATIONAL SAFETY VERIFICATION, MAINTENANCE OBSERVATION, PLANT MODIFICATION AND SURVEILLANCE OBSERVATION, AND REPORTABLE OCCURRENCES. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 23-27 (86-18): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED A REVIEW OF THE LICENSEE'S HEALTH PHYSICS PROGRAM, INCLUDING ORGANIZATION AND MANAGEMENT CONTROLS, INTERNAL EXPOSURE, EXTERNAL EXPOSURE CONTROL, CONTROL AND PERSONAL DOSIMETRY, THE LICENSEE'S PROGRAM TO MAINTAIN EXPOSURES AS LOW AS REASONABLY ACHIEVABLE (ALARA), AND THE CONTROL OF MATERIALS RELEASED FROM THE RCA. ONE VIOLATION - INADEQUATE RELEASE SURVEYS OF MATERIALS FOR UNRESTRICTED USE.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

+ NONE.

PLANT STATUS:

+ UNIT IS OPERATING AT REDUCED POWER.

LAST IE SITE INSPECTION DATE: JUNE 23-27, 1986 +

INSPECTION REPORT NO: 50-366/86-18 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-011	04/25/86	05/23/86	COMPONENT FAILURES PREVENT 4 HOUR GRAB SAMPLING; DUE TO A BLOWN POWER FUSE TO THE VACUUM PUMP DRIVE MOTOR.

1. Docket: 50-247 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: K. KRIEGER (914) 526-5155
4. Licensed Thermal Power (MWt): 2758
5. Nameplate Rating (Gross MWe): 1126 X 0.9 = 1013
6. Design Electrical Rating (Net MWe): 873
7. Maximum Dependable Capacity (Gross MWe): 885
8. Maximum Dependable Capacity (Net MWe): 849
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	105,936.0
13. Hours Reactor Critical	726.2	1,690.0	70,860.1
14. Rx Reserve Shtdwn Hrs	17.7	214.3	2,711.9
15. Hrs Generator On-Line	695.6	1,552.6	68,689.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,894,314	3,907,943	178,742,150
18. Gross Elec Ener (MWH)	589,870	1,208,970	55,459,446
19. Net Elec Ener (MWH)	566,135	1,125,016	52,303,874
20. Unit Service Factor	93.5	30.5	64.8
21. Unit Avail Factor	93.5	30.5	64.8
22. Unit Cap Factor (MDC Net)	89.6	25.9	58.2*
23. Unit Cap Factor (DER Net)	87.2	25.3	56.6
24. Unit Forced Outage Rate	6.5	18.9	9.4
25. Forced Outage Hours	48.4	361.6	6,835.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

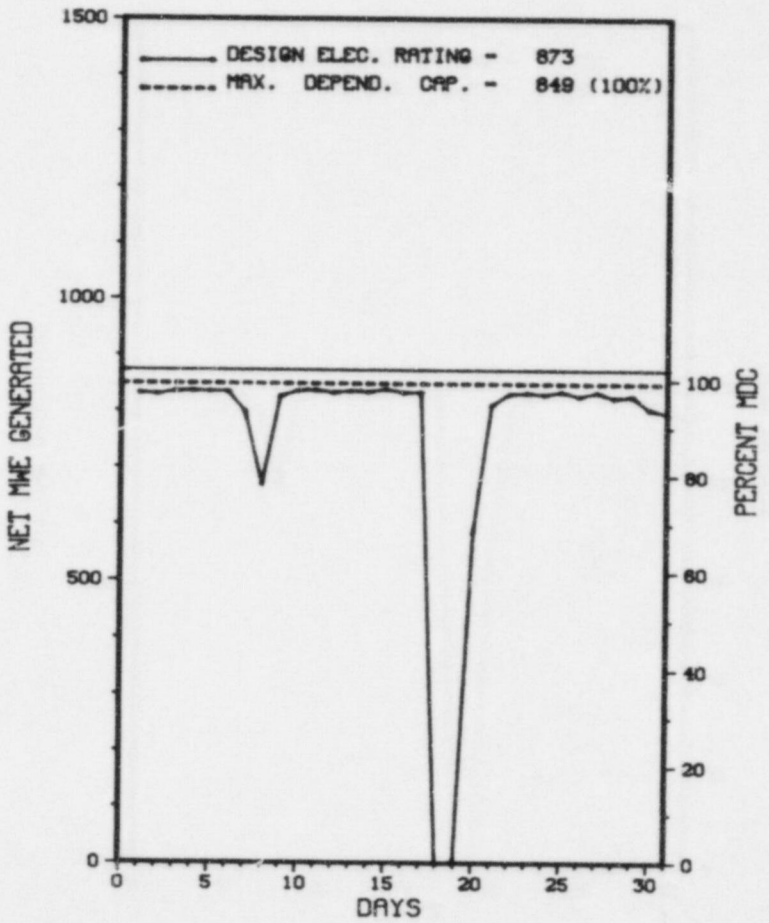
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* INDIAN POINT 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

INDIAN POINT 2



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986 U N I T S H U T D O W N S / R E D U C T I O N S

****
* INDIAN POINT 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
7	07/07/86	F	0.0		5		CH	PUMPXX	22 MBFP REPAIR.
8	07/18/86	F	48.4	G	3	86-024	EG	GENERA	MG SET OPERATOR ERROR/DISCIPLINARY ACTION TAKEN.

****
* SUMMARY *
****

INDIAN POINT 2 - OPERATED WITH 1 OUTAGE AND 1 REDUCTION DURING JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* INDIAN POINT 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....NEW YORK

COUNTY.....WESTCHESTER

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI N OF
NEW YORK CITY, NY

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 22, 1973

DATE ELEC ENER 1ST GENER...JUNE 26, 1973

DATE COMMERCIAL OPERATE....AUGUST 1, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....HUDSON RIVER

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CONSOLIDATED EDISON

CORPORATE ADDRESS.....4 IRVING PLACE
NEW YORK, NEW YORK 10003

CONTRACTOR
ARCHITECT/ENGINEER.....UNITED ENG. & CONSTRUCTORS

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....WESTINGHOUSE DEVELOPMENT CORP

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....L. ROSSBACH

LICENSING PROJ MANAGER.....M. SLOSSON
DOCKET NUMBER.....50-247

LICENSE & DATE ISSUANCE....DPR-26, SEPTEMBER 28, 1973

PUBLIC DOCUMENT ROOM.....WHITE PLAINS PUBLIC LIBRARY
100 MARTINE AVENUE
WHITE PLAINS, NEW YORK 10601

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 6.9.1.3 REQUIRES THAT THE ANNUAL RADIATION EXPOSURE REPORT BE SUBMITTED NO LATER THAN MARCH 1 OF EACH YEAR. TECHNICAL SPECIFICATION 3.13.A.3 REQUIRES THAT A TELEPHONE NOTIFICATION BE MADE TO REGION I WITHIN 24 HOURS OF IDENTIFYING THAT THE FIRE PROTECTION SYSTEM IS INOPERABLE. CONTRARY TO THE ABOVE, IN APRIL 1986, THE INSPECTOR IDENTIFIED THE FOLLOWING: (A) THE ANNUAL RADIATION EXPOSURE REPORT WAS SUBMITTED ON MARCH 31, 1986. (B) THE FIRE PROTECTION SYSTEM BECAME INOPERABLE ON MARCH 28, 1986, BUT THE TELEPHONE NOTIFICATION TO REGION I WAS NOT MADE UNTIL MARCH 31, 1986.
(8601 5)

OTHER ITEMS

SYSTEMS AND COMPONENTS:

OTHER ITEMS

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-286 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: L. KELLY (914) 739-8200
4. Licensed Thermal Power (MWt): 3025
5. Nameplate Rating (Gross MWe): 1126 X 0.9 = 1013
6. Design Electrical Rating (Net MWe): 965
7. Maximum Dependable Capacity (Gross MWe): 1000
8. Maximum Dependable Capacity (Net MWe): 965
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe): 515
11. Reasons for Restrictions, If Any:

#32 MAIN TRANSFORMER IS OUT OF SERVICE.

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	86,952.0
13. Hours Reactor Critical	135.3	3,788.5	51,055.7
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	110.0	3,696.4	49,330.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	181,463	9,823,102	128,545,066
18. Gross Elec Ener (MWH)	52,400	3,212,310	40,816,696
19. Net Elec Ener (MWH)	48,957	3,091,473	39,105,864
20. Unit Service Factor	14.8	72.7	56.7
21. Unit Avail Factor	14.8	72.7	56.7
22. Unit Cap Factor (MDC Net)	6.8	63.0	46.6
23. Unit Cap Factor (DER Net)	6.8	63.0	46.6
24. Unit Forced Outage Rate	85.2	18.0	19.6
25. Forced Outage Hours	634.0	812.5	12,005.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

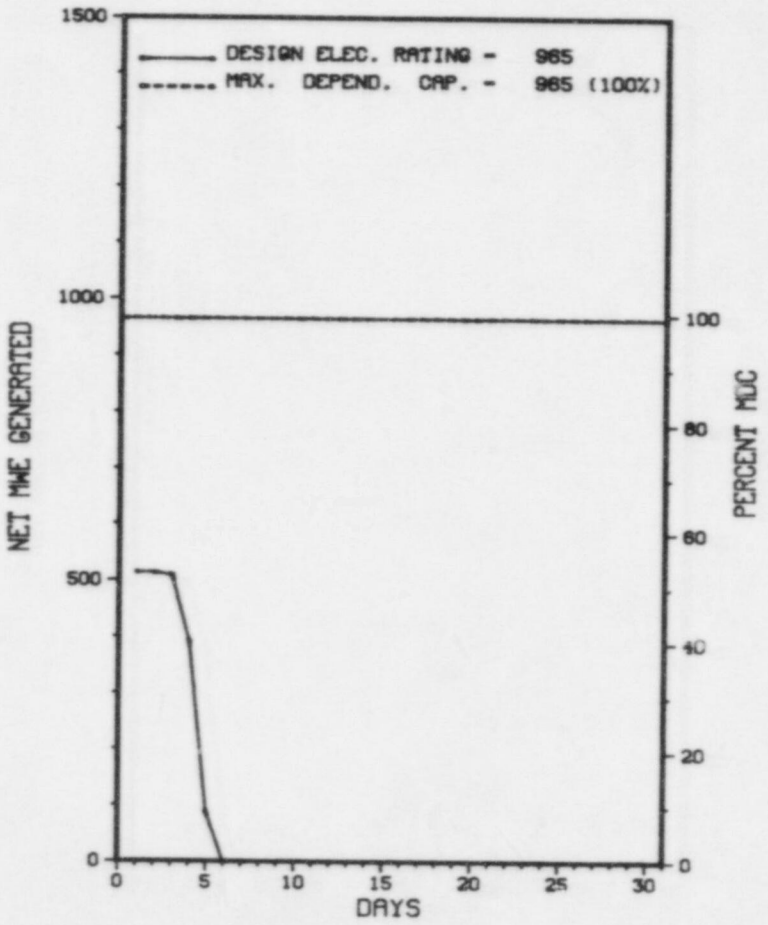
NONE

27. If Currently Shutdown Estimated Startup Date: 09/15/86

****
* INDIAN POINT 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

INDIAN POINT 3



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* INDIAN POINT 3 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
09	07/05/86	F	634.0	A	1		HC	XXXXXX	THE PLANT WAS REMOVED FROM SERVICE BECAUSE OF DECREASING CONDENSER VACUUM. THE DECREASING CONDENSER VACUUM WAS CAUSED BY A LEAKING FLEXIBLE EXPANSION JOINT BETWEEN LOW PRESSURE TURBINE 33 AND ITS ASSOCIATED MAIN CONDENSER 31.

****
* SUMMARY *
****

INDIAN POINT 3 SHUTDOWN ON JULY 5TH FOR EQUIPMENT REPAIRS.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* INDIAN POINT 3 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....NEW YORK

COUNTY.....WESTCHESTER

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI N OF
NEW YORK CITY, NY

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...APRIL 6, 1976

DATE ELEC ENER 1ST GENER...APRIL 27, 1976

DATE COMMERCIAL OPERATE....AUGUST 30, 1976

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....HUDSON RIVER

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NEW YORK POWER AUTHORITY

CORPORATE ADDRESS.....10 COLUMBUS CIRCLE
NEW YORK, NEW YORK 10019

CONTRACTOR
ARCHITECT/ENGINEER.....UNITED ENG. & CONSTRUCTORS

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....WESTINGHOUSE DEVELOPMENT CORP

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....P. KOLTAY

LICENSING PROJ MANAGER.....D. NEIGHBORS
DOCKET NUMBER.....50-286

LICENSE & DATE ISSUANCE....DPR-64, APRIL 5, 1976

PUBLIC DOCUMENT ROOM.....WHITE PLAINS PUBLIC LIBRARY
100 MARTINE AVENUE
WHITE PLAINS, NEW YORK 10601

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-305 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: G.RUITER (414) 388-2560 X207
4. Licensed Thermal Power (MWt): 1650
5. Nameplate Rating (Gross MWe): 622 X 0.9 = 560
6. Design Electrical Rating (Net MWe): 535
7. Maximum Dependable Capacity (Gross MWe): 529
8. Maximum Dependable Capacity (Net MWe): 503
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	106,296.0
13. Hours Reactor Critical	744.0	3,911.3	89,928.4
14. Rx Reserve Shtdwn Hrs	.0	.0	2,330.5
15. Hrs Generator On-Line	744.0	3,842.2	88,397.8
16. Unit Reserve Shtdwn Hrs	.0	.0	10.0
17. Gross Therm Ener (MWH)	1,218,187	6,139,763	138,847,496
18. Gross Elec Ener (MWH)	405,600	2,056,100	45,794,600
19. Net Elec Ener (MWH)	386,122	1,960,101	43,601,313
20. Unit Service Factor	100.0	75.5	83.2
21. Unit Avail Factor	100.0	75.5	83.2
22. Unit Cap Factor (MDC Net)	103.2	76.6	79.3*
23. Unit Cap Factor (DER Net)	97.0	72.0	76.7
24. Unit Forced Outage Rate	.0	.7	3.2
25. Forced Outage Hours	.0	28.5	2,819.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

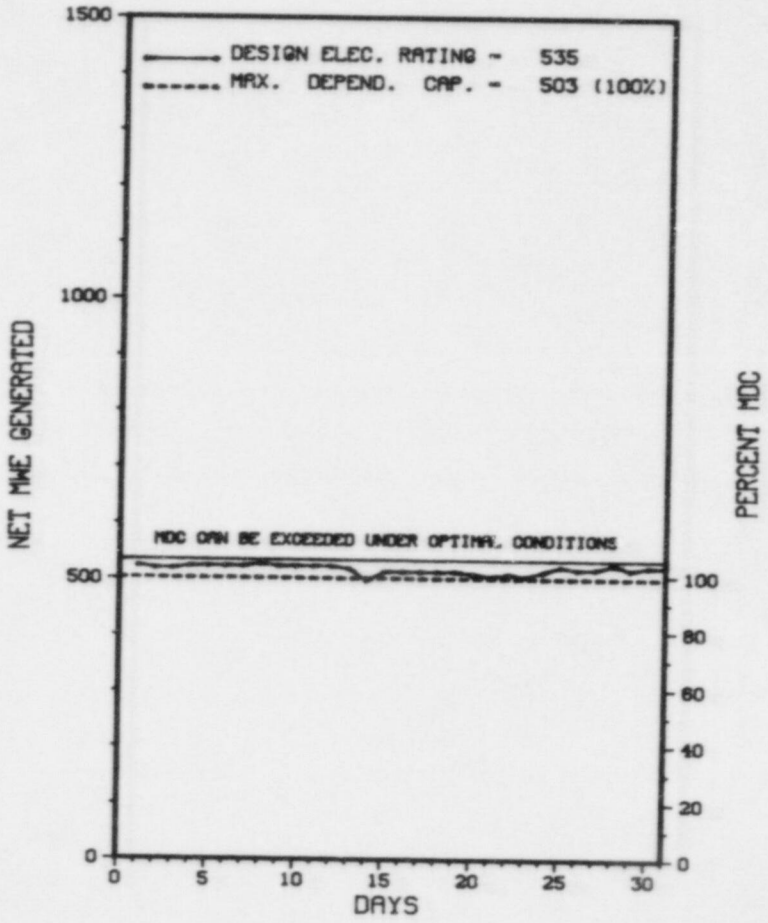
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* KEWAUNEE *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

KEWAUNEE



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

KEWAUNEE

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

KEWAUNEE OPERATED AT FULL POWER DURING THE JULY REPORT PERIOD.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* KEWAUNEE *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....WISCONSIN

COUNTY.....KEWAUNEE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...27 MI E OF
GREEN BAY, WI.

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MARCH 7, 1974

DATE ELEC ENER 1ST GENER...APRIL 3, 1974

DATE COMMERCIAL OPERATE....JUNE 16, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....WISCONSIN PUBLIC SERVICE

CORPORATE ADDRESS.....P.O. BOX 19002
GREEN BAY, WISCONSIN 54307

CONTRACTOR
ARCHITECT/ENGINEER.....PIONEER SERVICES & ENGINEERING

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....PIONEER SERVICES & ENGINEERING

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....R. NELSON

LICENSING PROJ MANAGER.....M. FAIRTILE
DOCKET NUMBER.....50-305

LICENSE & DATE ISSUANCE....DPR-43, DECEMBER 21, 1973

PUBLIC DOCUMENT ROOM.....UNIVERSITY OF WISCONSIN
LIBRARY LEARNING CENTER
2420 NICOLET DRIVE
GREEN BAY, WISCONSIN 54301

INSPECTION STATUS

INSPECTION SUMMARY

NONE

ENFORCEMENT SUMMARY

10 CFR, PART 50, APPENDIX B, CRITERION V, "INSTRUCTIONS, PROCEDURES, AND DRAWINGS" STATES, IN PART; "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, OR DRAWINGS, OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES, OR DRAWINGS." THIS CRITERION IS IMPLEMENTED BY THE LICENSEE'S OPERATIONAL QUALITY ASSURANCE PROGRAM, SECTION 12, "PLANT PROCEDURES." CONTRARY TO THE ABOVE, ACTIVITIES WERE NOT ACCOMPLISHED IN ACCORDANCE WITH APPROVED SURVEILLANCE PROCEDURES (SP) ON TWO IDENTIFIED OCCASIONS: ON APRIL 24, 1986, WITH THE PLANT AT 63% POWER, WHILE PERFORMING SP 06-033, "STEAM GENERATOR FLOW MISMATCH INSTRUMENT CHANNEL TEST," THE CONTROL ROOM OPERATOR DID NOT PLACE THE STEAM GENERATOR LEVEL LOOP B CONTROLLER IN THE MANUAL MODE PRIOR TO SHIFTING CONTROLLING CHANNELS, AS REQUIRED BY PROCEDURE STEP 6.7.A. A PLANT TRIP WAS EXPERIENCED DURING THE PERFORMANCE OF THIS PROCEDURE. ON MAY 13, 1986, WITH THE PLANT AT 100% POWER, WHILE PERFORMING SP 48-003D, "NUCLEAR POWER RANGE N-44 INSTRUMENT CHANNEL TEST", A TEST SIGNAL WAS MISTAKENLY APPLIED TO NUCLEAR POWER RANGE CHANNEL N-43, CAUSING A PLANT TRIP. CRITERION V, APPENDIX B, 10 CFR 50 REQUIRES ACTIVITIES AFFECTING QUALITY TO BE PRESCRIBED BY AND ACCOMPLISHED IN ACCORDANCE WITH PROCEDURES OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES. THE LICENSEE'S QUALITY ASSURANCE PROGRAM SECTION V.B.2 REQUIRES THAT APPLICABLE DEPARTMENTS SHALL BE RESPONSIBLE FOR THE

ENFORCEMENT SUMMARY

PREPARATION, REVIEW AND IMPLEMENTATION OF ALL INSTRUCTIONS AND PROCEDURES AFFECTING QUALITY. CONTRARY TO THE ABOVE, AS OF MARCH 14, 19 86, QUALITY-AFFECTING ACTIVITIES, RELATING TO THE SURVEYS OF SAFETY RELATED MASONRY WALLS THAT WERE CONDUCTED IN RESPONSE TO IE BULLETIN 80-11, WERE NOT PERFORMED UTILIZING CONTROLLED PROCEDURES AND INSTRUCTIONS. (8600 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

UNIT OPERATING NORMALLY

LAST IE SITE INSPECTION DATE: 08/07/86

INSPECTION REPORT NO: 86006

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-409 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: LOREE MALIN (608) 689-2331
4. Licensed Thermal Power (MWt): 165
5. Nameplate Rating (Gross MWe): 76.8 X 0.85 = 65
6. Design Electrical Rating (Net MWe): 50
7. Maximum Dependable Capacity (Gross MWe): 50
8. Maximum Dependable Capacity (Net MWe): 48
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

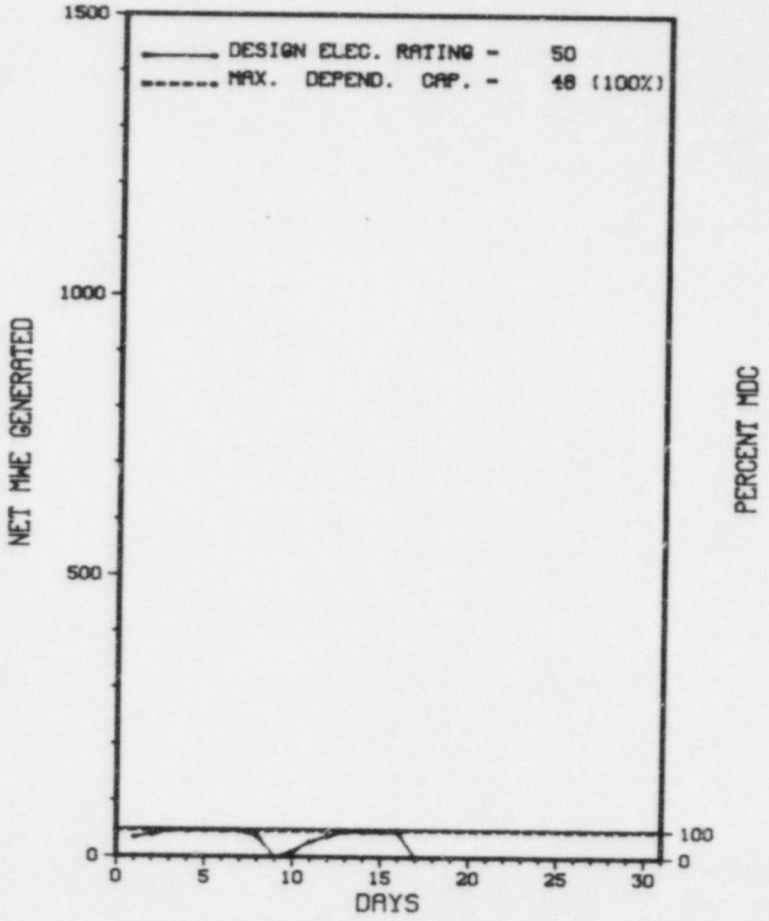
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	146,810.0
13. Hours Reactor Critical	358.8	2,866.1	98,804.7
14. Rx Reserve Shtdwn Hrs	.0	.0	478.0
15. Hrs Generator On-Line	348.3	2,591.2	92,092.4
16. Unit Reserve Shtdwn Hrs	.0	.0	79.0
17. Gross Therm Ener (MWH)	51,138	358,594	12,817,541
18. Gross Elec Ener (MWH)	15,543	109,788	3,848,997
19. Net Elec Ener (MWH)	14,266	100,926	3,569,674
20. Unit Service Factor	46.8	50.9	62.7
21. Unit Avail Factor	46.8	50.9	62.8
22. Unit Cap Factor (MDC Net)	39.9	41.3	50.7
23. Unit Cap Factor (DER Net)	38.3	39.7	48.6
24. Unit Forced Outage Rate	53.2	24.3	10.2
25. Forced Outage Hours	395.7	832.5	9,428.2

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: 09/01/86

****
* LA CROSSE *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

LA CROSSE



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

LA CROSSE

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-20	07/08/86	F	35.2	A	3	86-20	RB	CRDRVE	REACTOR AUTOMATICALLY SHUT DOWN WHEN A SOLENOID ON CONTROL ROD DRIVE MECHANISM (CRDM) NO. 12 FAILED. OPENING OF THE SOLENOID RESULTED IN LOW GAS PRESSURE FOR THAT CRDM, WHICH IS A SCRAM SIGNAL.
86-21	07/16/86	F	360.5	A	3	86-21	ED	GENERA	REACTOR AUTOMATICALLY SHUT DOWN WHEN THE 1A STATIC INVERTER TRANSFERRED TO ITS ALTERNATE SOURCE, MOMENTARILY DE-ENERGIZING A SCRAM TRAIN. EXTENSIVE TROUBLESHOOTING HAS BEEN CONDUCTED ON THE CAUSE OF THE INVERTER TRANSFER.

SUMMARY

LA CROSSE OPERATED WITH 2 OUTAGES DURING JULY, SHUTTING DOWN ON THE 16TH FOR EQUIPMENT REPAIRS.

Type

F-Forced
S-Sched

Reason

A-Equip Failure
B-Maint or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Admin
G-Oper Error
H-Other

Method

1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other

System & Component

Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* LA CROSSE *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....WISCONSIN

COUNTY.....VERNON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...19 MI S OF
LACROSSE, WISC

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JULY 11, 1967

DATE ELEC ENER 1ST GENER...APRIL 26, 1968

DATE COMMERCIAL OPERATE....NOVEMBER 1, 1969

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DAIRYLAND POWER

CORPORATE ADDRESS.....2615 EAST AVENUE SOUTH
LACROSSE, WISCONSIN 54601

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...ALLIS-CHALMERS

CONSTRUCTOR.....MAXON CONSTRUCTION COMPANY

TURBINE SUPPLIER.....ALLIS-CHALMERS

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....I. VILLALVA

LICENSING PROJ MANAGER.....R. BEVAN
DOCKET NUMBER.....50-409

LICENSE & DATE ISSUANCE....DPR-45, AUGUST 28, 1973

PUBLIC DOCUMENT ROOM.....LA CROSSE PUBLIC LIBRARY
800 MAIN STREET
LA CROSSE, WISCONSIN 54601

INSPECTION STATUS

INSPECTION SUMMARY

NONE

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

OTHER ITEMS

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

PLANT SHUTDOWN ON 07/16/86, DISCOVERED THRU-WALL LEAKS ON DECAY HEAT REMOVAL PIPING. PIPE REPLACEMENT IN PROGRESS. START-UP EXPECTED DURING LAST WEEK OF AUGUST 1986.

LAST IE SITE INSPECTION DATE: 08/30/86

INSPECTION REPORT NO: 86010

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-18	06/22/86	07/18/86	REACTOR SCRAM DUE TO RELAY MALFUNCTION/ LINE SEPARATION AT REACTOR WATER LEVEL TRANSMITTER
86-19	06/27/86	07/21/86	REACTOR SCRAM ON HIGH POWER/FLOW DUE TO REACTOR FEED PUMP MALFUNCTION

1. Docket: 50-373 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: JAMES P. PETERS (815) 357-6761

4. Licensed Thermal Power (MWt): 3323

5. Nameplate Rating (Gross MWe): 1078

6. Design Electrical Rating (Net MWe): 1078

7. Maximum Dependable Capacity (Gross MWe): 1078

8. Maximum Dependable Capacity (Net MWe): 1036

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	22,631.0
13. Hours Reactor Critical	.0	.0	12,037.5
14. Rx Reserve Shtdwn Hrs	.0	.0	1,640.9
15. Hrs Generator On-Line	.0	.0	11,639.9
16. Unit Reserve Shtdwn Hrs	.0	.0	1.0
17. Gross Therm Ener (MWH)	0	0	38,349,668
18. Gross Elec Ener (MWH)	0	0	10,499,394
19. Net Elec Ener (MWH)	-11,191	-69,983	9,945,621
20. Unit Service Factor	.0	.0	51.4
21. Unit Avail Factor	.0	.0	51.4
22. Unit Cap Factor (MDC Net)	.0	.0	42.4
23. Unit Cap Factor (DER Net)	.0	.0	40.8
24. Unit Forced Outage Rate	.0	.0	17.4
25. Forced Outage Hours	.0	.0	2,458.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

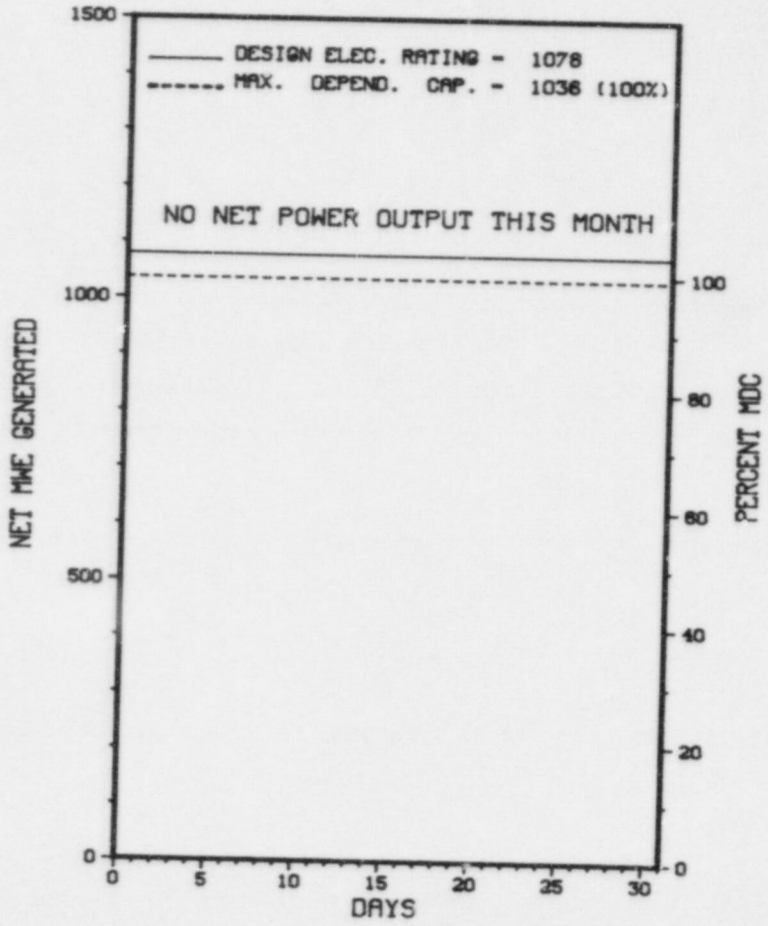
NONE

27. If Currently Shutdown Estimated Startup Date: 09/15/86

****
* LASALLE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

LASALLE 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* LASALLE 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
18	10/19/85	S	744.0	C	4				REFUELING CONTINUES.

****
* SUMMARY *
****

LASALLE 1 REMAINS SHUTDOWN FOR REFUELING.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* LASALLE 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....LA SALLE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...11 MI SE OF
OTTAWA, ILL

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JUNE 21, 1982

DATE ELEC ENER 1ST GENER...SEPTEMBER 4, 1982

DATE COMMERCIAL OPERATE....JANUARY 1, 1984

CONDENSER COOLING METHOD...POND

CONDENSER COOLING WATER....RESERVOIR

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....COMMONWEALTH EDISON

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....M. JORDAN

LICENSING PROJ MANAGER.....A. BOURNIA
DOCKET NUMBER.....50-373

LICENSE & DATE ISSUANCE....NPF-11, AUGUST 13, 1982

PUBLIC DOCUMENT ROOM.....ILLINOIS VALLEY COMMUNITY COLLEGE
RURAL ROUTE NO. 1
OGLESBY, ILLINOIS 16348

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON OCTOBER 23-24, NOVEMBER 5, DECEMBER 10-12, 1985, AND JANUARY 14-16, 29-30, FEBRUARY 12-13, MARCH 5-6, AND 13-14, 1986 (85035; 85036): ROUTINE UNANNOUNCED INSPECTION OF LICENSEE ACTION ON OPEN ITEMS, INSERVICE INSPECTION ACTIVITIES, IMPLEMENTATION OF ACTIONS SET FORTH IN NRC GENERIC LETTERS 84-11, AND ACTIVITIES RELATED TO INDUCTION HEATING STRESS IMPROVEMENT (IHSI). NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION BETWEEN MARCH 24 - JUNE 16 (86020; 86019): INCLUDED INSPECTION OF ALLEGATIONS PERTAINING TO OFF DUTY DRUG AND ALCOHOL USE BY CONTRACTOR PERSONNEL, AND INADEQUATE SECURITY SCREENING PROCEDURES FOR CONTRACTOR PERSONNEL. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED. AN ALLEGATION PERTAINING TO EXCESSIVE CONSUMPTION OF ALCOHOL BY CERTAIN CRAFT PERSONNEL PRIOR TO REPORTING TO WORK HAS BEEN FORWARDED TO THE LICENSEE FOR ACTION. NRC REGION III WILL REVIEW THE INVESTIGATION RESULTS.

INSPECTION ON JUNE 3 - JULY 3 (86022): ROUTINE, ANNOUNCED INSPECTION BY TWO REGIONAL INSPECTORS OF CONTAINMENT INTEGRATED LEAK RATE TEST (CILRT); CILRT PERFORMANCE WITNESSING; LOCAL LEAK RATE AND TEST RESULTS; AS-FOUND CILRT RESULTS; ACTION ON A PREVIOUS INSPECTION FINDING; AND LICENSEE EVENT REPORT FOLLOWUP. NRC INSPECTION MODULES UTILIZED DURING THIS INSPECTION INCLUDE 70307, 70313, 70323, 61720, AND 92701. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON MAY 27-30 AND JUNE 2-3 (86023; 86022): ROUTINE UNANNOUNCED INSPECTION OF THE SOLID RADWASTE MANAGEMENT AND TRANSPORTATION PROGRAMS. ALSO REVIEWED WERE CORRECTIVE ACTIONS FOR PAST INSPECTION FINDINGS, OPEN ITEMS, SELECTED IE INFORMATION NOTICES, AN AIRBORNE RADIOACTIVE MATERIALS OCCURRENCE, AND A REVIEW OF PROGRESS MADE CONCERNING THE RADIATION PROTECTION

INSPECTION SUMMARY

IMPROVEMENT PROGRAM. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON JUNE 18-19 AND 23-24 1986 (86026; 86027): ROUTINE, UNANNOUNCED INSPECTION OF GASEOUS AND LIQUID RADIOACTIVE EFFLUENTS INCLUDING: EFFLUENT RELEASES; RECORDS AND REPORTS OF EFFLUENTS; EFFLUENT CONTROL INSTRUMENTATION, REACTOR COOLANT CHEMISTRY AND ACTIVITY; AND GASEOUS EFFLUENT FILTRATION. ONE VIOLATION INVOLVING FAILURE TO ANALYZE FOR TRITIUM IN MONTHLY COMPOSITE SAMPLES FROM COOLING LAKE BLOWDOWN BETWEEN JUNE 18, 1985 AND MARCH 1, 1986.

INSPECTION ON JULY 2 (86027; 86028): ROUTINE UNANNOUNCED INSPECTION OF THE RESOLUTION OF AN IE BULLETIN. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

SHUT DOWN FOR ITS FIRST REFUELING ON OCTOBER 18, 1985

LAST IE SITE INSPECTION DATE: 07/01/86

INSPECTION REPORT NO: 86034

REPORTS FROM LICENSEE

LASALLE 1

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
84-54	09/21/86	07/07/86	RCIC INOPERABLE AND STEAM LINE ISOLATION
85-32	03/28/85	07/15/86	INBOARD FEEDWATER CHECK VALVE LLRT FAILURE
86-21	06/08/86	07/01/86	CONTROL ROOM VENTILATION ACTUATION DUE TO SPURIOUS RADIATION MONITOR TRIP
86-22	06/12/86	07/02/86	SHUTDOWN COOLING ISOLATION DUE TO OPERATOR ERROR (INATTENTION TO DETAIL)
86-23	06/13/86	07/07/86	SECONDARY CONTAINMENT DAMPER ISOLATION DURING RPS BUS TRANSFER PROCEDURE PREPARATION
86-24	04/22/86	07/30/86	SMALL BORE PIPING DESIGN ERROR
86-25	06/25/86	07/22/86	SPURIOUS TRIP OF CONTROL ROOM VENTILATION HI RADIATION MONITOR DUE TO INCOMPLETE STABILIZATION FOLLOWING CALIBRATION
86-26	06/23/86	07/22/86	FREQUENCY OF PRIMARY CONTAINMENT AIR LOCK LEAKAGE RATE TEST EXCEEDED
86-27	06/28/86	07/28/86	SPURIOUS AMMONIA DETECTOR TRIP CAUSED BY WARPED CAPSTAN ROLLER
86-28	07/02/86	07/31/86	CONTROL ROOM HVAC SYSTEM "B" AMMONIA DETECTOR TRIP DUE TO BROKEN POWER LEAD
86-29	07/03/86	08/01/86	IRM FUSE DESIGN PROBLEM
86-30	07/04/86	08/04/86	CONTROL ROOM VENTILATION ACTUATION DUE TO SPURIOUS RADIATION MONITOR TRIP

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1. Docket: 50-374 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: JAMES P. PETERS (815) 357-6761

4. Licensed Thermal Power (MWt): 3323

5. Nameplate Rating (Gross MWe): 1078

6. Design Electrical Rating (Net MWe): 1078

7. Maximum Dependable Capacity (Gross MWe): 1078

8. Maximum Dependable Capacity (Net MWe): 1036

9. If Changes Occur Above Since Last Report, Give Reasons:

---

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	15,623.0
13. Hours Reactor Critical	.0	3,213.3	8,602.7
14. Rx Reserve Shtdwn Hrs	.0	29.8	1,716.7
15. Hrs Generator On-Line	.0	3,152.2	8,389.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	9,148,872	24,657,423
18. Gross Elec Ener (MWH)	0	3,050,559	8,160,628
19. Net Elec Ener (MWH)	-10,758	2,932,374	7,755,389
20. Unit Service Factor	.0	62.0	53.7
21. Unit Avail Factor	.0	62.0	53.7
22. Unit Cap Factor (MDC Net)	.0	55.6	47.9
23. Unit Cap Factor (DER Net)	.0	53.5	46.0
24. Unit Forced Outage Rate	100.0	38.0	31.0
25. Forced Outage Hours	744.0	1,934.8	3,775.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

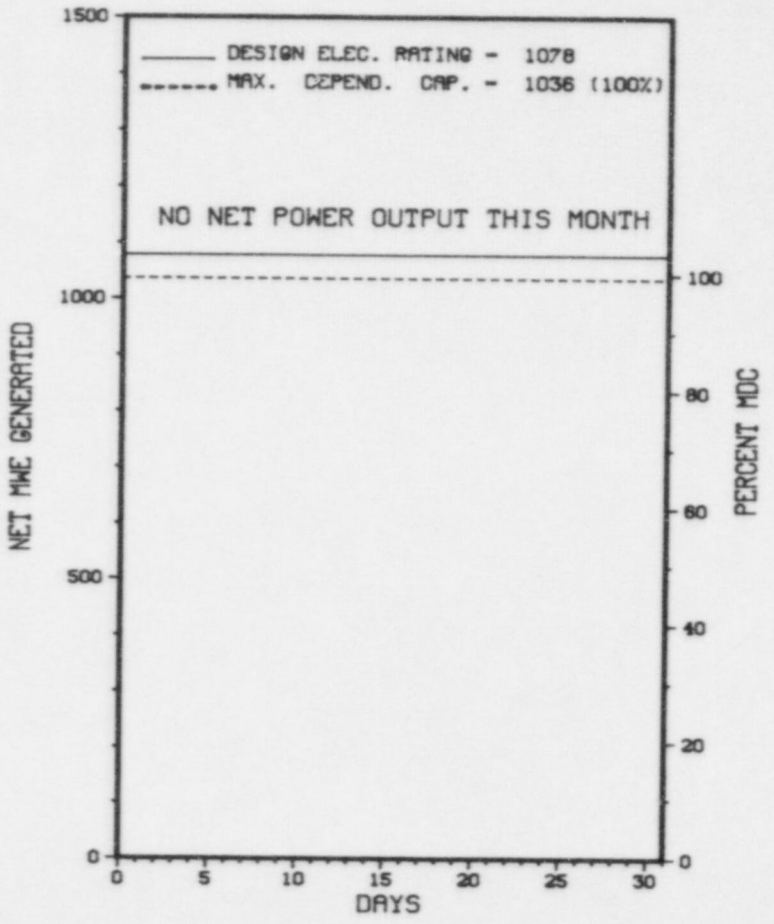
NONE

27. If Currently Shutdown Estimated Startup Date: 08/07/86

****
* LASALLE 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

LASALLE 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

LASALLE 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
23	06/02/86	F	744.0	A	4				STATIC O-RING PROBLEMS.

SUMMARY

LASALLE 2 REMAINS SHUTDOWN FOR EQUIPMENT REPAIRS.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* LASALLE 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....LA SALLE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...11 MI SE OF
OTTAWA, ILL

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...MARCH 10, 1984

DATE ELEC ENER 1ST GENER...APRIL 20, 1984

DATE COMMERCIAL OPERATE....OCTOBER 19, 1984

CONDENSER COOLING METHOD...POND

CONDENSER COOLING WATER....RESERVOIR

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....COMMONWEALTH EDISON

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....M. JORDAN

LICENSING PROJ MANAGER.....A. BOURNIA
DOCKET NUMBER.....50-374

LICENSE & DATE ISSUANCE....NPF-18, MARCH 23, 1984

PUBLIC DOCUMENT ROOM.....ILLINOIS VALLEY COMMUNITY COLLEGE
RURAL ROUTE NO. 1
OGLESBY, ILLINOIS 16348

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON OCTOBER 23-24, NOVEMBER 5, DECEMBER 10-12, 1985, AND JANUARY 14-16, 29-30, FEBRUARY 12-13, MARCH 5-6, AND 13-14, 1986 (85035; 85036): ROUTINE UNANNOUNCED INSPECTION OF LICENSEE ACTION ON OPEN ITEMS, INSERVICE INSPECTION ACTIVITIES, IMPLEMENTATION OF ACTIONS SET FORTH IN NRC GENERIC LETTERS 84-11, AND ACTIVITIES RELATED TO INDUCTION HEATING STRESS IMPROVEMENT (IHSI). NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION BETWEEN MARCH 24 - JUNE 16 (86020; 86019): INCLUDED INSPECTION OF ALLEGATIONS PERTAINING TO OFF DUTY DRUG AND ALCOHOL USE BY CONTRACTOR PERSONNEL, AND INADEQUATE SECURITY SCREENING PROCEDURES FOR CONTRACTOR PERSONNEL. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED. AN ALLEGATION PERTAINING TO EXCESSIVE CONSUMPTION OF ALCOHOL BY CERTAIN CRAFT PERSONNEL PRIOR TO REPORTING TO WORK HAS BEEN FORWARDED TO THE LICENSEE FOR ACTION. NRC REGION III WILL REVIEW THE INVESTIGATION RESULTS.

INSPECTION ON MAY 27-30 AND JUNE 2-3 (86023; 86022): ROUTINE UNANNOUNCED INSPECTION OF THEHE SOLID RADWASTE MANAGEMENT AND TRANSPORTATION PROGRAMS. ALSO REVIEWED WERE CORRECTIVE ACTIONS FOR PAST INSPECTION FINDINGS, OPEN ITEMS, SELECTED IE INFORMATION NOTICES, AN AIRBORNE RADIOACTIVE MATERIALS OCCURRENCE, AND A REVIEW OF PROGRESS MADE CONCERNING THE RADIATION PROTECTION IMPROVEMENT PROGRAM. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON JUNE 18-19 AND 23-24 1986 (86026; 86027): ROUTINE, UNANNOUNCED INSPECTION OF GASEOUS AND LIQUID RADIOACTIVE EFFLUENTS INCLUDING: EFFLUENT RELEASES; RECORDS AND REPORTS OF EFFLUENTS; EFFLUENT CONTROL INSTRUMENTATION, REACTOR COOLANT CHEMISTRY AND ACTIVITY; AND GASEOUS EFFLUENT FILTRATION. ONE VIOLATION INVOLVING FAILURE TO ANALYZE FOR TRITIUM IN MONTHLY

INSPECTION SUMMARY

COMPOSITE SAMPLES FROM COOLING LAKE BLOWDOWN BETWEEN JUNE 18, 1985 AND MARCH 1, 1986.

INSPECTION ON JULY 2 (86027; 86028): ROUTINE UNANNOUNCED INSPECTION OF THE RESOLUTION OF AN IE BULLETIN. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

SHUT DOWN 06/02/86 DUE TO SOR SWITCH PROBLEMS

LAST IE SITE INSPECTION DATE: 07/31/86

INSPECTION REPORT NO: 86035

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
84-16	04/23/86	07/21/86	REACTOR WATER CLEANUP HIGH AMBIENT TEMPERATURE ISOLATION
85-48	12/24/86	07/10/86	RCIC WATER LEG PUMP FAILURE
86-11	06/01/86	07/01/86	FAILURE OF LEVEL 3 SCRAM SWITHCES TO OPERATE AT CALIBRATED SETPOINT
86-12	06/20/86	07/16/86	IMPROPER TERMINATIONS AT ENVIRONMENTALLY QUALIFIED EQUIPMENT

1. Docket: 50-352 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: BILL ALDEN (215) 841-5022
4. Licensed Thermal Power (MWt): 3293
5. Nameplate Rating (Gross MWe): 1092
6. Design Electrical Rating (Net MWe): 1055
7. Maximum Dependable Capacity (Gross MWe): 1092
8. Maximum Dependable Capacity (Net MWe): 1055
9. If Changes Occur Above Since Last Report, Give Reasons:

---

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	4,343.0	4,343.0
13. Hours Reactor Critical	745.3	3,044.0	3,044.0
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	532.7	2,963.0	2,963.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,679,930	9,638,998	9,638,998
18. Gross Elec Ener (MWH)	545,730	3,177,130	3,177,130
19. Net Elec Ener (MWH)	521,012	3,044,102	3,044,102
20. Unit Service Factor	71.6	68.2	68.2
21. Unit Avail Factor	71.6	68.2	68.2
22. Unit Cap Factor (MDC Net)	66.4	66.4	66.4
23. Unit Cap Factor (DER Net)	66.4	66.4	66.4
24. Unit Forced Outage Rate	28.4	7.4	7.4
25. Forced Outage Hours	211.3	237.7	237.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

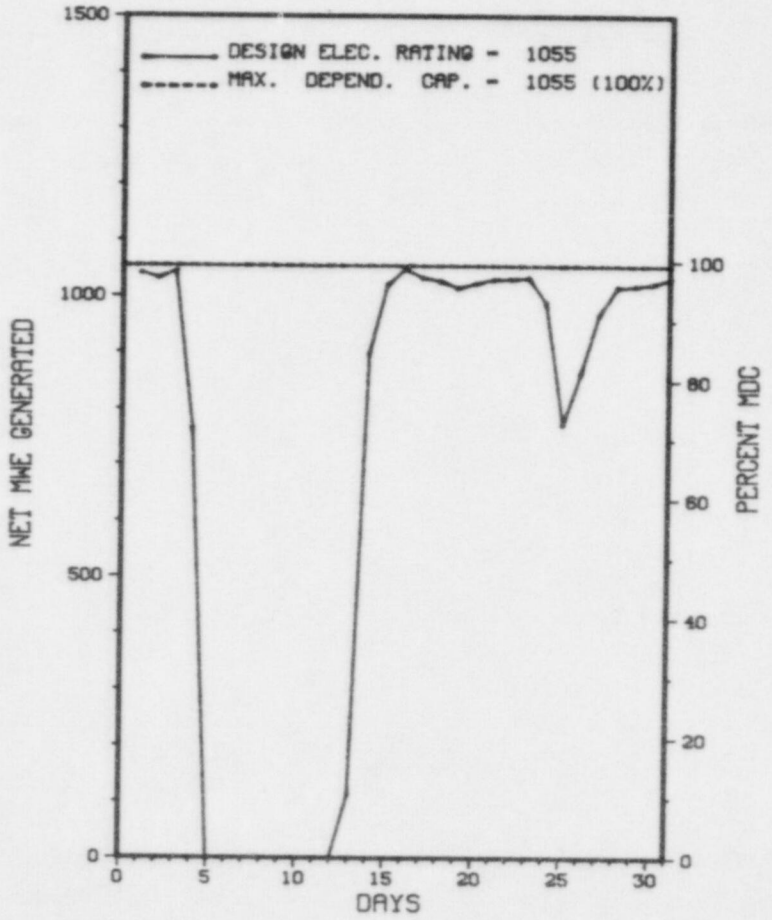
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* LIMERICK 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

LIMERICK 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

LIMERICK 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	07/04/86	F	211.3	A	1		CF	VALVEX	INCREASING DRYWELL UNIDENTIFIED LEAKAGE, AS MEASURED BY FLOOR DRAIN FLOW (FI-61-115) AND AIR COOLER DRAIN FLOW (FI-87-120), APPROPRIATE TECH SPEC 3.4.3.2 LIMIT OF 5GPM INITIATED DECISION TO SHUTDOWN. PACKING WAS REPLACED ON 'A' LPCI MANUAL ISOLATION VALVE 51-1F065.
2	07/24/86	S	0.0	H	5		ZZ	ZZZZZZ	POWER REDUCTION FROM 1070MWE TO 700MWE FOR SCHEDULED CONTROL ROD PATTERN ADJUSTMENT.

SUMMARY

LIMERICK 1 OPERATED WITH 1 OUTAGE AND 1 REDUCTION IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* LIMERICK 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....PENNSYLVANIA

COUNTY.....MONTGOMERY

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...21 MI NW OF
PHILADELPHIA,PA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...DECEMBER 22, 1984

DATE ELEC ENER 1ST GENER...APRIL 13, 1985

DATE COMMERCIAL OPERATE....FEBRUARY 1, 1986

CONDENSER COOLING METHOD...CC HNDCT

CONDENSER COOLING WATER....SCHUYLKILL RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PHILADELPHIA ELECTRIC

CORPORATE ADDRESS.....2301 MARKET STREET
PHILADELPHIA, PENNSYLVANIA 19105

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....G. KELLY

LICENSING PROJ MANAGER.....R. MARTIN
DOCKET NUMBER.....50-352

LICENSE & DATE ISSUANCE....NPF-39, AUGUST 8, 1985

PUBLIC DOCUMENT ROOM.....POTTSTOWN PUBLIC LIBRARY
500 HIGH STREET
POTTSTOWN, PENNSYLVANIA 19464

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

****
* LIMERICK 1 *
****

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-309 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: STEVEN L. JACOBS (207) 623-3521
4. Licensed Thermal Power (MWt): 2630
5. Nameplate Rating (Gross MWe): 864
6. Design Electrical Rating (Net MWe): 825
7. Maximum Dependable Capacity (Gross MWe): 850
8. Maximum Dependable Capacity (Net MWe): 810
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	120,323.6
13. Hours Reactor Critical	320.0	4,169.7	96,507.1
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	304.3	4,084.9	93,633.6
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	724,170	9,972,953	211,973,637
18. Gross Elec Ener (MWH)	240,350	3,352,660	69,592,380
19. Net Elec Ener (MWH)	232,241	3,233,728	66,424,020
20. Unit Service Factor	40.9	80.3	77.8
21. Unit Avail Factor	40.9	80.3	77.8
22. Unit Cap Factor (MDC Net)	38.5	78.5	69.9*
23. Unit Cap Factor (DER Net)	37.8	77.1	68.1*
24. Unit Forced Outage Rate	.0	11.4	7.1
25. Forced Outage Hours	.0	523.5	6,296.0

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

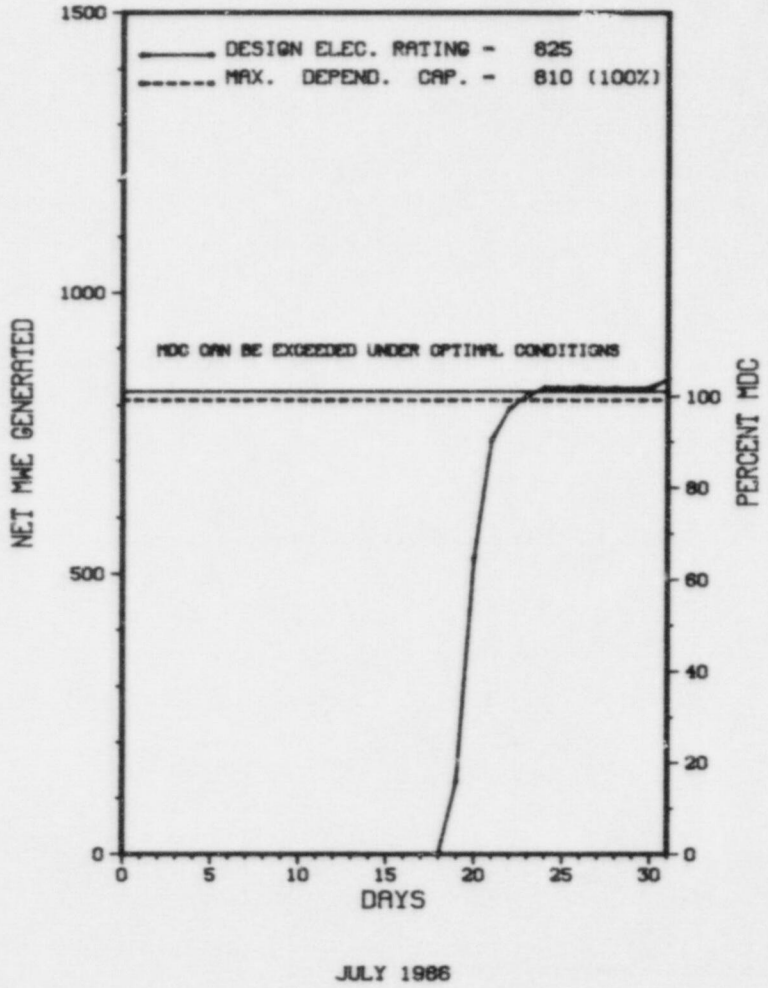
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* MAINE YANKEE *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MAINE YANKEE



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

MAINE YANKEE

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-9	06/30/86	S	439.7	B	4		CD	VALVEX	MAINTENANCE OUTAGE TO REPAIR EFCV AIR CYLINDERS AND TO INSPECT CONDENSER. INSTALLED NEW TYPE RUPTURE DISK ASSEMBLY IN EFCV AIR CYLINDERS. INSPECTED CONDENSER AND LP TURBINE.

SUMMARY

MAINE YANKEE RETURNED ONLINE FROM A MAINTENANCE OUTAGE ON JULY 19TH AND OPERATED ROUTINELY THE REMAINDER OF THE MONTH.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* MAINE YANKEE *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....MAINE

COUNTY.....LINCOLN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...10 MI N OF
BATH, ME

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...OCTOBER 23, 1972

DATE ELEC ENER 1ST GENER...NOVEMBER 8, 1972

DATE COMMERCIAL OPERATE....DECEMBER 28, 1972

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....BACK RIVER

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....MAINE YANKEE ATOMIC POWER

CORPORATE ADDRESS.....83 EDISON DRIVE
AUGUSTA, MAINE 04366

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....C. HOLDEN

LICENSING PROJ MANAGER.....P. SEARS
DOCKET NUMBER.....50-309

LICENSE & DATE ISSUANCE....DPR-36, JUNE 29, 1973

PUBLIC DOCUMENT ROOM.....WISCASSET PUBLIC LIBRARY
HIGH STREET
WISCASSET, MAINE 04578

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 5.8.1.A AND REGULATORY GUIDE 1.33 REQUIRE PROCEDURES FOR ACTIVITIES AFFECTING QUALITY. OPERATIONS PROCEDURE 1-26-4, RESPONSIBILITIES AND AUTHORITIES OF OPERATING PERSONNEL, ASSIGNS THE RESPONSIBILITY FOR OPERATION AND MONITORING OF EQUIPMENT TO THE AUXILIARY OPERATOR THROUGH THE SHIFT OPERATING SUPERVISORS. CONTRARY TO THE ABOVE, ON FEBRUARY 10, 1986 A PLANT ENGINEER OPERATED A VALVE IN THE AUXILIARY FEEDWATER SYSTEM IN ORDER TO OBTAIN A PRESSURE READING. (8600 5)

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

OTHER ITEMS

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-369 O P E R A T I N G S T A T U S

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: J. A. REAVIS (704) 373-7567

4. Licensed Thermal Power (MWt): 3411

5. Nameplate Rating (Gross MWe): 1305

6. Design Electrical Rating (Net MWe): 1150

7. Maximum Dependable Capacity (Gross MWe): 1225

8. Maximum Dependable Capacity (Net MWe): 1150

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	40,895.0
13. Hours Reactor Critical	.0	3,113.4	28,119.9
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	3,094.7	27,806.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	9,789,256	77,154,669
18. Gross Elec Ener (MWH)	0	3,415,928	26,726,581
19. Net Elec Ener (MWH)	-4,339	3,265,836	25,417,850
20. Unit Service Factor	.0	60.8	68.0
21. Unit Avail Factor	.0	60.8	68.0
22. Unit Cap Factor (MDC Net)	.0	55.8	54.0
23. Unit Cap Factor (DER Net)	.0	55.8	54.0
24. Unit Forced Outage Rate	.0	4.6	13.3
25. Forced Outage Hours	.0	150.0	4,261.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

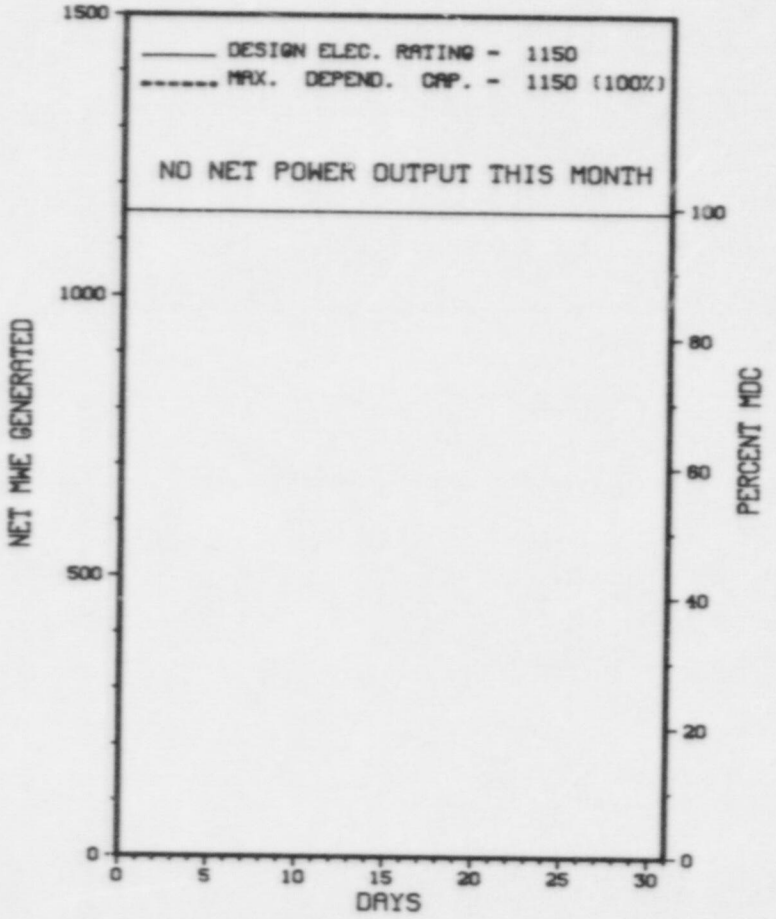
NONE

27. If Currently Shutdown Estimated Startup Date: 09/02/86

****
* MCGUIRE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MCGUIRE 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* MCGUIRE 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
7	05/16/86	S	744.0	C	4		RC	FUELXX	END OF CYCLE 3 REFUELING OUTAGE CONTINUES.

****
* SUMMARY *
****

MCGUIRE 1 REMAINS SHUTDOWN FOR REFUELING.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* MCGUIRE 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....NORTH CAROLINA

COUNTY.....MECKLENBURG

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...17 MI N OF
CHARLOTTE, NC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 8, 1981

DATE ELEC ENER 1ST GENER...SEPTEMBER 12, 1981

DATE COMMERCIAL OPERATE....DECEMBER 1, 1981

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE NORMAN

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUKE POWER

CORPORATE ADDRESS.....422 SOUTH CHURCH STREET
CHARLOTTE, NORTH CAROLINA 28242

CONTRACTOR
ARCHITECT/ENGINEER.....DUKE POWER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....DUKE POWER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....W. ORDERS

LICENSING PROJ MANAGER.....D. HOOD
DOCKET NUMBER.....50-369

LICENSE & DATE ISSUANCE....NPF-9, JULY 8, 1981

PUBLIC DOCUMENT ROOM.....MS. DAWN HUBBS
ATKINS LIBRARY
UNIVERSITY OF NORTH CAROLINA - CHARLOTTE
UNCC STATION,
CHARLOTTE, NC 28223

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MARCH 1 - APRIL 20 (86-08): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED ON SITE IN THE AREAS OF OPERATIONS, SURVEILLANCE TESTING AND MAINTENANCE ACTIVITIES. OF THE AREAS INSPECTED, TWO VIOLATIONS WERE IDENTIFIED IN THE AREAS OF SURVEILLANCE TESTING AND PROCEDURE ADHERENCE.

INSPECTION APRIL 21 - MAY 20 (86-12): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED ON SITE IN THE AREAS OF OPERATIONS, SURVEILLANCE TESTING AND MAINTENANCE ACTIVITIES. OF THE AREAS INSPECTED, NO VIOLATIONS WERE IDENTIFIED IN THE AREAS OF SURVEILLANCE TESTING AND MAINTENANCE ACTIVITIES.

INSPECTION MAY 27-30 (86-13): THIS ROUTINE, UNANNOUNCED INSPECTION WAS SCHEDULED FOR THE PURPOSE OF OBSERVING INSERVICE INSPECTION (ISI) ACTIVITIES ON SAFETY-RELATED COMPONENTS. HOWEVER, THE PRESENCE OF UNEXPECTEDLY HIGH AIRBORNE ACTIVITY PRECLUDED THE LICENSEE FROM PERFORMING ISI WORK SCHEDULED FOR THIS TIME. THEREFORE, THE INSPECTOR DISCUSSED THE REPLACEMENT OF FLOW ELEMENTS IN THE NUCLEAR SERVICE WATER SYSTEM, REVIEWED LICENSEE ACTIONS ON IE BULLETINS, PREVIOUS ENFORCEMENT MATTERS, AND INSPECTOR IDENTIFIED ITEMS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 9-13 (86-14): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED ON SITE IN THE AREAS OF INSERVICE INSPECTION (ISI), WORK OBSERVATION, EVALUATION AND REVIEW OF RECORDS, EDDY CURRENT (EC) EXAMINATION RESULTS OF STEAM GENERATORS (SG) TUBES; NUCLEAR SERVICE WATER FLOW ELEMENT REPLACEMENT ON PUMP 1-B; SHOT PEENING OF S&G TUBES; REVIEW OF SNUBBER FUNCTIONAL TEST RECORDS.

INSPECTION SUMMARY

ONE VIOLATION WAS IDENTIFIED - VIOLATION 369/86-14-01, INSTRUMENTS AND SHOT PEENING MATERIAL USED TO QUALIFY SG TUBE SHOT PEENING PROCEDURE AND PERSONNEL NOT IDENTIFIED.

INSPECTION JUNE 3-5 (86-15): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED ONSITE INSPECTION DURING NORMAL DUTY HOURS IN THE AREA OF EXTERNAL EXPOSURE CONTROL. ONE VIOLATION - FAILURE TO FOLLOW RADIOLOGICAL PROTECTION PROCEDURES.

ENFORCEMENT SUMMARY

CONTRARY TO TS 6.8.1.A, THE REACTIVITY BALANCE CALCULATION PROCEDURE, AND OPERATIONS MANAGEMENT PROCEDURE 2.5, SECTION 6.6: (1) ON MARCH 11, 1986, MCGUIRE UNIT 1 WAS BEING TAKEN CRITICAL WHEN THE CONTROL RODS WERE WITHDRAWN TO THE WITHDRAWAL LIMIT SPECIFIED IN THE REACTIVITY BALANCE PROCEDURE AND THE CONTROL RODS WERE NOT INSERTED AS REQUIRED. (2) ON MARCH 12, 1986 TRAIN 'B' OF CONTAINMENT SPRAY AND TRAIN 'A' OF SSPS (SOLID STATE PROTECTION SYSTEM) WERE REMOVED FROM SERVICE SIMULTANEOUSLY. THUS, BOTH TRAINS OF CONTAINMENT SPRAY WERE REMOVED FROM SERVICE SIMULTANEOUSLY. CONTRARY TO TS 4.11.1.2, 4.11.2.2, 4.11.2.3 AND 4.11.4.1, THE CONTRIBUTIONS FROM LIQUID AND GASEOUS EFFLUENTS FOR THE CURRENT CALENDAR QUARTER OR YEAR WERE NOT CALCULATED WITHIN THE 31 DAY SPAN PRECEDING JANUARY 29, 1986.
(8600 4)

CONTRARY TO TS 6.8.1, APPENDIX A, REGULATORY GUIDE 1.33, PARAGRAPH 7.E.1, AND HEALTH PHYSICS MANUAL SECTION 2.4, RADIATION WORK PERMITS, PARAGRAPH 2.4.1, SPECIFIC PROTECTIVE CLOTHING REQUIREMENTS WERE NOT DELINEATED ON RWP NO. 86-1183 IN THAT VERBAL UPGRADES IN RADIOLOGICAL CONTROL REQUIRMENTS HAD BEEN MADE WITHOUT WRITTEN RWP REVISIONS SUCH THAT ON MAY 29, 1986, AN INDIVIDUAL IN UNIT 1 LOWER CONTAINMENT, WHO WAS FOLLOWING THE WRITTEN RWP, DRESSED INAPPROPRIATELY FOR THE RADIATION HAZARD THAT HAD BEEN DETERMINED TO BE PRESENT.
(8601 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REFUELING.

LAST IE SITE INSPECTION DATE: JUNE 3-5, 1986 +

INSPECTION REPORT NO: 50-369/86-15 +

REPORTS FROM LICENSEE

MCGUIRE 1

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-011	06/05/86	07/18/86	DIESEL GENERATOR 1A AUTOMATICALLY STARTED DUE TO A UNIT 1 TRAIN "A" BLACKOUT ACTUATION; CAUSE - DESIGN DEFICIENCY.

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1. Docket: 50-370 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: J. A. REAVIS EXT (704) 373-7567
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1450 X .9 = 1305
6. Design Electrical Rating (Net MWe): 1150
7. Maximum Dependable Capacity (Gross MWe): 1225
8. Maximum Dependable Capacity (Net MWe): 1150
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	21,191.0
13. Hours Reactor Critical	728.5	2,614.3	14,243.1
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	725.7	2,480.7	13,746.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,419,839	9,155,531	45,306,561
18. Gross Elec Ener (MWH)	843,355	2,861,132	15,561,124
19. Net Elec Ener (MWH)	811,080	2,724,707	14,878,360
20. Unit Service Factor	97.5	48.8	64.9
21. Unit Avail Factor	97.5	48.8	64.9
22. Unit Cap Factor (MDC Net)	94.8	46.6	61.1
23. Unit Cap Factor (DER Net)	94.8	46.6	61.1
24. Unit Forced Outage Rate	2.5	5.5	16.0
25. Forced Outage Hours	18.3	143.4	2,610.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

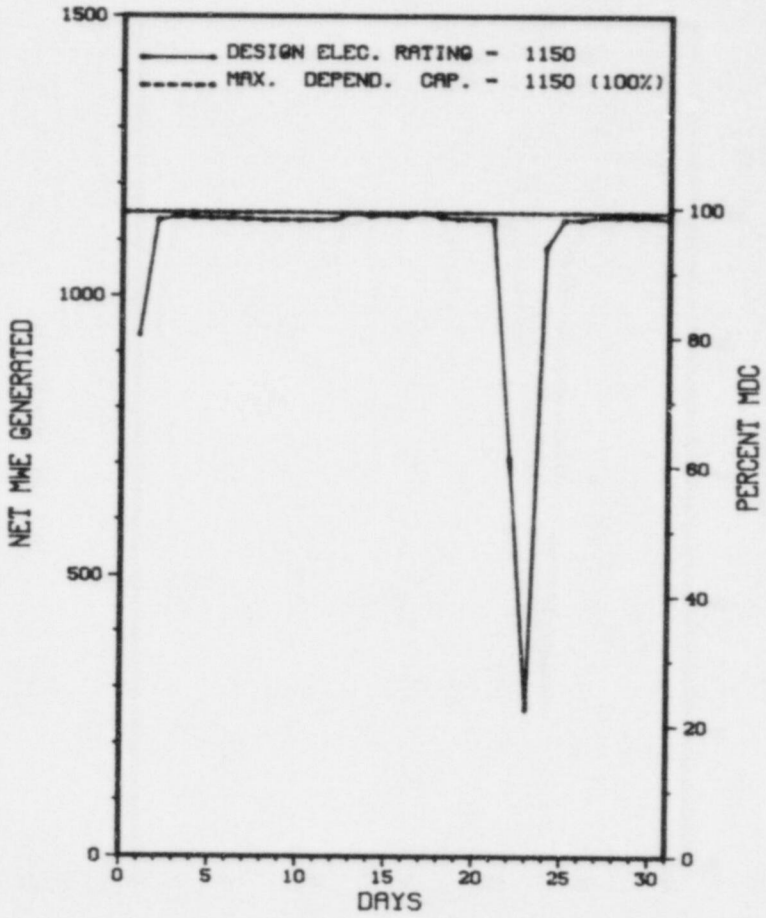
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* MCGUIRE 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MCGUIRE 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

MCGUIRE 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
11-P	07/01/86	S	0.0	B	5		IB	INSTRU	NUCLEAR INSTRUMENTATION CALIBRATION.
12-P	07/21/86	F	0.0	A	5		HA	INSTRU	TURBINE GOVERNING SYSTEM PROBLEM.
9	07/22/86	F	18.3	A	3		HA	INSTRU	A FAILURE IN TURBINE CONTROL CIRCUITRY CAUSED TURBINE GOVERNOR VALVES TO CLOSE.
13-P	07/23/86	F	0.0	F	5		XX	ZZZZZZ	HOLD FOR SECONDARY CHEMISTRY.
14-P	07/23/86	F	0.0	B	5		IB	INSTRU	NUCLEAR INSTRUMENTATION CALIBRATION.
15-P	07/23/86	F	0.0	F	5		XX	ZZZZZZ	HOLD FOR SECONDARY CHEMISTRY (SULFATES 100 PPB).
16-P	07/23/86	F	0.0	B	5		RC	ZZZZZZ	REACTOR COOLANT SYSTEM LEAKAGE CALCULATION.
17-P	07/24/86	F	0.0	H	5		HH	PUMPXX	PUT HEATER DRAIN PUMPS IN SERVICE.
18-P	07/24/86	F	0.0	B	5		RC	ZZZZZZ	REACTOR COOLANT SYSTEM LEAKAGE CALCULATION.

SUMMARY

MCGUIRE 2 OPERATED WITH 1 OUTAGE AND NUMEROUS REDUCTIONS LISTED IN DETAIL ABOVE.

Type

F-Forced
S-Sched

Reason

A-Equip Failure
B-Maint or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Admin
G-Oper Error
H-Other

Method

1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other

System & Component

Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* MCGUIRE 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....NORTH CAROLINA

COUNTY.....MECKLENBURG

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...17 MI N OF
CHARLOTTE, NC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 8, 1983

DATE ELEC ENER 1ST GENER...MAY 23, 1983

DATE COMMERCIAL OPERATE....MARCH 1, 1984

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE NORMAN

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUKE POWER

CORPORATE ADDRESS.....POWER BLDG., BOX 2178
CHARLOTTE, NORTH CAROLINA 28201

CONTRACTOR
ARCHITECT/ENGINEER.....DUKE POWER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....DUKE POWER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....W. ORDERS

LICENSING PROJ MANAGER.....D. HOOD
DOCKET NUMBER.....50-370

LICENSE & DATE ISSUANCE....NPF-17, MAY 27, 1983

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UNIVERSITY OF NORTH CAROLINA - CHARLOTTE
UNCC STATION,
CHARLOTTE, NC 28223

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MARCH 1 - APRIL 20 (86-08): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED ON SITE IN THE AREAS OF OPERATIONS, SURVEILLANCE TESTING AND MAINTENANCE ACTIVITIES. OF THE AREAS INSPECTED, TWO VIOLATIONS WERE IDENTIFIED IN THE AREAS OF SURVEILLANCE TESTING AND PROCEDURE ADHERENCE.

INSPECTION APRIL 21 - MAY 20 (86-12): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED ON SITE IN THE AREAS OF OPERATIONS, SURVEILLANCE TESTING AND MAINTENANCE ACTIVITIES. OF THE AREAS INSPECTED, NO VIOLATIONS WERE IDENTIFIED IN THE AREAS OF SURVEILLANCE TESTING AND MAINTENANCE ACTIVITIES.

INSPECTION MAY 27-30 (86-13): THIS ROUTINE, UNANNOUNCED INSPECTION WAS SCHEDULED FOR THE PURPOSE OF OBSERVING INSERVICE INSPECTION (ISI) ACTIVITIES ON SAFETY-RELATED COMPONENTS. HOWEVER, THE PRESENCE OF UNEXPECTEDLY HIGH AIRBORNE ACTIVITY PRECLUDED THE LICENSEE FROM PERFORMING ISI WORK SCHEDULED FOR THIS TIME. THEREFORE, THE INSPECTOR DISCUSSED THE REPLACEMENT OF FLOW ELEMENTS IN THE NUCLEAR SERVICE WATER SYSTEM, REVIEWED LICENSEE ACTIONS ON IE BULLETINS, PREVIOUS ENFORCEMENT MATTERS, AND INSPECTOR IDENTIFIED ITEMS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 9-13 (86-14): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED ON SITE IN THE AREAS OF INSERVICE INSPECTION (ISI), WORK OBSERVATION, EVALUATION AND REVIEW OF RECORDS, EDDY CURRENT (EC) EXAMINATION RESULTS OF STEAM GENERATORS (SG) TUBES; NUCLEAR SERVICE WATER FLOW ELEMENT REPLACEMENT ON PUMP 1-B; SHOT PEENING OF S&G TUBES; REVIEW OF SNUBBER FUNCTIONAL TEST RECORDS.

****
* MCGUIRE 2 *
****

INSPECTION SUMMARY

ONE VIOLATION WAS IDENTIFIED - VIOLATION 369/86-14-01, INSTRUMENTS AND SHOT PEENING MATERIAL USED TO QUALIFY SG TUBE SHOT PEENING PROCEDURE AND PERSONNEL NOT IDENTIFIED.

INSPECTION JUNE 3-5 (86-15): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED ONSITE INSPECTION DURING NORMAL DUTY HOURS IN THE AREA OF EXTERNAL EXPOSURE CONTROL. ONE VIOLATION - FAILURE TO FOLLOW RADIOLOGICAL PROTECTION PROCEDURES.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

POWER OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 3-5, 1986 +

INSPECTION REPORT NO: 50-370/86-15 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-245 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: G. NEWBURGH (203) 447-1791 X4400
4. Licensed Thermal Power (MWt): 2011
5. Nameplate Rating (Gross MWe): 735 X 0.9 = 662
6. Design Electrical Rating (Net MWe): 660
7. Maximum Dependable Capacity (Gross MWe): 684
8. Maximum Dependable Capacity (Net MWe): 654
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	137,375.0
13. Hours Reactor Critical	744.0	4,903.4	105,982.5
14. Rx Reserve Shtdwn Hrs	.0	148.9	2,983.4
15. Hrs Generator On-Line	744.0	4,862.0	103,051.2
16. Unit Reserve Shtdwn Hrs	.0	183.7	277.4
17. Gross Therm Ener (MWH)	1,471,518	9,630,694	190,244,892
18. Gross Elec Ener (MWH)	496,000	3,285,600	63,986,896
19. Net Elec Ener (MWH)	474,229	3,141,789	61,034,302
20. Unit Service Factor	100.0	95.6	75.0
21. Unit Avail Factor	100.0	99.2	75.2
22. Unit Cap Factor (MDC Net)	97.5	94.4	67.9
23. Unit Cap Factor (DER Net)	96.6	93.6	67.3
24. Unit Forced Outage Rate	.0	.8	11.9
25. Forced Outage Hours	.0	41.3	5,855.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

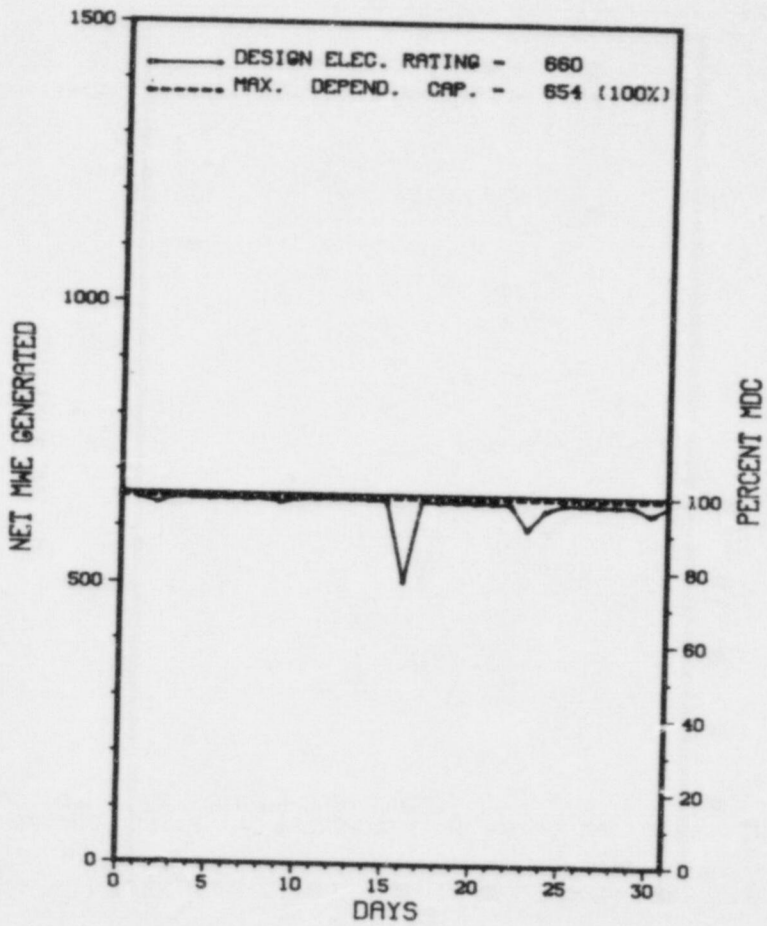
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* MILLSTONE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MILLSTONE 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

MILLSTONE 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
6	07/16/86	S	0.0	B	5				POWER REDUCTION FOR CONTROL ROD PATTERN ADJUSTMENT AND TO FIND AND REPAIR CONDENSER TUBE LEAKS.

SUMMARY

MILLSTONE 1 OPERATED WITH 1 REDUCTION FOR MAINTENANCE DURING JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* MILLSTONE 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....CONNECTICUT

COUNTY.....NEW LONDON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI SW OF
NEW LONDON, CONN

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...OCTOBER 26, 1970

DATE ELEC ENER 1ST GENER...NOVEMBER 29, 1970

DATE COMMERCIAL OPERATE....MARCH 1, 1971

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LONG ISLAND SOUND

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NORTHEAST NUCLEAR ENERGY

CORPORATE ADDRESS.....P.O. BOX 270
HARTFORD, CONNECTICUT 06101

CONTRACTOR
ARCHITECT/ENGINEER.....EBASCO

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....EBASCO

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....J. SHEDLOSKY

LICENSING PROJ MANAGER.....J. SHEA
DOCKET NUMBER.....50-245

LICENSE & DATE ISSUANCE....DPR-21, OCTOBER 26, 1970

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WATERFORD, CONNECTICUT 06385

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

****
* MILLSTONE 1 *
****

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-336 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: G. NERON (203) 447-1791 X4417
4. Licensed Thermal Power (MWt): 2700
5. Nameplate Rating (Gross MWe): 1011 X 0.9 = 910
6. Design Electrical Rating (Net MWe): 870
7. Maximum Dependable Capacity (Gross MWe): 889
8. Maximum Dependable Capacity (Net MWe): 857
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	92,903.0
13. Hours Reactor Critical	744.0	5,080.4	66,502.8
14. Rx Reserve Shtdwn Hrs	.0	.0	2,166.9
15. Hrs Generator On-Line	744.0	4,986.9	63,704.3
16. Unit Reserve Shtdwn Hrs	.0	.0	468.2
17. Gross Therm Ener (MWH)	2,007,906	13,356,360	162,264,869
18. Gross Elec Ener (MWH)	648,700	4,323,900	52,656,373
19. Net Elec Ener (MWH)	625,598	4,165,559	50,488,340
20. Unit Service Factor	100.0	98.0	68.6
21. Unit Avail Factor	100.0	98.0	69.1
22. Unit Cap Factor (MDC Net)	98.1	95.6	64.4*
23. Unit Cap Factor (DER Net)	96.7	94.1	63.4*
24. Unit Forced Outage Rate	.0	.3	16.2
25. Forced Outage Hours	.0	13.0	11,075.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

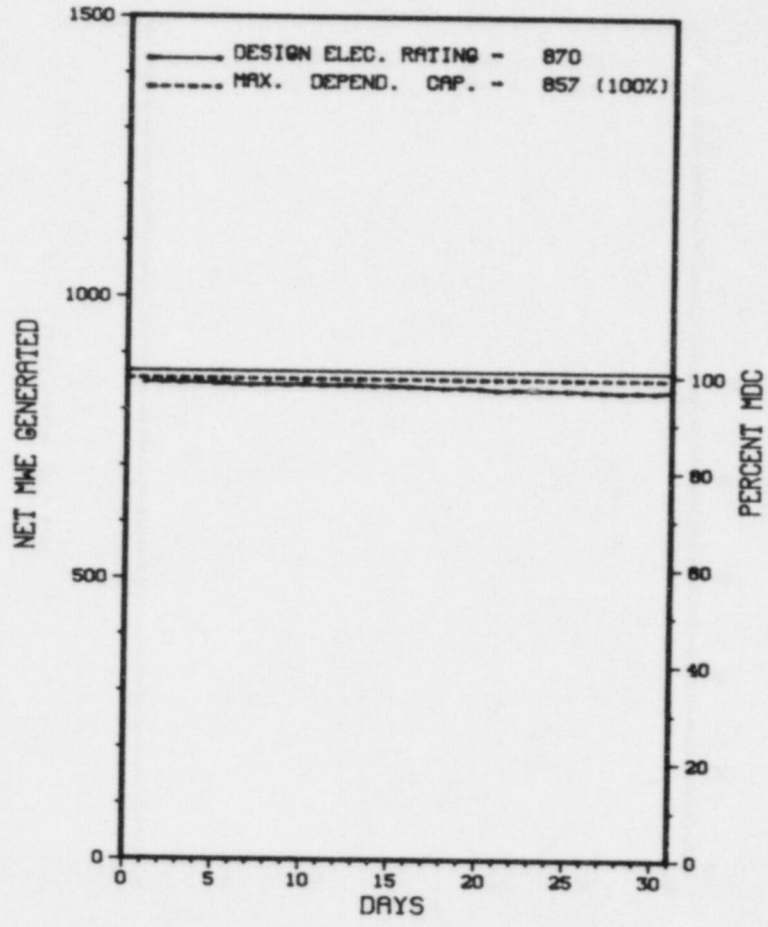
REFUELING: SEPTEMBER, 1986 - 14 WEEKS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* MILLSTONE 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MILLSTONE 2



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

MILLSTONE 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence

NONE

SUMMARY

MILLSTONE 2 OPERATED AT NEAR FULL POWER IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* MILLSTONE 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....CONNECTICUT

COUNTY.....NEW LONDON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI SW OF
NEW LONDON, CONN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...OCTOBER 17, 1975

DATE ELEC ENER 1ST GENER...NOVEMBER 9, 1975

DATE COMMERCIAL OPERATE....DECEMBER 26, 1975

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LONG ISLAND SOUND

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NORTHEAST NUCLEAR ENERGY

CORPORATE ADDRESS.....P.O. BOX 270
HARTFORD, CONNECTICUT 06101

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....T. REBELOWSKI

LICENSING PROJ MANAGER.....D. JAFFE
DOCKET NUMBER.....50-336

LICENSE & DATE ISSUANCE....DPR-65, SEPTEMBER 30, 1975

PUBLIC DOCUMENT ROOM.....WATERFORD PUBLIC LIBRARY
49 ROPE FERRY ROAD
WATERFORD, CONNECTICUT 06385

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

****
* MILLSTONE 2 *
****

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-423 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: A. ELMS (203) 444-5388

4. Licensed Thermal Power (MWt): 3411

5. Nameplate Rating (Gross MWe): 1253

6. Design Electrical Rating (Net MWe): 1154

7. Maximum Dependable Capacity (Gross MWe): 1197

8. Maximum Dependable Capacity (Net MWe): 1149

9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	2,399.0	2,399.0
13. Hours Reactor Critical	575.0	2,135.3	2,135.3
14. Rx Reserve Shtdwn Hrs	.0	83.2	83.2
15. Hrs Generator On-Line	575.0	2,119.2	2,119.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,943,450	11,049,833	11,049,833
18. Gross Elec Ener (MWH)	669,546	2,449,912	2,449,912
19. Net Elec Ener (MWH)	639,674	2,342,519	2,342,519
20. Unit Service Factor	77.3	88.3	88.3
21. Unit Avail Factor	77.3	88.3	88.3
22. Unit Cap Factor (MDC Net)	74.8	85.0	85.0
23. Unit Cap Factor (DER Net)	74.5	84.6	84.6
24. Unit Forced Outage Rate	22.7	11.7	11.7
25. Forced Outage Hours	169.0	279.8	279.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

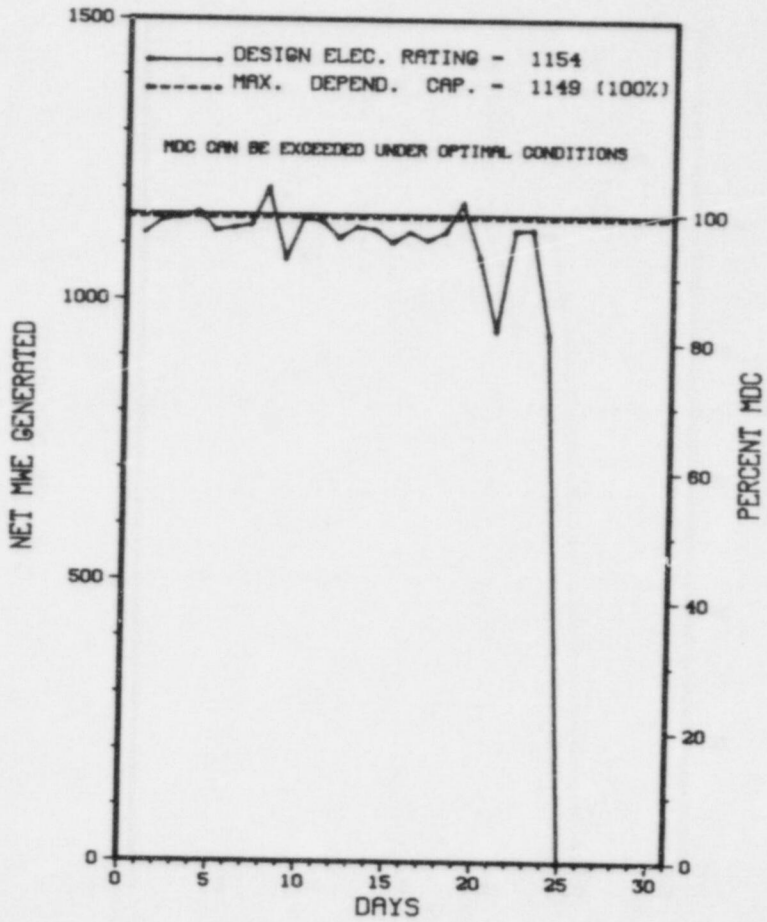
NONE

27. If Currently Shutdown Estimated Startup Date: 09/13/86

****
* MILLSTONE 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MILLSTONE 3



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

MILLSTONE 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-04	07/21/86	F	0.0	G	5		SM	P	LEVEL CONTROL VALVE FAILURE DUE TO IMPROPER TAGGING CAUSES HEATER DRAIN PUMP TRIP.
86-05	07/24/86	F	169.0	A	3	86-41	SJ	TC	TRIP DURING SHUTDOWN TO REPAIR MAIN GENERATOR STATOR COOLANT LEAK AND VALVE IN LETDOWN SYSTEM.

SUMMARY

MILLSTONE 3 OPERATED WITH 1 REDUCTION AND SHUTDOWN FOR REPAIRS ON JULY 24TH.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* MILLSTONE 3 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....CONNECTICUT

COUNTY.....NEW LONDON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...3.2 MI WSW OF
NEW LONDON CT.

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JANUARY 23, 1986

DATE ELEC ENER 1ST GENER...FEBRUARY 12, 1986

DATE COMMERCIAL OPERATE....APRIL 23, 1986

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....NIANTIC BAY

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NORTHEAST NUCLEAR ENERGY

CORPORATE ADDRESS.....P.O. BOX 270
HARTFORD, CONNECTICUT 06101

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESILENT INSPECTOR.....T. REBELOWSKI

LICENSING PROJ MANAGER.....E. DOOLITTLE
DOCKET NUMBER.....50-423

LICENSE & DATE ISSUANCE....NPF-49, JANUARY 31, 1986

PUBLIC DOCUMENT ROOM.....WATERFORD PUBLIC LIBRARY
49 ROPE FERRY ROAD
WATERFORD, CONNECTICUT 06385

INSPECTION STATUS

INSPECTION SUMMARY

INFO. NOT SUPPLIED BY REGION

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

INFO. NOT SUPPLIED BY REGION

FACILITY ITEMS (PLANS AND PROCEDURES):

INFO. NOT SUPPLIED BY REGION

MANAGERIAL ITEMS:

INFO. NOT SUPPLIED BY REGION

I N S P E C T I O N S T A T U S - (CONTINUED)

****
* MILLSTONE 3 *
****

PLANT STATUS:

INFO. NOT SUPPLIED BY REGION

LAST IE SITE INSPECTION DATE: INFO. NOT SUPPLIED BY REGION

INSPECTION REPORT NO: INFO. NOT SUPPLIED BY REGION

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
INFO. NOT SUPPLIED BY REGION			

1. Docket: 50-263 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: A. L. Myrabo (612) 295-5151
4. Licensed Thermal Power (MWt): 1670
5. Nameplate Rating (Gross MWe): 632 X 0.9 = 569
6. Design Electrical Rating (Net MWe): 545
7. Maximum Dependable Capacity (Gross MWe): 564
8. Maximum Dependable Capacity (Net MWe): 536
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	132,240.0
13. Hours Reactor Critical	485.0	3,354.6	101,433.1
14. Rx Reserve Shtdwn Hrs	.0	.0	940.7
15. Hrs Generator On-Line	442.2	3,309.9	99,343.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	642,029	4,561,363	158,854,873
18. Gross Elec Ener (MWH)	215,959	1,549,567	51,200,844
19. Net Elec Ener (MWH)	203,211	1,475,967	48,938,378
20. Unit Service Factor	59.4	65.1	75.1
21. Unit Avail Factor	59.4	65.1	75.1
22. Unit Cap Factor (MDC Net)	51.0	54.1	69.0
23. Unit Cap Factor (DER Net)	50.1	53.2	67.9
24. Unit Forced Outage Rate	.0	.0	4.8
25. Forced Outage Hours	.0	.0	1,335.0

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

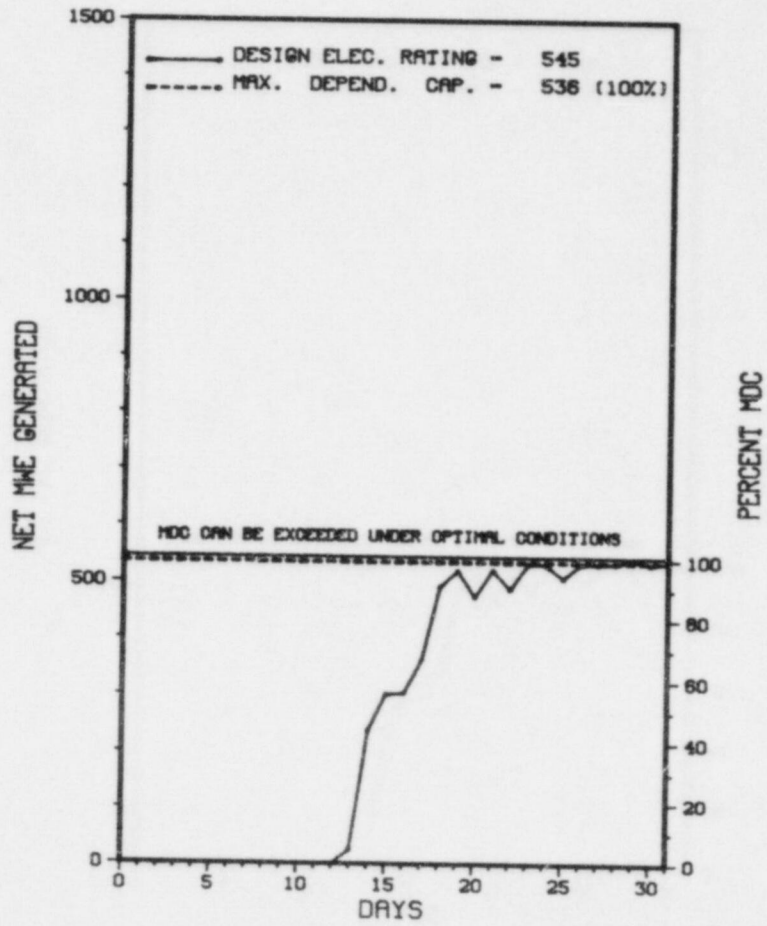
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* MONTICELLO *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

MONTICELLO



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

MONTICELLO

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	04/30/86	S	288.8	C	4		Z		CONTINUATION OF THE 1986 REFUELING OUTAGE.
2	07/13/86	S	13.0	B	1		HA	TURBIN	GENERATOR TAKEN OFF-LINE FOR TURBINE OVERSPEED TEST.

SUMMARY

MONTICELLO COMPLETED A REFUELING OUTAGE ON JULY 13.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* MONTICELLO *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....MINNESOTA

COUNTY.....WRIGHT

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...30 MI NW OF
MINNEAPOLIS, MINN

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...DECEMBER 10, 1970

DATE ELEC ENER 1ST GENER...MARCH 5, 1971

DATE COMMERCIAL OPERATE....JUNE 30, 1971

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NORTHERN STATES POWER

CORPORATE ADDRESS.....414 NICOLLET MALL
MINNEAPOLIS, MINNESOTA 55401

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....P. HARTMAN

LICENSING PROJ MANAGER.....D. LYNCH
DOCKET NUMBER.....50-263

LICENSE & DATE ISSUANCE....DPR-22, JANUARY 9, 1981

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MINNEAPOLIS PUBLIC LIBRARY
300 NICOLLET MALL
MINNEAPOLIS, MINNESOTA 55401

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON JUNE 30 (86005): ROUTINE UNANNOUNCED INSPECTION OF THE RESOLUTION OF AN IE BULLETIN. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON JUNE 23-25 (86006): SPECIAL, ANNOUNCED INSPECTION BY REGIONAL INSPECTORS OF THE CAUSE(S) WHICH CREATED THE INTERMEDIATE RANGE MONITOR (IRMS) INOPERABILITY EVENT OF JUNE 14, 1986; THE ADEQUACY OF PLANT PROCEDURES TO ENSURE SAFE OPERATION HAD THE EVENT OCCURRED DURING UNIT STARTUP OR SHUTDOWN; AND THE CORRECTIVE ACTIONS TAKEN TO PREVENT RECURRENCE. THE INSPECTION WAS CONDUCTED IN ACCORDANCE WITH NRC INSPECTION PROCEDURE 93702. OF THE THREE AREAS INSPECTED NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS. TWO VIOLATIONS WERE IDENTIFIED IN THE REMAINING AREA (FAILURE TO FOLLOW PROCEDURES AND TECHNICAL SPECIFICATION LCO 3.1.A AND 3.2.C)

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

OPERATING ROUTINELY.

LAST IE SITE INSPECTION DATE: 06/25/86

INSPECTION REPORT NO: 86006

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-15	06/03/86	07/03/86	RHR SHUTDOWN COOLING INTERLOCK TRIP
86-16	06/11/86	07/11/86	STANDBY LIQUID CONTROL SYSTEM IMPROPER FUSE COORDINATION
86-17	06/15/86	07/15/86	ECCS INITIATION DURING REACTOR VESSEL WATER LEVEL INSTRUMENTATION INSTALLATION
86-16	06/16/86	07/16/86	INOPERABLE INTERMEDIATE RANGE MONITORS #15, 16 AND 17
86-19	06/17/86	07/17/86	REACTOR PROTECTION SYSTEM INITIATION
86-20	06/19/86	07/18/86	RPS AND GROUP II TRIP FROM LOSS OF POWER
86-21	06/20/86	07/18/86	ESF ACTUATIONS FROM WRGM TRIPS
86-22	06/23/86	07/18/86	RBV ISOLATION, CONTAINMENT GROUP II ISOLATION, AND SBGTS INITIATION DURING SWITCHING
86-23	07/03/86	08/01/86	LPCI LOOP SELECTION DESIGN PROBLEMS

1. Docket: 50-220 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: THOMAS W. ROMAN (315) 349-2422
4. Licensed Thermal Power (MWt): 1850
5. Nameplate Rating (Gross MWe): 755 X 0.85 = 642
6. Design Electrical Rating (Net MWe): 620
7. Maximum Dependable Capacity (Gross MWe): 630
8. Maximum Dependable Capacity (Net MWe): 610
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	146,807.0
13. Hours Reactor Critical	608.8	2,336.9	103,577.4
14. Rx Reserve Shtdwn Hrs	.0	.0	1,204.2
15. Hrs Generator On-Line	590.3	2,254.4	100,502.4
16. Unit Reserve Shtdwn Hrs	.0	.0	20.2
17. Gross Therm Ener (MWH)	974,468	3,400,222	167,895,897
18. Gross Elec Ener (MWH)	319,537	1,117,479	55,586,935
19. Net Elec Ener (MWH)	308,401	1,076,549	53,838,876
20. Unit Service Factor	79.3	44.3	68.5
21. Unit Avail Factor	79.3	44.3	68.5
22. Unit Cap Factor (MDC Net)	68.0	34.7	60.1
23. Unit Cap Factor (DER Net)	66.9	34.1	59.2
24. Unit Forced Outage Rate	20.7	12.5	15.4
25. Forced Outage Hours	153.7	322.6	13,699.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

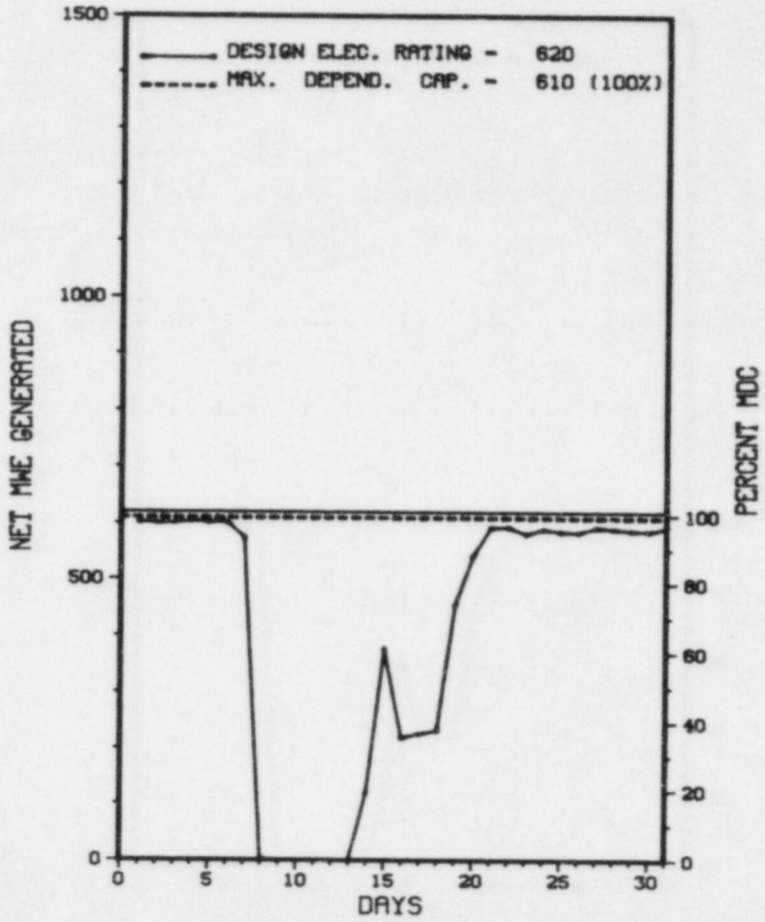
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* NINE MILE POINT 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

NINE MILE POINT 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* NINE MILE POINT 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
2	07/08/86	F	153.7	A	1				CONTROL ROD DRIVE PUMP #11 HAD FAILED AND POSSIBLE FAILURE TO CONTROL ROD DRIVE PUMP #12.

****
* SUMMARY *
****

NINE MILE POINT 1 HAD 1 SHUTDOWN IN JULY FOR EQUIPMENT FAILURE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* NINE MILE POINT 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....NEW YORK

COUNTY.....OSWEGO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...8 MI NE OF
OSWEGO, NY

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...SEPTEMBER 5, 1969

DATE ELEC ENER 1ST GENER...NOVEMBER 9, 1969

DATE COMMERCIAL OPERATE....DECEMBER 1, 1969

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE ONTARIO

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NIAGARA MOHAWK POWER CORP.

CORPORATE ADDRESS.....300 ERIE BOULEVARD WEST
SYRACUSE, NEW YORK 13202

CONTRACTOR
ARCHITECT/ENGINEER.....NIAGARA MOHAWK POWER CORP.

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....S. HUDSON

LICENSING PROJ MANAGER.....J. KELLY
DOCKET NUMBER.....50-220

LICENSE & DATE ISSUANCE....DPR-63, DECEMBER 26, 1974

PUBLIC DOCUMENT ROOM.....STATE UNIVERSITY COLLEGE OF OSWEGO
PENFIELD LIBRARY - DOCUMENTS
OSWEGO, NY 13126
(315) 341-2323

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATIONS, SECTION 6.12.1, REQUIRES IN PART THAT EACH HIGH RADIATION AREA NORMALLY ACCESSIBLE BY PERSONNEL IN WHICH THE INTENSITY OF RADIATION IS GREATER THAN 100 MREM/HR BUT LESS THAN 1000 MREM/HR SHALL BE BARRICADED AND CONSPICUOUSLY POSTED AS A HIGH RADIATION AREA. CONTRARY TO THE ABOVE, ON MARCH 7, 1986, A HIGH RADIATION AREA ACCESSIBLE TO PERSONNEL AND MEASURING UP TO 400 MREM/HR, ON THE TURBINE DECK, 300 FT. ELEVATION, WAS LEFT NOT BARRICADED AND NOT POSTED.
(8600 4)

OTHER ITEMS

SYSTEMS AND COMPONENTS:

OTHER ITEMS

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-338 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: B. GARNER (703) 894-5151 X2527
4. Licensed Thermal Power (MWt): 2775
5. Nameplate Rating (Gross MWe): 947
6. Design Electrical Rating (Net MWe): 907
7. Maximum Dependable Capacity (Gross MWe): 941
8. Maximum Dependable Capacity (Net MWe): 893
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

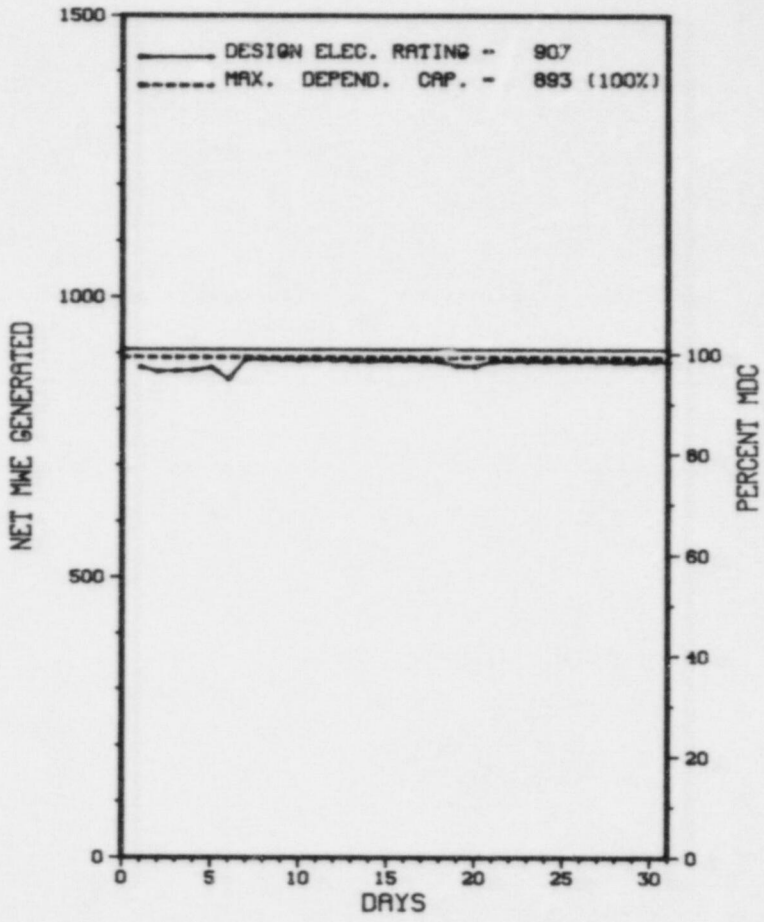
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	71,472.0
13. Hours Reactor Critical	744.0	4,619.4	49,905.2
14. Rx Reserve Shtdwn Hrs	.0	467.6	4,474.2
15. Hrs Generator On-Line	744.0	4,524.9	48,437.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,062,274	12,263,735	127,240,287
18. Gross Elec Ener (MWH)	692,430	4,124,805	41,602,431
19. Net Elec Ener (MWH)	657,005	3,916,296	39,331,244
20. Unit Service Factor	100.0	89.0	67.8
21. Unit Avail Factor	100.0	89.0	67.8
22. Unit Cap Factor (MDC Net)	98.9	86.2	61.6
23. Unit Cap Factor (DER Net)	97.4	84.9	60.7
24. Unit Forced Outage Rate	.0	11.0	12.1
25. Forced Outage Hours	.0	558.1	6,542.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: N/A

****
* NORTH ANNA 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

NORTH ANNA 1



JULY 1986

Report Period JUL 1986 UNIT SHUTDOWNS / REDUCTIONS

****
* NORTH ANNA 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence

NONE

****
* SUMMARY *
****

NORTH ANNA 1 OPERATED ROUTINELY IN JULY WITH NO OUTAGES OR POWER REDUCTIONS REPORTED.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* NORTH ANNA 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....VIRGINIA

COUNTY.....LOUISA

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...40 MI NW OF
RICHMOND, VA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...APRIL 5, 1978

DATE ELEC ENER 1ST GENER...APRIL 17, 1978

DATE COMMERCIAL OPERATE....JUNE 6, 1978

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE ANNA

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....VIRGINIA POWER

CORPORATE ADDRESS.....P.O. BOX 26666
RICHMOND, VIRGINIA 23261

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....M. BRANCH

LICENSING PROJ MANAGER.....L. ENGLE
DOCKET NUMBER.....50-338

LICENSE & DATE ISSUANCE....NPF-4, APRIL 1, 1978

PUBLIC DOCUMENT ROOM.....ALDERMAN LIBRARY/MANUSCRIPTS DEPT.
UNIV. OF VIRGINIA/CHARLOTTESVILLE VA 22901

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MAY 5 - JUNE 1 (86-13): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED THE AREAS OF LICENSEE EVENT REPORT (LER) REVIEW, ENGINEERING SAFETY FEATURES (ESF) WALKDOWN, OPERATIONAL SAFETY VERIFICATION, MONTHLY MAINTENANCE, MONTHLY SURVEILLANCE AND SAFEGUARDS. ONE VIOLATION WAS IDENTIFIED - FAILURE TO FOLLOW DESIGN CHANGE PROCEDURES FOR THE BATTERY INSTALLATION.

INSPECTION JUNE 17-19 (86-16): THIS WAS A ROUTINE, ANNOUNCED INSPECTION OF THE LICENSEE'S PERFORMANCE AND CAPABILITIES DURING AN EXERCISE OF THE EMERGENCY PLAN AND PROCEDURES. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 2 - JULY 6 (86-17): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTORS INVOLVED THE FOLLOWING AREAS: PLANT STATUS, INSPECTOR FOLLOW-UP AND UNRESOLVED ITEMS, LICENSEE EVENT REPORT (LER) FOLLOWUP, ROUTINE INSPECTION, MONTHLY MAINTENANCE OBSERVATION, MONTHLY SURVEILLANCE OBSERVATION, ESF SYSTEM WALKDOWN, AND POTENTIAL GENERIC ITEMS. TWO VIOLATIONS WERE IDENTIFIED - FAILURE TO REQUIRE A REVIEW OF DESIGN CHANGE ACCEPTANCE TESTS FOR TECHNICAL SPECIFICATION REQUIREMENTS AND FAILURE TO PERFORM A WRITTEN SAFETY EVALUATION.

ENFORCEMENT SUMMARY

FAILURE TO CONTROL PROTECTED AREA ACCESS.
(8600 3)

ENFORCEMENT SUMMARY

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 2 - JULY 6, 1986 +

INSPECTION REPORT NO: 50-338/86-17 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-012	06/10/86	07/08/86	FIRE DETECTION SYSTEM OUT-OF-SERVICE GREATER THAN 14 DAYS; CAUSE - LACK OF COORDINATION BETWEEN CONSTRUCTION PLANNING AND STATION PLANNING.

1. Docket: 50-339 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: B. GARNER (703) 894-5151 X2527
4. Licensed Thermal Power (MWt): 2775
5. Nameplate Rating (Gross MWe): 947
6. Design Electrical Rating (Net MWe): 907
7. Maximum Dependable Capacity (Gross MWe): 941
8. Maximum Dependable Capacity (Net MWe): 893
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	49,343.0
13. Hours Reactor Critical	744.0	3,978.7	38,296.0
14. Rx Reserve Shtdwn Hrs	.0	1,108.3	3,680.4
15. Hrs Generator On-Line	744.0	3,896.2	37,328.3
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,979,991	10,039,481	97,188,870
18. Gross Elec Ener (MWH)	650,235	3,329,605	32,231,787
19. Net Elec Ener (MWH)	615,238	3,153,752	30,536,612
20. Unit Service Factor	100.0	76.6	75.7
21. Unit Avail Factor	100.0	76.6	75.7
22. Unit Cap Factor (MDC Net)	92.6	69.4	69.3
23. Unit Cap Factor (DER Net)	91.2	68.4	68.2
24. Unit Forced Outage Rate	.0	4.3	10.7
25. Forced Outage Hours	.0	176.5	4,456.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

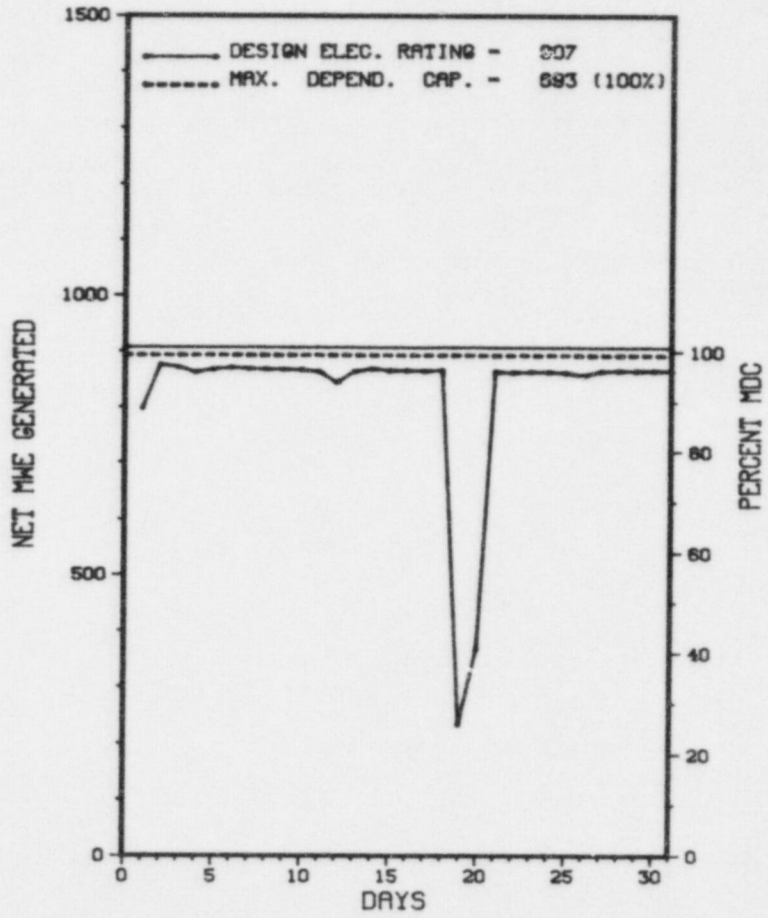
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* NORTH ANNA 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

NORTH ANNA 2



JULY 1986

Report Period JUL 1986 UNIT SHUTDOWNS / REDUCTIONS

****
* NORTH ANNA 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-07	07/19/86	S	0.0	H	5				RAMPED UNIT DOWN TO 30% POWER FOR REPAIRS ON MAIN FEED REGULATOR VALVE. REPAIRS WERE COMPLETED AND UNIT RETURNED TO 100 POWER.

****
* SUMMARY *
****

NORTH ANNA 2 OPERATED ROUTINELY IN JULY WITH NO OUTAGES AND 1 POWER REDUCTION REPORTED.

Type	Reason	Method	System & Component
F-Forced	A-Equip Failure F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction	4-Continued	Data Entry Sheet
	E-Operator Training	5-Reduced Load	Licensee Event Report
	& License Examination	9-Other	(LER) File (NUREG-0161)

****
* NORTH ANNA 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....VIRGINIA

COUNTY.....LOUISA

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...40 MI NW OF
RICHMOND, VA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JUNE 12, 1980

DATE ELEC ENER 1ST GENER...AUGUST 25, 1980

DATE COMMERCIAL OPERATE....DECEMBER 14, 1980

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE ANNA

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....VIRGINIA POWER

CORPORATE ADDRESS.....P.O. BOX 26666
RICHMOND, VIRGINIA 23261

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....M. BRANCH

LICENSING PROJ MANAGER.....L. ENGLE
DOCKET NUMBER.....50-339

LICENSE & DATE ISSUANCE....NPF-7, AUGUST 21, 1980

PUBLIC DOCUMENT ROOM.....ALDERMAN LIBRARY/MANUSCRIPTS DEPT.
UNIV. OF VIRGINIA/CHARLOTTESVILLE VA 22901

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MAY 5 - JUNE 1 (86-13): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED THE AREAS OF LICENSEE EVENT REPORT (LER) REVIEW, ENGINEERING SAFETY FEATURES (ESF) WALKDOWN, OPERATIONAL SAFETY VERIFICATION, MONTHLY MAINTENANCE, MONTHLY SURVEILLANCE AND SAFEGUARDS. ONE VIOLATION WAS IDENTIFIED - FAILURE TO FOLLOW DESIGN CHANGE PROCEDURES FOR THE BATTERY INSTALLATION.

INSPECTION JUNE 17-19 (86-16): THIS WAS A ROUTINE, ANNOUNCED INSPECTION OF THE LICENSEE'S PERFORMANCE AND CAPABILITIES DURING AN EXERCISE OF THE EMERGENCY PLAN AND PROCEDURES. OF THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 2 - JULY 6 (86-17): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTORS INVOLVED THE FOLLOWING AREAS: PLANT STATUS, INSPECTOR FOLLOW-UP AND UNRESOLVED ITEMS, LICENSEE EVENT REPORT (LER) FOLLOWUP, ROUTINE INSPECTION, MONTHLY MAINTENANCE OBSERVATION, MONTHLY SURVEILLANCE OBSERVATION, ESF SYSTEM WALKDOWN, AND POTENTIAL GENERIC ITEMS. TWO VIOLATIONS WERE IDENTIFIED FAILURE TO REQUIRE A REVIEW OF DESIGN CHANGE ACCEPTANCE TESTS FOR TECHNICAL SPECIFICATION REQUIREMENTS AND FAILURE TO PERFORM A WRITTEN SAFETY EVALUATION.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

DIESEL GENERATOR RELIABILITY PROBLEM - LICENSEE IS INVESTIGATING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 2 - JULY 6, 1986 +

INSPECTION REPORT NO: 50-339/86-17 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-006	04/16/86	05/15/86	SPECIFIC GRAVITIES OF VARIOUS BATTERY CELLS ON THREE OF FOUR DC BUSSES BELOW TECHNICAL SPECIFICATION, UNIT SHUTDOWN.
86-007	04/16/86	05/09/86	REACTOR TRIP CAUSED BY TURBINE FIRST STAGE PRESSURE SPIKE; DUE TO THROTTLE STOP VALVE LIMIT SWITCHES INDICATING A CLOSED POSITION ON ALL FOUR VALVES.

1. Docket: 50-269 O P E R A T I N G S T A T U S

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: J. A. REAVIS (704) 373-7567

4. Licensed Thermal Power (MWt): 2568

5. Nameplate Rating (Gross MWe): 1038 X 0.9 = 934

6. Design Electrical Rating (Net MWe): 887

7. Maximum Dependable Capacity (Gross MWe): 899

8. Maximum Dependable Capacity (Net MWe): 860

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	114,336.0
13. Hours Reactor Critical	744.0	2,918.9	83,365.6
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	744.0	2,859.6	79,989.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,911,216	7,119,737	193,419,937
18. Gross Elec Ener (MWH)	656,210	2,463,680	67,210,520
19. Net Elec Ener (MWH)	625,931	2,331,825	63,730,074
20. Unit Service Factor	100.0	56.2	70.0
21. Unit Avail Factor	100.0	56.2	70.0
22. Unit Cap Factor (MDC Net)	97.8	53.3	64.7*
23. Unit Cap Factor (DER Net)	94.8	51.7	62.9*
24. Unit Forced Outage Rate	.0	9.0	14.7
25. Forced Outage Hours	.0	283.8	12,823.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

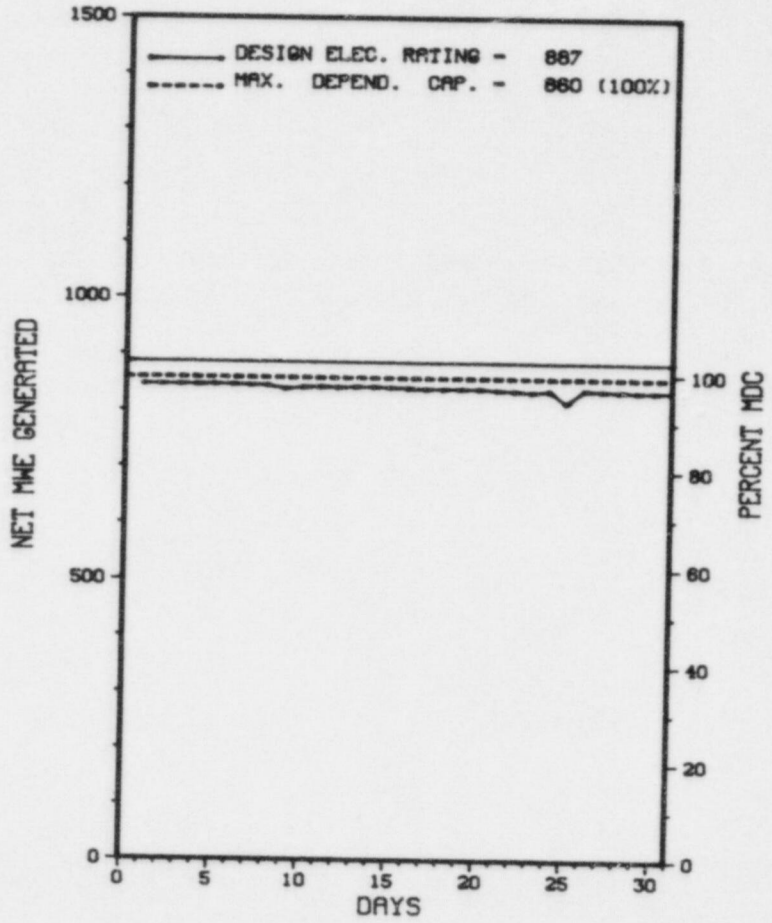
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* OCONEE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

OCONEE 1



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* OCONEE 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
32-P	07/25/86	S	0.0	B	5		CC	VALVEX	TURBINE CONTROL & STOP VALVE MOVEMENT PT'S.
33-P	07/25/86	F	0.0	A	5		CA	INSTRU	CHANGE OUT BAD DIODE ON ROD GROUP 6.

****
* SUMMARY *
****

OCONEE 1 HAD NO SHUTDOWNS AND 2 POWER REDUCTIONS IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* OCONEE 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....SOUTH CAROLINA

COUNTY.....OCONEE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...30 MI W OF
GREENVILLE, SC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...APRIL 19, 1973

DATE ELEC ENER 1ST GENER...MAY 6, 1973

DATE COMMERCIAL OPERATE....JULY 15, 1973

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE KEOWEE

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUKE POWER

CORPORATE ADDRESS.....422 SOUTH CHURCH STREET
CHARLOTTE, NORTH CAROLINA 28242

CONTRACTOR
ARCHITECT/ENGINEER.....DUKE & BECHTEL

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....DUKE POWER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....J. BRYANT

LICENSING PROJ MANAGER.....H. PASTIS
DOCKET NUMBER.....50-269

LICENSE & DATE ISSUANCE....DPR-38, FEBRUARY 6, 1973

PUBLIC DOCUMENT ROOM.....OCONEE COUNTY LIBRARY
501 W. SOUTH BROAD ST.
WALHALLA, SOUTH CAROLINA 29691

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MAY 20 - JUNE 9 (86-18): THIS ROUTINE, ANNOUNCED INSPECTION WAS CONDUCTED ONSITE IN THE AREAS OF OPERATIONS, SURVEILLANCE, MAINTENANCE, VERIFICATION OF ENGINEERED SAFETY FEATURES LINEUPS, FOLLOWUP OF EVENTS, AND FOLLOWUP OF UNRESOLVED AND INSPECTOR FOLLOWUP ITEMS. OF THE SIX AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: VIOLATION 269/86-18-02, FAILURE TO FOLLOW PROCEDURE DURING VALVE SURVEILLANCE.

INSPECTION JUNE 23-26 (86-19): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED ONSITE REVIEW OF QUALITY CONTROL AND CONFIRMATORY MEASUREMENTS INCLUDING: REVIEW OF THE LABORATORY QUALITY PROGRAM; REVIEW OF CHEMICAL AND RADIOCHEMICAL PROCEDURES; REVIEW OF QUALITY CONTROL RECORDS AND LOGS; AND COMPARISON OF THE RESULTS OF SPLIT SAMPLES ANALYZED BY THE LICENSEE AND NRC REGION II MOBILE LABORATORY. IN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

CONTRARY TO TS 6.4.1, ON APRIL 4, 1986, DURING SURVEILLANCE TESTING ON VALVE 1LP-21, THE TECHNICIAN INACCURATELY SIGNED THE PROCEDURE STEP STATING THAT ACCEPTANCE CRITERIA HAD BEEN MET; WHEREAS, IN FACT THE VALVE HAD NOT PASSED ITS EXERCISE TEST. THE VALVE REQUIRED 16 SECONDS TO OPEN WHILE THE MAXIMUM STROKE TIME ALLOWED BY THE PROCEDURE IS 15 SECONDS. THE COMPLETED TEST WAS SUBSEQUENTLY REVIEWED AND APPROVED. WHILE THE VALVE WAS TECHNICALLY INOPERABLE DURING THE PERIOD APRIL 4 UNTIL IT PASSED A SURVEILLANCE TEST ON JUNE 4, THE OPERABILITY OF THE LOW PRESSURE INJECTION SYSTEM WAS NOT IN QUESTION SINCE THE VALVE IS OPEN FOR

ENFORCEMENT SUMMARY

NORMAL OPERATION. THE VALVE WOULD BE REQUIRED TO BE OPEN FOLLOWING ENGINEERED SAFETY FEATURES ACTUATION. (8601 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

POWER OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 23-26, 1986 +

INSPECTION REPORT NO: 50-269/86-19 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-006	04/10/86	05/12/86	END OF CYCLE HOT FULL POWER LOWER MODERATOR TEMPERATURE COEFFICIENT; CAUSE - A DISPARITY BETWEEN MEASURED DATA AND CALCULATED VALVE TABULATED IN O1C9 RELOAD REPORT.
86-007	05/02/86	06/02/86	TWO CHANNELS OF REACTOR PROTECTIVE SYSTEM INOPERABLE AT THE SAME TIME DURING ADJUSTMENT OF HI-FLUX TRIP SETPOINT; CAUSE - FAILURE TO PROPERLY TAG INOPERABLE MODULE.
86-009	06/13/86	07/14/86	KEOWEE BATTERY RACKS OUTSIDE OF SEISMIC DESIGN BASIS; CAUSE - POOR COMMUNICATION BETWEEN CLEARANCE BETWEEN END CELL AND STRINGER.

1. Docket: 50-270 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: J. A. REAVIS (704) 373-7567

4. Licensed Thermal Power (MWt): 2568

5. Nameplate Rating (Gross MWe): 1038 X 0.9 = 934

6. Design Electrical Rating (Net MWe): 887

7. Maximum Dependable Capacity (Gross MWe): 899

8. Maximum Dependable Capacity (Net MWe): 860

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	104,256.0
13. Hours Reactor Critical	736.0	5,071.0	77,909.1
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	725.8	5,047.7	76,649.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,841,568	12,871,231	183,275,133
18. Gross Elec Ener (MWH)	616,060	4,365,530	62,417,871
19. Net Elec Ener (MWH)	586,641	4,175,925	59,343,486
20. Unit Service Factor	97.6	99.2	73.5
21. Unit Avail Factor	97.6	99.2	73.5
22. Unit Cap Factor (MDC Net)	91.7	95.5	66.0*
23. Unit Cap Factor (DER Net)	88.9	92.5	64.2*
24. Unit Forced Outage Rate	2.4	.8	13.1
25. Forced Outage Hours	18.2	39.3	10,713.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

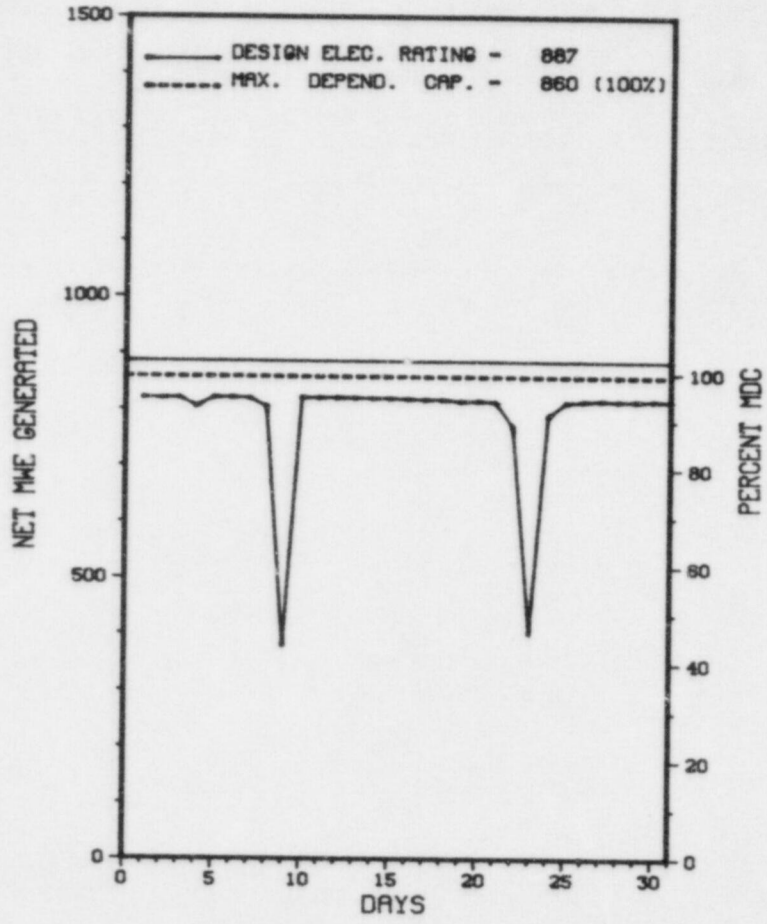
REFUELING - AUGUST 14, 1986 - 9 WEEKS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* OCONEE 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

OCONEE 2



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

OCONEE 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
34-P	07/01/86	F	0.0	A	5		HB	HTEXCH	SECOND STAGE REHEATER VALVED OUT.
35-P	07/04/86	S	0.0	B	5		CC	VALVEX	CONTROL VALVE & STOP VALVE MOVEMENT PT'S.
36-P	07/04/86	F	0.0	A	5		HB	HTEXCH	SECOND STAGE REHEATER VALVED OUT.
3	07/08/86	F	9.6	A	3		IF	INSTRU	REACTOR, TURBINE & FEEDWATER PUMP TRIP ON STEAM GENERATOR "2B" HIGH LEVEL.
37-P	07/09/86	S	0.0	B	5		SF	PUMPXX	HOLDING POWER DUE TO HIGH PRESSURE INJECTION PUMP "2B" PERFORMANCE TESTING.
38-P	07/09/86	F	0.0	A	5		HB	HTEXCH	SECOND STAGE REHEATER VALVED OUT.
39-P	07/22/86	F	0.0	A	5		HH	PUMPXX	REDUCED REACTOR POWER DUE TO "2D2" HEATER DRAIN PUMP OIL LEAK.
40-P	07/22/86	F	0.0	A	5		HB	HTEXCH	SECOND STAGE REHEATER VALVED OUT.
4	07/23/86	F	8.6	A	3		CB	INSTRU	REACTOR/TURBINE TRIP FOLLOWING LOSS OF "A" REACTOR LOOP FLOW TRANSMITTER.
41-P	07/24/86	F	0.0	A	5		HH	VALVEX	HOLDING POWER DUE TO OPENING "2D1" HEATER DRAIN PUMP DISCHARGE CHECK VALVE.
42-P	07/24/86	F	0.0	A	5		HB	HTEXCH	SECOND STAGE REHEATER VALVED OUT.

SUMMARY

OCONEE 2 INCURRED 2 SHUTDOWNS AND SEVERAL POWER REDUCTIONS IN JULY AS DISCUSSED ABOVE.

Type
F-Forced
S-Sched

Reason
A-Equip Failure
B-Maint or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Admin
G-Oper Error
H-Other

Method
1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other

System & Component
Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* OCONEE 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....SOUTH CAROLINA

COUNTY.....OCONEE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...30 MI W OF
GREENVILLE, SC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...NOVEMBER 11, 1973

DATE ELEC ENER 1ST GENER...DECEMBER 5, 1973

DATE COMMERCIAL OPERATE....SEPTEMBER 9, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE KEOWEE

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUKE POWER

CORPORATE ADDRESS.....422 SOUTH CHURCH STREET
CHARLOTTE, NORTH CAROLINA 28242

CONTRACTOR
ARCHITECT/ENGINEER.....DUKE & BECHTEL

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....DUKE POWER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....J. BRYANT

LICENSING PROJ MANAGER.....H. PASTIS
DOCKET NUMBER.....50-270

LICENSE & DATE ISSUANCE....DPR-47, OCTOBER 6, 1973

PUBLIC DOCUMENT ROOM.....OCONEE COUNTY LIBRARY
501 W. SOUTH BROAD ST.
WALHALLA, SOUTH CAROLINA 29691

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MAY 20 - JUNE 9 (86-13): THIS ROUTINE, ANNOUNCED INSPECTION WAS CONDUCTED ONSITE IN THE AREAS OF OPERATIONS, SURVEILLANCE, MAINTENANCE, VERIFICATION OF ENGINEERED SAFETY FEATURES LINEUPS, FOLLOWUP OF EVENTS, AND FOLLOWUP OF UNRESOLVED AND INSPECTOR FOLLOWUP ITEMS. OF THE SIX AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: VIOLATION 269/86-18-02, FAILURE TO FOLLOW PROCEDURE DURING VALVE SURVEILLANCE.

INSPECTION JUNE 23-26 (86-19): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED ONSITE REVIEW OF QUALITY CONTROL AND CONFIRMATORY MEASUREMENTS INCLUDING: REVIEW OF THE LABORATORY QUALITY PROGRAM; REVIEW OF CHEMICAL AND RADIOCHEMICAL PROCEDURES; REVIEW OF QUALITY CONTROL RECORDS AND LOGS; AND COMPARISON OF THE RESULTS OF SPLIT SAMPLES ANALYZED BY THE LICENSEE AND NRC REGION II MOBILE LABORATORY. IN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

POWER OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 23-26, 1986 +

INSPECTION REPORT NO: 50-270/86-19 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

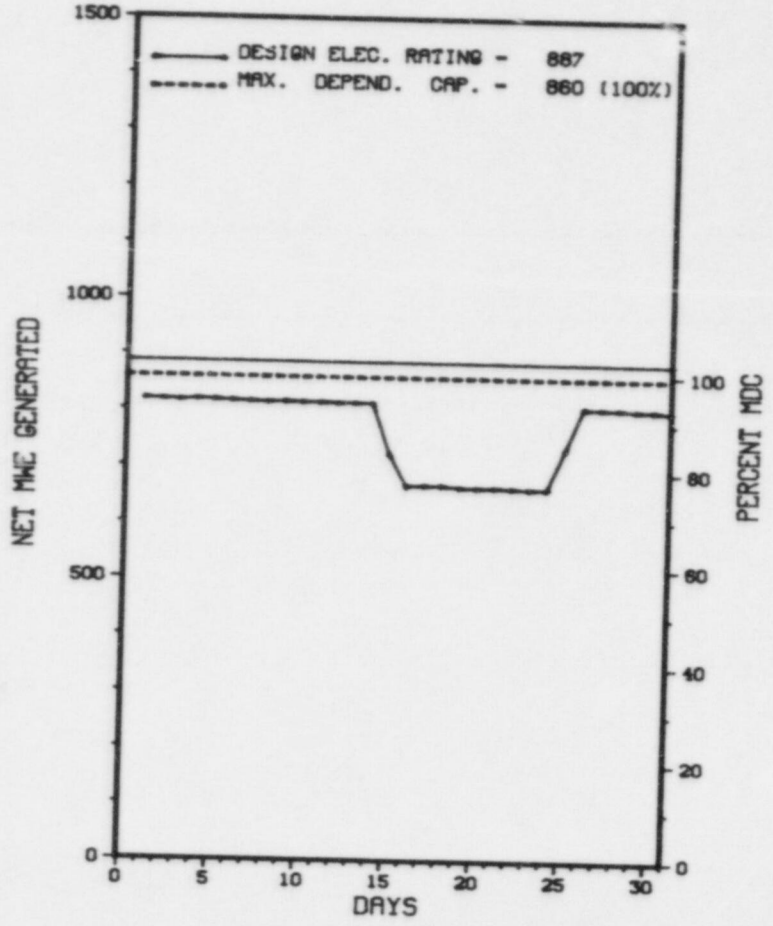
1. Docket: 50-287 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: J. A. REAVIS (704) 373-7567
4. Licensed Thermal Power (MWt): 2568
5. Nameplate Rating (Gross MWe): 1038 X 0.9 = 934
6. Design Electrical Rating (Net MWe): 887
7. Maximum Dependable Capacity (Gross MWe): 899
8. Maximum Dependable Capacity (Net MWe): 860
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	101,903.0
13. Hours Reactor Critical	744.0	5,083.8	74,455.3
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	744.0	5,063.3	73,196.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,807,056	12,732,560	179,401,005
18. Gross Elec Ener (MWH)	597,760	4,359,380	61,887,434
19. Net Elec Ener (MWH)	569,299	4,173,925	58,953,282
20. Unit Service Factor	100.0	99.5	71.8
21. Unit Avail Factor	100.0	99.5	71.8
22. Unit Cap Factor (MDC Net)	89.0	95.4	67.1*
23. Unit Cap Factor (DER Net)	86.3	92.5	65.3*
24. Unit Forced Outage Rate	.0	.5	13.7
25. Forced Outage Hours	.0	23.7	11,789.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: N/A

****
* OCONEE 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT
OCONEE 3



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* OCONEE 3 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
20-P	07/15/86	F	0.0	A	5		HH	PUMPXX	"3D2" HEATER DRAIN PUMP REPAIR.

****
* SUMMARY *
****

OCONEE 3 HAD NO OUTAGES AND 1 POWER REDUCTION IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* OCONEE 3 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....SOUTH CAROLINA

COUNTY.....OCONEE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...30 MI W OF
GREENVILLE, SC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...SEPTEMBER 5, 1974

DATE ELEC ENER 1ST GENER...SEPTEMBER 18, 1974

DATE COMMERCIAL OPERATE....DECEMBER 16, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE KEOWEE

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....DUKE POWER

CORPORATE ADDRESS.....422 SOUTH CHURCH STREET
CHARLOTTE, NORTH CAROLINA 28242

CONTRACTOR
ARCHITECT/ENGINEER.....DUKE & BECHTEL

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....DUKE POWER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....J. BRYANT

LICENSING PROJ MANAGER.....H. PASTIS
DOCKET NUMBER.....50-287

LICENSE & DATE ISSUANCE....DPR-55, JULY 19, 1974

PUBLIC DOCUMENT ROOM.....OCONEE COUNTY LIBRARY
501 W. SOUTH BROAD ST.
WALHALLA, SOUTH CAROLINA 29691

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MAY 20 - JUNE 9 (86-18): THIS ROUTINE, ANNOUNCED INSPECTION WAS CONDUCTED ONSITE IN THE AREAS OF OPERATIONS, SURVEILLANCE, MAINTENANCE, VERIFICATION OF ENGINEERED SAFETY FEATURES LINEUPS, FOLLOWUP OF EVENTS, AND FOLLOWUP OF UNRESOLVED AND INSPECTOR FOLLOWUP ITEMS. OF THE SIX AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: VIOLATION 269/86-18-02, FAILURE TO FOLLOW PROCEDURE DURING VALVE SURVEILLANCE.

INSPECTION JUNE 23-26 (86-19): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED ONSITE REVIEW OF QUALITY CONTROL AND CONFIRMATORY MEASUREMENTS INCLUDING: REVIEW OF THE LABORATORY QUALITY PROGRAM; REVIEW OF CHEMICAL AND RADIOCHEMICAL PROCEDURES; REVIEW OF QUALITY CONTROL RECORDS AND LOGS; AND COMPARISON OF THE RESULTS OF SPLIT SAMPLES ANALYZED BY THE LICENSEE AND NRC REGION II MOBILE LABORATORY. IN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

****
* OCONEE 3 *
****

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

POWER OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 23-26, 1986 +

INSPECTION REPORT NO: 50-287/86-19 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-219 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: JOSEPH R. MOLNAR (609) 971-4699

4. Licensed Thermal Power (MWt): 1930

5. Nameplate Rating (Gross MWe): 722 X .9 = 650

6. Design Electrical Rating (Net MWe): 650

7. Maximum Dependable Capacity (Gross MWe): 650

8. Maximum Dependable Capacity (Net MWe): 620

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	145,559.0
13. Hours Reactor Critical	.0	2,311.5	95,453.9
14. Rx Reserve Shtdwn Hrs	.0	448.5	1,208.0
15. Hrs Generator On-Line	.0	2,273.9	92,332.0
16. Unit Reserve Shtdwn Hrs	.0	452.8	1,761.4
17. Gross Therm Ener (MWH)	0	4,074,338	153,028,338
18. Gross Elec Ener (MWH)	0	1,371,480	51,662,165
19. Net Elec Ener (MWH)	-1,840	1,311,153	49,621,646
20. Unit Service Factor	.0	44.7	63.4
21. Unit Avail Factor	.0	53.6	64.6
22. Unit Cap Factor (MDC Net)	.0	41.6	55.0*
23. Unit Cap Factor (DER Net)	.0	39.7	52.4
24. Unit Forced Outage Rate	.0	6.3	12.3
25. Forced Outage Hours	0	152.3	11,057.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

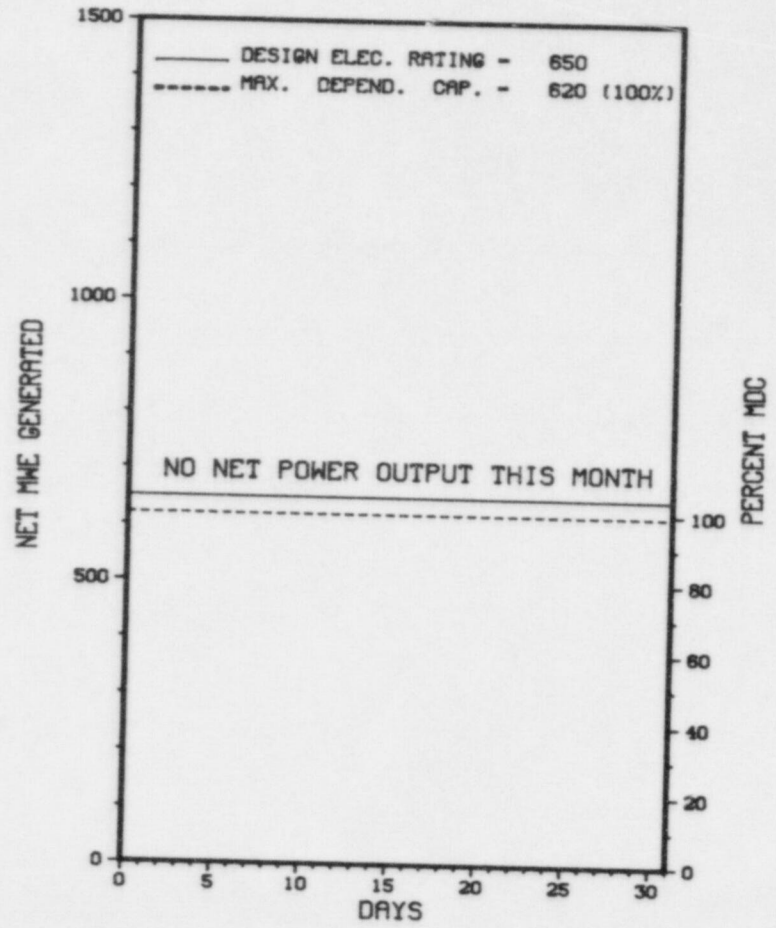
NONE

27. If Currently Shutdown Estimated Startup Date: 10/12/86

****
* OYSTER CREEK 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

OYSTER CREEK 1



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

OYSTER CREEK 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
46	04/12/86	S	744.0	C	4		RC	FUELXX	REFUELING/MAINTENANCE OUTAGE CONTINUES.

SUMMARY

OYSTER CREEK 1 REMAINS SHUTDOWN FOR REFUELING AND MAINTENANCE.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* OYSTER CREEK 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....NEW JERSEY

COUNTY.....OCEAN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...9 MI S OF
TOMS RIVER, NJ

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...MAY 3, 1969

DATE ELEC ENER 1ST GENER...SEPTEMBER 23, 1969

DATE COMMERCIAL OPERATE....DECEMBER 1, 1969

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....BARNEGAT BAY

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....GPU NUCLEAR CORPORATION

CORPORATE ADDRESS.....100 INTERPACE PARKWAY
PARSIPPANY, NEW JERSEY 07054

CONTRACTOR
ARCHITECT/ENGINEER.....BURNS & ROE

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BURNS & ROE

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....W. BATEMAN

LICENSING PROJ MANAGER.....J. DONOHEW
DOCKET NUMBER.....50-219

LICENSE & DATE ISSUANCE....DPR-16, AUGUST 1, 1969

PUBLIC DOCUMENT ROOM.....OCEAN COUNTY LIBRARY
101 WASHINGTON STREET
TOMS RIVER, NEW JERSEY 08753

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

10 CFR 50, APPENDIX B, CRITERION V AND SECTION 3 OF THE OYSTER CREEK OPERATION QUALITY ASSURANCE PLAN REQUIRE, IN PART, THAT ACTIVITIES AFFECTING QUALITY BE PRESCRIBED BY AND ACCOMPLISHED IN ACCORDANCE WITH DOCUMENTED INSTRUCTIONS, PROCEDURES, AND DRAWINGS. CONTRARY TO THE ABOVE, AS OF MARCH 27, 1986, THE LICENSEE FAILED TO: (1) FOLLOW THE REQUIREMENTS OF TECH FUNCTIONS PROCEDURE EMP-014, REV. 1-01, PROJECT REVIEWS, BY NOT ISSUING A MEMORANDUM OF CONCURRENCE FOR MODIFICATION PACKAGES BA 402786 AND BA 402775. (2) ASSIGN A PRELIMINARY ENGINEERING DESIGN REVIEW CHAIRMAN FOR BA 402786 WHO WAS INDEPENDENT OF THE ENGINEERING OR MANAGING OF THE DESIGN PACKAGE AS REQUIRED BY EMP-014. (3) ENFORCE THE MANDATORY ATTENDANCE REQUIREMENTS FOR THE OPERABILITY, MAINTAINABILITY, AND CONSTRUCTABILITY REVIEW FOR BA 402786 AND BA 402775. (4) PROPERLY SCHEDULE THE RESPONSIBLE TECHNICAL REVIEW PRIOR TO THE SAFETY EVALUATION FOR BA 402775 AS REQUIRED BY TECH FUNCTIONS PROCEDURES LP-009, REV. 1-00, INDEPENDENT SAFETY REVIEWS AND EP-016, REV. 0, NUCLEAR SAFETY/ENVIRONMENTAL IMPACT EVALUATIONS. TECH SPEC 6.8.1 REQUIRES THAT WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED. CONTRARY TO ABOVE, AS OF 4/3/86, STAT PROC NO. 125 WAS NOT BEING MAINTAINED. SIGNIFICANT DISCREPANCIES EXISTED BETWEEN PROCEDURAL DESCRIPTIONS AND REQUIREMENTS AND WAY PLANT ENGINEERING ORGAN WAS CONDUCTING ACTIVITIES. THESE DISCREPANCIES INCLUDED: (1) DESCRIPTION OF ORGANIZATION AND RESPONSIBILITIES, (2) PERFORMANCE OF ENGINEERING TASKS INCLUDING INITIAL REVIEW AND ASSIGNMENT OF ENGINEERING TASKS, ACKNOWLEDGEMENT TO REQUESTOR AND CLOSE OUT OF TASKS INCLUDING

ENFORCEMENT SUMMARY

RESPONSE TO REQUESTOR, (3) MAINTENANCE OF RECORDS, AND (4) PRIORITIZATION OF PLANT ENGINEERING TASK ASSIGNMENTS. CONTRARY TO ABOVE, AS OF 4/4/86, STAT PROC NO. 125 WAS NOT ADEQUATELY IMPLEMENTED IN THAT A PLANT ENGINEERING WORK REQUEST WRITTEN 8/19/85 BY PLANT MATERIAL DEPT AND RECEIVED BY PLANT ENGINEERING ON 8/20/85 HAD BEEN GIVEN A PRIORITY TWO RATING AND WAS NOT SCHEDULED FOR REVIEW. THE ISSUE INVOLVED LACK OF PLANS AND PROCEDURES GOVERNING MOVEMENT OF HEAVY LOADS (NUREG-0612) USING A PORTABLE CRANE AT THE INTAKE STRUCTURE IN THE VICINITY OF THE EMERGENCY SERVICE WATER PUMPS. A SAFETY EVALUATION, IN PART, DOCUMENTING THE LICENSEE'S COMMITMENTS REGARDING MOVEMENT OF HEAVY LOADS, WAS ISSUED 6/21/83. IN THIS DOCUMENT IT WAS EXPLAINED THAT MOVEMENT OF HEAVY LOADS AT THE INTAKE WAS NOT A CONCERN BECAUSE THE INTAKE GANTRY CRANE HAD BEEN REMOVED BUT THAT, IF AT SOME TIME IN THE FUTURE THIS CRANE IS PLACED BACK IN SERVICE, AN EVALUATION WOULD BE PERFORMED TO ENSURE THAT NUREG-0612 CRITERIA ARE SATISFIED. ALTHOUGH THE GANTRY CRANE HAS NOT BEEN PLACED IN SERVICE, THE USE OF A PORTABLE CRANE TO MOVE HEAVY LOADS IS AN EQUIVALENT SITUATION.

(8600 4)

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-255 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: P. A. SMITH (616) 764-8913
4. Licensed Thermal Power (MWt): 2530
5. Nameplate Rating (Gross MWe): 955 X 0.85 = 812
6. Design Electrical Rating (Net MWe): 805
7. Maximum Dependable Capacity (Gross MWe): 770
8. Maximum Dependable Capacity (Net MWe): 730
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	128,126.0
13. Hours Reactor Critical	.0	1,490.5	69,790.9
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	1,324.4	66,283.6
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	2,760,600	138,362,640
18. Gross Elec Ener (MWH)	0	896,530	43,113,930
19. Net Elec Ener (MWH)	0	841,244	40,582,604
20. Unit Service Factor	.0	26.0	51.7
21. Unit Avail Factor	.0	26.0	51.7
22. Unit Cap Factor (MDC Net)	.0	22.7	43.4
23. Unit Cap Factor (DER Net)	.0	20.5	39.3
24. Unit Forced Outage Rate	100.0	63.3	32.5
25. Forced Outage Hours	744.0	2,285.1	17,841.2

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

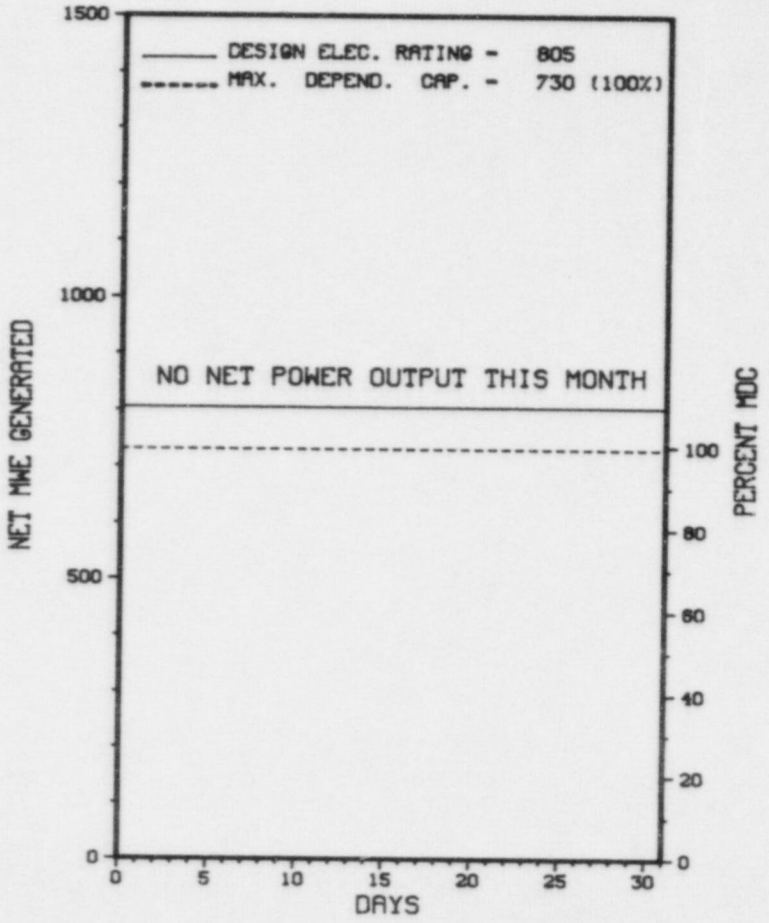
NONE

27. If Currently Shutdown Estimated Startup Date: 09/15/86

****
* PALISADES *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PALISADES



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* PALISADES *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
7	05/19/86	F	744.0	A	4				TURBINE EHC CONTROL SYSTEM PROBLEMS.

****
* SUMMARY *
****

PALISADES REMAINS SHUT DOWN FOR TURBINE EHC CONTROL PROBLEMS.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* PALISADES *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....MICHIGAN

COUNTY.....VANBUREN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI S OF
SOUTH HAVEN, MI

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 24, 1971

DATE ELEC ENER 1ST GENER...DECEMBER 31, 1971

DATE COMMERCIAL OPERATE....DECEMBER 31, 1971

CONDENSER COOLING METHOD...COOLING TOWERS

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....EAST CENTRAL AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CONSUMERS POWER

CORPORATE ADDRESS.....212 WEST MICHIGAN AVENUE
JACKSON, MICHIGAN 49201

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....E. SWANSON

LICENSING PROJ MANAGER.....T. WAMBACH
DOCKET NUMBER.....50-255

LICENSE & DATE ISSUANCE....DPR-20, OCTOBER 16, 1972

PUBLIC DOCUMENT ROOM.....VAN ZOEREN LIBRARY
HOPE COLLEGE
HOLLAND, MICHIGAN
49423

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON JUNE 16-20 (86012): ROUTINE, UNANNOUNCED INSPECTION OF THE RADIATION PROTECTION PROGRAM INCLUDING: ORGANIZATION AND MANAGEMENT CONTROLS, INTERNAL AND EXTERNAL EXPOSURE CONTROLS, CONTROL OF RADIOACTIVE MATERIALS AND CONTAMINATION, TRAINING AND QUALIFICATION, AUDITS AND APPRAISALS, SOLID RADWASTE, TRANSPORTATION, AND THE ALARA PROGRAM. ALSO, HVAC FILTER HOUSING DRAIN SYSTEMS, THE DECONTAMINATION PROGRAM, IE INFORMATION NOTICE NO. 86-22, TERMINATION DOSE RECORDS, AND OPEN ITEMS WERE REVIEWED. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON JULY 1-3 (86020): ROUTINE UNANNOUNCED INSPECTION OF GASEOUS AND LIQUID RADIOACTIVE EFFLUENTS INCLUDING: EFFLUENT RELEASES; RECORDS AND REPORTS OF EFFLUENTS; EFFLUENT CONTROL INSTRUMENTATION; GASEOUS EFFLUENT FILTRATION; AUDITS; AND FOLLOWUP OF A VIOLATION AND TWO OPEN ITEMS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

JR SCHEPERS IS THE CHEMISTRY SUPERINTENDENT IN THE OPERATIONS DEPARTMENT.

PLANT STATUS:

THE UNIT IS IN COLD SHUTDOWN

LAST IE SITE INSPECTION DATE: 08/20/86

INSPECTION REPORT NO: 86025

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-19	06/10/86	07/10/86	INADVERTENT EMERGENCY DIESEL GENERATOR ACTUATION
86-20	06/17/86	07/17/86	INADVERTENT EMERGENCY DIESEL GENERATOR ACTUATION

1. Docket: 50-528 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: J. M. COLVILLE (602) 932-5300 X6593
4. Licensed Thermal Power (MWt): 3800
5. Nameplate Rating (Gross MWe): 1403
6. Design Electrical Rating (Net MWe): 1221
7. Maximum Dependable Capacity (Gross MWe): 1303
8. Maximum Dependable Capacity (Net MWe): 1221
9. If Changes Occur Above Since Last Report, Give Reasons:
DER CHANGED TO REFLECT LIC. THERM. PWR.
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

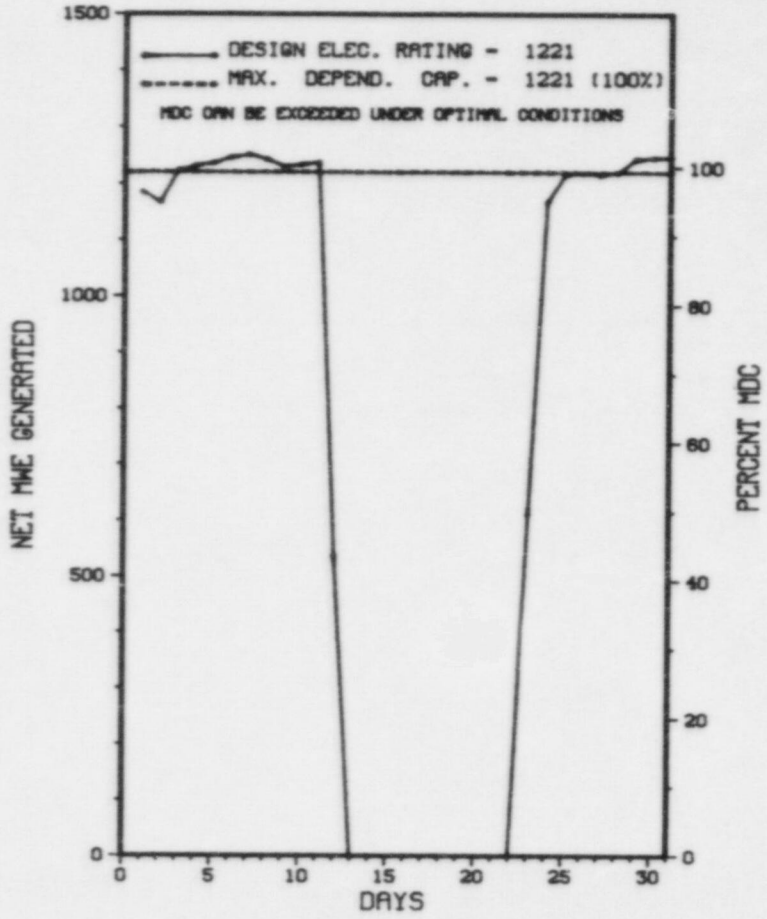
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	4,055.0	4,055.0
13. Hours Reactor Critical	506.9	1,958.2	1,958.2
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	498.9	1,905.8	1,905.8
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,814,285	6,897,623	6,897,623
18. Gross Elec Ener (MWH)	626,200	2,377,800	2,377,800
19. Net Elec Ener (MWH)	572,632	2,188,757	2,188,757
20. Unit Service Factor	67.1	47.0	47.0
21. Unit Avail Factor	67.1	47.0	47.0
22. Unit Cap Factor (MDC Net)	63.0	44.4	44.2
23. Unit Cap Factor (DER Net)	63.0	43.7	44.2
24. Unit Forced Outage Rate	32.9	45.5	45.5
25. Forced Outage Hours	245.1	1,589.4	1,589.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: N/A

****
* PALO VERDE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PALO VERDE 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

PALO VERDE 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
5	07/12/86	F	245.1	A	3	86-047	JC	PD	RX TRIP ON CHANNEL "A" & "B" LOW SG 2 RC FLOW.

SUMMARY

PALO VERDE 1 INCURRED 1 SHUTDOWN IN JULY BECAUSE OF A REACTOR TRIP.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* PALO VERDE 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....ARIZONA

COUNTY.....MARICOPA

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...36 MI W OF
PHOENIX, AZ

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 25, 1985

DATE ELEC ENER 1ST GENER...JUNE 10, 1985

DATE COMMERCIAL OPERATE....FEBRUARY 13, 1986

CONDENSER COOLING METHOD...TREATED SEWAGE

CONDENSER COOLING WATER....SEWAGE TREATMENT

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....ARIZONA PUBLIC SERVICE

CORPORATE ADDRESS.....P.O. BOX 21666
PHOENIX, ARIZONA 85036

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....R. ZIMMERMAN

LICENSING PROJ MANAGER.....E. LICITRA
DOCKET NUMBER.....50-528

LICENSE & DATE ISSUANCE....NPF-41, JUNE 1, 1985

PUBLIC DOCUMENT ROOM.....MS STEFANIE MORITZ
DOCUMENTS LIBRARIAN
PHOENIX PUBLIC LIBRARY
12 EAST MCDOWELL ROAD
PHOENIZ, ARIZONA 85004

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON MAY 19 - AUGUST 2, 1986 (REPORT NO. 50-528/86-15) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON MAY 27 - JUNE 30, 1986 (REPORT NO. 50-528/86-20) AREAS INSPECTED: ROUTINE, ONSITE INSPECTION BY THE FOUR RESIDENT INSPECTORS AND ONE REGIONALLY BASED REACTOR INSPECTOR. AREAS INSPECTED INCLUDED: FOLLOWUP OF PREVIOUSLY IDENTIFIED ITEMS; REVIEW OF PLANT ACTIVITIES; ENGINEERED SAFETY SYSTEM WALKDOWNS; SURVEILANCE TESTING; MAINTENANCE; CONSTRUCTION ACTIVITIES; PREOPERATIONAL AND POWER ASCENSION TEST WITNESSING; PREOPERATIONAL TEST PROCEDURE REVIEW; POWER ASCENSION TEST RESULTS REVIEW; DIESEL GENERATOR BRIDGE CRANE SEISMIC QUALIFICATION REVIEW; ALLEGATION FOLLOWUP; REACTOR COOLANT PRESSURE ISOLATION VALVE LEAK TEST; LICENSEE EVENT REPORT FOLLOWUP; CONSTRUCTION DEFICIENCY REPORT, PART 21 REPORT, AND IE BULLETIN AND INFORMATION NOTICE FOLLOWUP; PERIODIC AND SPECIAL REPORT REVIEW; AND PLANT TOURS. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: OF THE 16 AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED IN ONE AREA (FAILURE TO PERFORM REQUIRED HOURLY FIRE PATROLS).

+ INSPECTION ON JUNE 9-26, 1986 (REPORT NO. 50-528/86-21) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION BY A REGIONALLY BASED INSPECTOR OF LICENSEE ACTION ON INSPECTOR-IDENTIFIED ITEMS. DURING THIS INSPECTION, ONE INSPECTION PROCEDURE WAS UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

****
* PALO VERDE 1 *
****

INSPECTION SUMMARY

+ INSPECTION ON JUNE 23 - JULY 16, 1986 (REPORT NO. 50-528/86-22) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, RADIATION PROTECTION, PLANT CHEMISTRY, AND RADWASTE ORGANIZATION AND MANAGEMENT CONTROLS, INTERNAL EXPOSURE CONTROL, CONTAINMENT ATMOSPHERE MONITOR RU-1, REVIEW OF LICENSEE'S REPORTS, FOLLOWUP ON IE INFORMATION NOTICES AND FACILITY TOURS. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED. ONE UNRESOLVED ITEM RELATING TO THE OPERABILITY OF THE CONTAINMENT ATMOSPHERE MONITOR WAS IDENTIFIED.

+ INSPECTION ON JULY 7-25, 1986 (REPORT NO. 50-528/86-23) AREAS INSPECTED: UNANNOUNCED INSPECTION BY A REGIONALLY BASED INSPECTOR OF 10 CFR 50.55(E) CONSTRUCTION DEFICIENCIES, LICENSEE ACTION IN RESPONSE TO PART 21 REPORTS, ALLEGATIONS, TMI ACTION PLAN ITEMS, FOLLOWUP OF LICENSEE ACTION ON INSPECTOR-IDENTIFIED ITEMS, AND LERS. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JULY 1 - AUGUST 3, 1986 (REPORT NO. 50-528/86-24) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JULY 14-18, 1986 (REPORT NO. 50-528/86-25) REPORT CANCELLED.

+ INSPECTION ON JULY 28 - AUGUST 1, 1986 (REPORT NO. 50-528/86-26) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

AUXILIARY SPRAY SYSTEM SAFETY GRADE ISSUE IS BEING RESOLVED BY NRR.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

ANNUAL SALP BOARD MEETING HELD ON NOVEMBER 14, 1985; SALP REPORT TRANSMITTED TO THE LICENSEE ON DECEMBER 19, 1985.

PLANT STATUS:

THE PLANT ACHIEVED INITIAL CRITICALITY ON MAY 25, 1985. THE PLANT ACHIEVED 100% POWER ON DECEMBER 9, 1985. POWER ASCENSION TESTING IS COMPLETED. COMMERCIAL OPERATION WAS DECLARED ON FEBRUARY 13, 1986. CURRENTLY, THE PLANT IS IN MODE 3 AFTER A MAINTENANCE OUTAGE.

LAST IE SITE INSPECTION DATE: 07/01-08/03/86+

OTHER ITEMS

INSPECTION REPORT NO: 50-528/86-24+

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-28-L0	05-16-86	06-16-86	SURVEILLANCE TESTING OF HALON SYSTEMS INVALIDATED DUE TO PERSONNEL ERROR
86-30-L0	05-12-86	06-11-86	MAIN STEAM ISOLATION SIGNAL DUE TO PERSONNEL ERROR
86-32-L0	05-15-86	06-16-86	PROCEDURAL DEFICIENCY RESULTS IN CONTROL ROOM EMERGENCY FEEDWATER ACTUATION SIGNAL ACTUATION
86-42-X0	04-28-86	05-27-86	APPARENT NONVALID DIESEL FAILURE (SPECIAL REPORT)

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1. Docket: 50-529 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: J. M. COLVILLE (602) 932-5300 X6593
4. Licensed Thermal Power (MWt): 3800
5. Nameplate Rating (Gross MWe): 1403
6. Design Electrical Rating (Net MWe): 1221
7. Maximum Dependable Capacity (Gross MWe): 1221
8. Maximum Dependable Capacity (Net MWe): 1221
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	1,734.3	1,734.3
13. Hours Reactor Critical	171.3	962.6	962.6
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	117.6	794.0	794.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	179,055	1,106,307	1,106,307
18. Gross Elec Ener (MWH)	50,900	296,500	296,500
19. Net Elec Ener (MWH)	26,475	204,271	204,271
20. Unit Service Factor		NOT IN	
21. Unit Avail Factor		NOT IN	
22. Unit Cap Factor (MDC Net)		COMMERCIAL	
23. Unit Cap Factor (DER Net)		OPERATION	
24. Unit Forced Outage Rate			
25. Forced Outage Hours	626.4	807.5	807.5

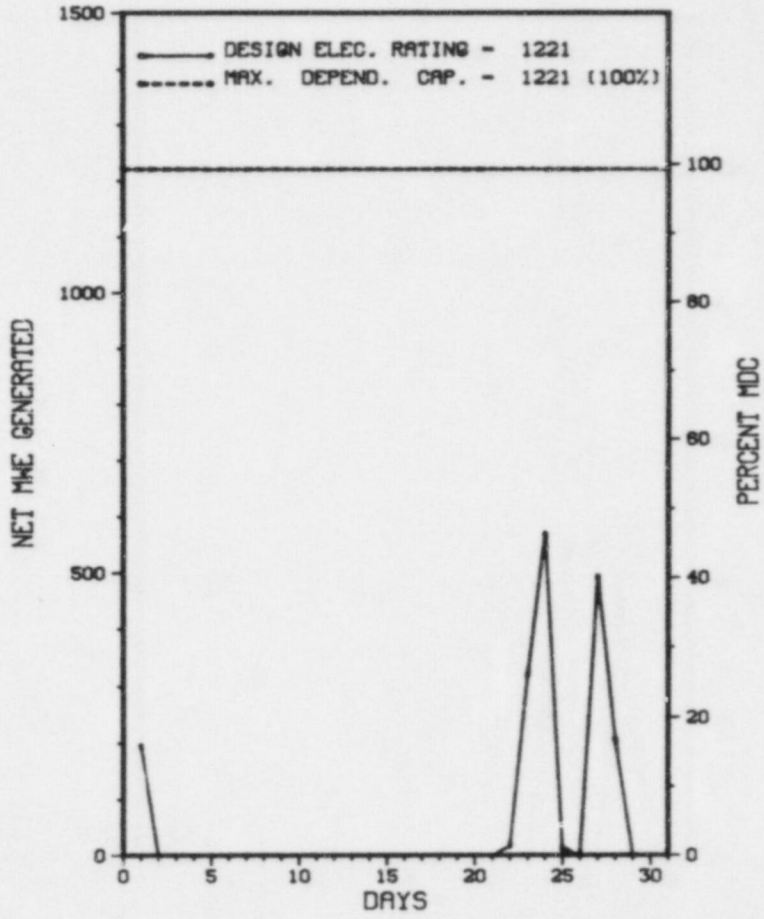
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE

27. If Currently Shutdown Estimated Startup Date: 08/15/86

****
* PALO VERDE 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PALO VERDE 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

PALO VERDE 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
11	07/01/86	F	529.0	D	1		AB	SEAL	COMMENCED A UNIT SHUTDOWN DUE TO UNIDENTIFIED LEAKAGE BEING HIGH. SHUTDOWN REQUIRED BY TECH. SPEC. LCO PROBLEM CORRECTED BY REPLACING RCP SEALS.
12	07/25/86	F	25.4	A	3	86-047	JC	CPU	REACTOR TRIP ON CPC GENERATED HI LPD TRIP.
13	07/28/86	F	72.0	A	1		SG	COND	MAIN TURBINE TRIP UNABLE TO MAINTAIN VACUUM IN CONDENSER.

SUMMARY

PALO VERDE 2 CONTINUES IN POWER ASCENSION AND TESTING.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* PALO VERDE 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....ARIZONA

COUNTY.....MARICOPA

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...36 MI W OF
PHOENIX, AZ

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY... APRIL 18, 1986

DATE ELEC ENER 1ST GENER... MAY 20, 1986

DATE COMMERCIAL OPERATE....**********

CONDENSER COOLING METHOD...COOLING TOWERS

CONDENSER COOLING WATER....SEWAGE TREATMENT

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....ARIZONA PUBLIC SERVICE

CORPORATE ADDRESS.....P.O. BOX 21666
PHOENIX, ARIZONA 85036

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....G. FIORELLI

LICENSING PROJ MANAGER.....E. LICITRA
DOCKET NUMBER.....50-529

LICENSE & DATE ISSUANCE....NPF-51, APRIL 24, 1986

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PHOENIZ, ARIZONA 85004

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON MAY 19 - AUGUST 2, 1986 (REPORT NO. 50-529/86-15) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON MAY 27 - JUNE 30, 1986 (REPORT NO. 50-529/86-20) AREAS INSPECTED: ROUTINE, ONSITE INSPECTION BY THE FOUR RESIDENT INSPECTORS AND ONE REGIONALLY BASED REACTOR INSPECTOR. AREAS INSPECTED INCLUDED: FOLLOWUP OF PREVIOUSLY IDENTIFIED ITEMS; REVIEW OF PLANT ACTIVITIES; ENGINEERED SAFETY SYSTEM WALKDOWNS; SURVEILLANCE TESTING; MAINTENANCE; CONSTRUCTION ACTIVITIES; PREOPERATIONAL AND POWER ASCENSION TEST WITNESSING; PREOPERATIONAL TEST PROCEDURE REVIEW; POWER ASCENSION TEST RESULTS REVIEW; DIESEL GENERATOR BRIDGE CRANE SEISMIC QUALIFICATION REVIEW; ALLEGATION FOLLOWUP; REACTOR COOLANT PRESSURE ISOLATION VALVE LEAK TEST; LICENSEE EVENT REPORT FOLLOWUP; CONSTRUCTION DEFICIENCY REPORT, PART 21 REPORT, AND IE BULLETIN AND INFORMATION NOTICE FOLLOWUP; PERIODIC AND SPECIAL REPORT REVIEW; AND PLANT TOURS. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JUNE 9-26, 1986 (REPORT NO. 50-529/86-21) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPETION BY A REGIONALLY BASED INSPECTOR OF LICENSEE ACTION ON INSPECTOR-IDENTIFIED ITEMS. DURING THIS INSPECTION, ONE INSPECTION PROCEDURE WAS UTILIZED.

RESULT: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION SUMMARY

+ INSPECTION ON JUNE 23 - JULY 16, 1986 (REPORT NO. 50-529/86-22) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, RADIATION PROTECTION, PLANT CHEMISTRY, AND RADWASTE ORGANIZATION AND MANAGEMENT CONTROLS, RADIATION PROTECTION-STARTUP, RADWASTE-STARTUP, INTERNAL EXPOSURE CONTROL, REVIEW OF LICENSEE'S REPORTS, FOLLOWUP ON IE INFORMATION NOTICES AND FACILITY TOURS. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED. ONE UNRESOLVED ITEM RELATING TO THE OPERABILITY OF THE CONTAINMENT ATMOSPHERE MONITOR WAS IDENTIFIED.

+ INSPECTION ON JULY 7-25, 1986 (REPORT NO. 50-529/86-23) AREAS INSPECTED: UNANNOUNCED INSPECTION BY A REGIONALLY BASED INSPECTOR OF 10 CFR 50.55(E) CONSTRUCTION DEFICIENCIES, LICENSEE ACTION IN RESPONSE TO PART 21 REPORTS, ALLEGATIONS, TMI ACTION PLAN ITEMS, FOLLOWUP OF LICENSEE ACTION ON INSPECTOR-IDENTIFIED ITEMS, AND LERS. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: OF THE AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED (FAILURE TO PROVIDE CORRECTIVE ACTION TO PRECLUDE REPETITION OF A SIGNIFICANT CONDITION ADVERSE TO QUALITY).

+ INSPECTION ON JULY 1 - AUGUST 3, 1986 (REPORT NO. 50-529/86-24) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JULY 28 - AUGUST 1, 1986 (REPORT NO. 50-529/86-25) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

1. BALANCE OF PLANT ENGINEERED SAFETY FEATURES ACTUATION SYSTEM. 2. SECONDARY CHEMISTRY

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

POWER ASCENSION TESTING IS ONGOING. UNIT IS AT 50% POWER.

LAST IE SITE INSPECTION DATE: 07/01-08/03/86+

INSPECTION REPORT NO: 50-529/86-24+

REPORTS FROM LICENSEE

****
* PALO VERDE 2 *
****

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-15-X0	05-25-86	05-30-86	REACTOR TRIP RESULTING IN OVERCOOLING EVENT (SPECIAL REPORT)
86-24-L0	05-14-86	06-13-86	INOPERABLE STEAM GENERATOR SNUBBERS DUE TO PERSONNEL ERROR

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1. Docket: 50-277 O P E R A T I N G S T A T U S

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: W. M. Alden (215) 841-5022

4. Licensed Thermal Power (MWt): 3293

5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152

6. Design Electrical Rating (Net MWe): 1065

7. Maximum Dependable Capacity (Gross MWe): 1098

8. Maximum Dependable Capacity (Net MWe): 1051

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	105,839.0
13. Hours Reactor Critical	744.0	4,400.7	69,594.3
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	715.1	4,273.8	67,402.6
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,334,648	12,972,182	199,238,697
18. Gross Elec Ener (MWH)	760,260	4,286,460	65,499,420
19. Net Elec Ener (MWH)	729,091	4,130,025	62,722,320
20. Unit Service Factor	96.1	84.0	63.7
21. Unit Avail Factor	96.1	84.0	63.7
22. Unit Cap Factor (MDC Net)	93.2	77.2	56.4
23. Unit Cap Factor (DER Net)	92.0	76.2	55.6
24. Unit Forced Outage Rate	.0	13.4	13.0
25. Forced Outage Hours	.0	662.7	10,075.2

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

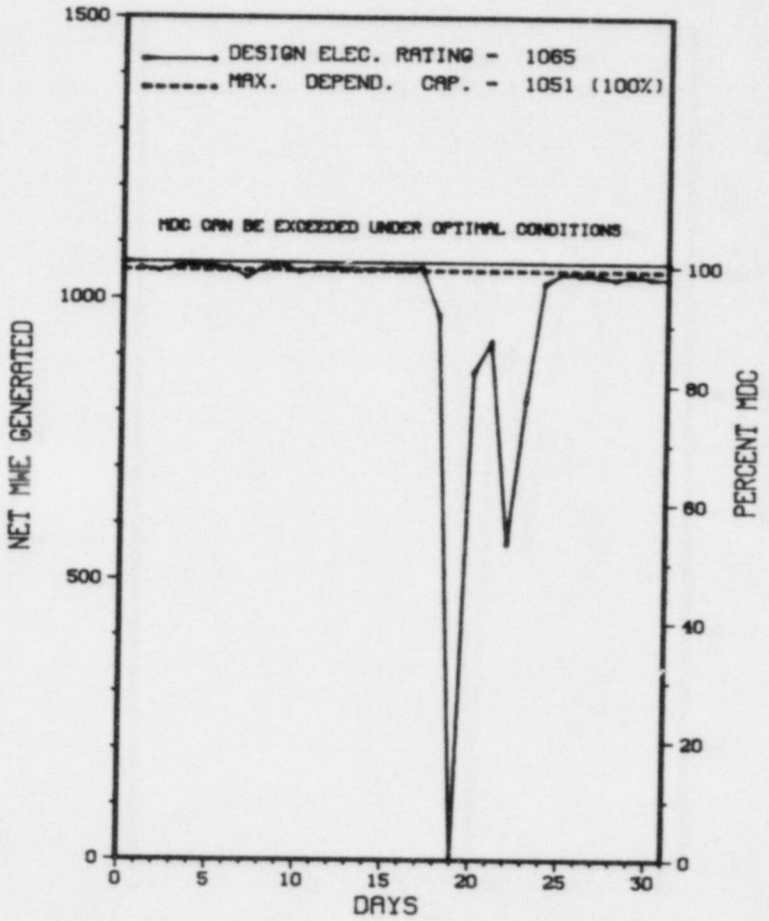
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* PEACH BOTTOM 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PEACH BOTTOM 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

PEACH BOTTOM 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	07/19/86	S	28.9	B	3		HB	PIPEXX	GENERATOR SHUTDOWN FOR FLANGE LEAK REPAIR ON HIGH PRESSURE TURBINE "C" MOISTURE SEPARATOR STEAM LINE. REACTOR REMAINED AT 22% POWER.
2	07/22/86	S	0.0	H	5		RD	CONROD	LOAD REDUCTION TO 650MW FOR CONTROL ROD PATTERN ADJUSTMENTS.

SUMMARY

PEACH BOTTOM 2 INCURRED 1 OUTAGE AND 1 POWER REDUCTION IN JULY AS DISCUSSED ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* PEACH BOTTOM 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....PENNSYLVANIA

COUNTY.....YORK

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...19 MI S OF
LANCASTER, PA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...SEPTEMBER 16, 1973

DATE ELEC ENER 1ST GENER...FEBRUARY 18, 1974

DATE COMMERCIAL OPERATE....JULY 5, 1974

CONDENSER COOLING METHOD..ONCE THRU

CONDENSER COOLING WATER....SUSQUEHANNA RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PHILADELPHIA ELECTRIC

CORPORATE ADDRESS.....2301 MARKET STREET
PHILADELPHIA, PENNSYLVANIA 19105

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....T. JOHNSON

LICENSING PROJ MANAGER.....R. CLARK
DOCKET NUMBER.....50-277

LICENSE & DATE ISSUANCE....DPR-44, DECEMBER 14, 1973

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HARRISBURG, PENNSYLVANIA 17105

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-278 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: W. M. Alden (215) 841-5022
4. Licensed Thermal Power (MWt): 3293
5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152
6. Design Electrical Rating (Net MWe): 1065
7. Maximum Dependable Capacity (Gross MWe): 1098
8. Maximum Dependable Capacity (Net MWe): 1035
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

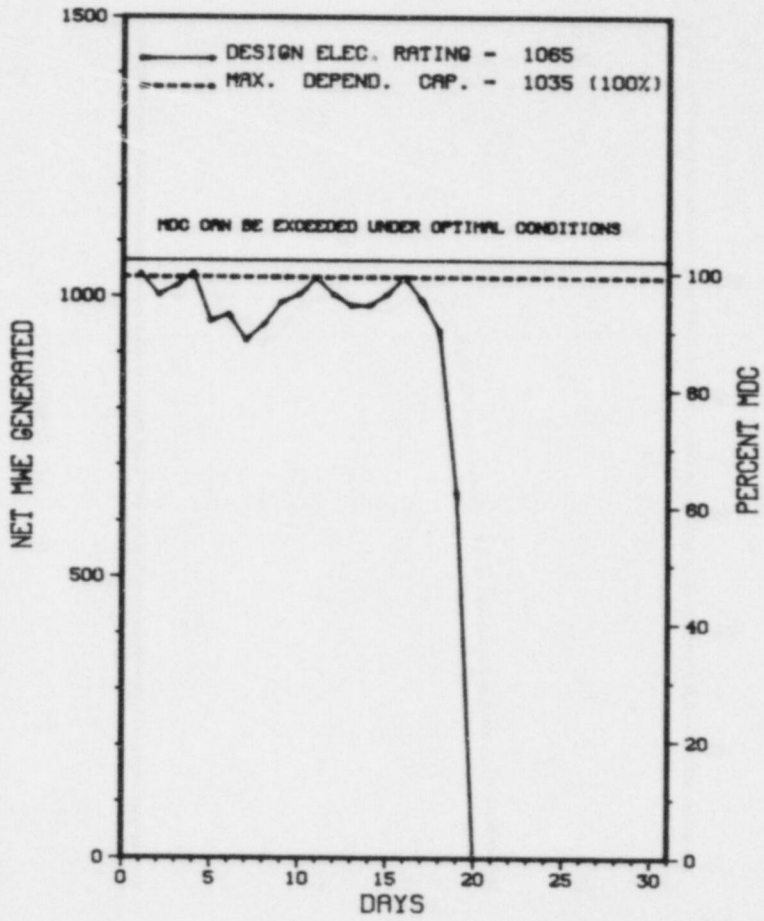
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	101,735.0
13. Hours Reactor Critical	448.2	2,938.1	71,551.6
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	448.2	2,681.5	69,535.9
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,409,544	7,314,605	202,311,269
18. Gross Elec Ener (MWH)	463,720	2,368,950	66,362,620
19. Net Elec Ener (MWH)	440,295	2,257,687	63,650,428
20. Unit Service Factor	60.2	52.7	68.4
21. Unit Avail Factor	60.2	52.7	68.4
22. Unit Cap Factor (MDC Net)	57.2	42.9	60.4
23. Unit Cap Factor (DER Net)	55.6	41.7	58.7
24. Unit Forced Outage Rate	39.8	13.4	7.4
25. Forced Outage Hours	295.8	415.5	5,542.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
SAFE-END/PIPE WELD INSPECTION: 1/3/87 TO 1/23/87.
27. If Currently Shutdown Estimated Startup Date: 08/20/86

PEACH BOTTOM 3

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PEACH BOTTOM 3



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

PEACH BOTTOM 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
0	07/19/86	F	295.8	A	3	3-86-16	ED	CKTBRK	REACTOR SCRAM FROM 87% POWER ON HIGH NEUTRON FLUX AS A RESULT OF THE TRIP OF THE 83 STARTUP FEED BREAKER WHICH CAUSED THE FAST CLOSURE OF THE 80-B MSIV.

SUMMARY

PEACH BOTTOM 3 HAD 1 REACTOR SCRAM IN JULY AS DISCUSSED ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report (LER) File (NUREG-0161)
			9-Other	

****
* PEACH BOTTOM 3 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....PENNSYLVANIA

COUNTY.....YORK

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...19 MI S OF
LANCASTER, PA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...AUGUST 7, 1974

DATE ELEC ENER 1ST GENER...SEPTEMBER 1, 1974

DATE COMMERCIAL OPERATE....DECEMBER 23, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....SUSQUEHANNA RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PHILADELPHIA ELECTRIC

CORPORATE ADDRESS.....2301 MARKET STREET
PHILADELPHIA, PENNSYLVANIA 19105

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....T. JOHNSON

LICENSING PROJ MANAGER.....R. CLARK
DOCKET NUMBER.....50-278

LICENSE & DATE ISSUANCE....DPR-56, JULY 2, 1974

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INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-293 O P E R A T I N G S T A T U S

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: P. HAMILTON (617) 746-7900

4. Licensed Thermal Power (MWt): 1998

5. Nameplate Rating (Gross MWe): 780 X 0.87 = 678

6. Design Electrical Rating (Net MWe): 655

7. Maximum Dependable Capacity (Gross MWe): 690

8. Maximum Dependable Capacity (Net MWe): 670

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	119,591.0
13. Hours Reactor Critical	.0	1,715.5	79,778.7
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	1,646.0	77,216.9
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	3,094,416	135,480,048
18. Gross Elec Ener (MWH)	0	1,068,000	45,444,604
19. Net Elec Ener (MWH)	0	1,027,531	43,675,429
20. Unit Service Factor	.0	32.4	64.6
21. Unit Avail Factor	.0	32.4	64.6
22. Unit Cap Factor (MDC Net)	.0	30.1	54.5
23. Unit Cap Factor (DER Net)	.0	30.8	55.8
24. Unit Forced Outage Rate	100.0	67.2	12.4
25. Forced Outage Hours	744.0	3,378.5	10,922.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

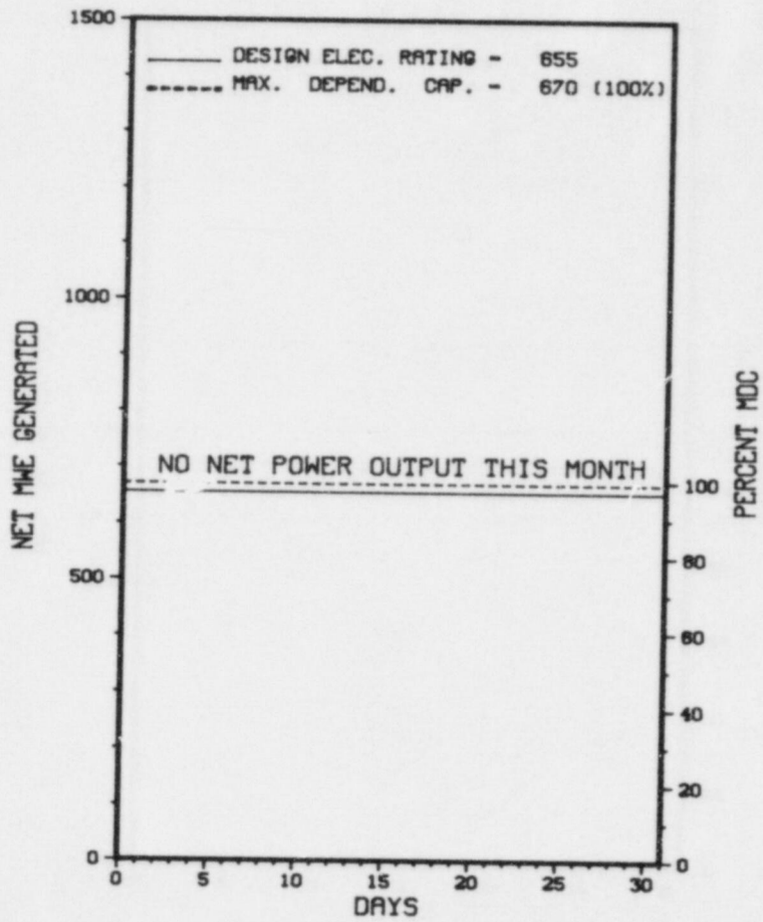
NONE

27. If Currently Shutdown Estimated Startup Date: 02/14/87

****
* PILGRIM 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PILGRIM 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

PILGRIM 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
08	07/01/86	F	744.0	A	4				THE UNIT REMAINED SHUTDOWN TO ADDRESS VARIOUS HARDWARE AND ADMINISTRATIVE ITEMS INCLUDING RHR PUMP WEAR RING INSPECTIONS AND LOCAL LEAK RATE TESTING. RFO 7 COMMENCED 7/25/86.

SUMMARY

PILGRIM 1 REMAINS SHUT DOWN FOR VARIOUS HARDWARE AND ADMINISTRATIVE ITEMS.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* PILGRIM 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....MASSACHUSETTS

COUNTY.....PLYMOUTH

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...4 MI SE OF
PLYMOUTH, MASS

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JUNE 16, 1972

DATE ELEC ENER 1ST GENER...JULY 19, 1972

DATE COMMERCIAL OPERATE....DECEMBER 1, 1972

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CAPE COD BAY

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....BOSTON EDISON

CORPORATE ADDRESS.....800 BOYLSTON STREET
BOSTON, MASSACHUSETTS 02199

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....M. MCBRIDE

LICENSING PROJ MANAGER.....R. AULUCK
DOCKET NUMBER.....50-293

LICENSE & DATE ISSUANCE....DPR-35, SEPTEMBER 15, 1972

PUBLIC DOCUMENT ROOM.....PLYMOUTH PUBLIC LIBRARY
11 NORTH STREET
PLYMOUTH, MASSACHUSETTS 02360

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 6.8 REQUIRES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES BE ESTABLISHED, IMPLEMENTED AND MAINTAINED, THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF APPENDIX 'A' OF REGULATORY GUIDE 1.33 - 1972. STATION APPROVED WRITTEN PROCEDURES BE ADHERED TO BY ALL STATION PERSONNEL. PROCEDURE PNPS-6.9-160, REVISION 20 SPECIFIED IN PART, TO SURVEY MATERIAL TO BE PACKAGED AND TO ENTER THE MAXIMUM RADIATION LEVEL AT THE SURFACE OF THE PACKAGE, AND TO LIST THE TOTAL ACTIVITY WITHIN THE PACKAGE. APPROPRIATE RADIATION SURVEY FORMS AND MAPS ARE ALSO SPECIFIED. CONTRARY TO THE ABOVE, ON JANUARY 23, 1985, JULY 9, 1985, AND AUGUST 2, 1985, RADIATION SURVEYS OF PACKAGED IRRADIATED REACTOR COMPONENTS WERE NOT DOCUMENTED ON APPROPRIATE RADIATION SURVEY FORMS AND MAPS; AND THE MAXIMUM RADIATION LEVEL AT THE SURFACE OF THE PACKAGE WAS INCORRECT FOR SHIPMENT NUMBERS 85-06 AND 85-66; AND THE TOTAL RADIOACTIVITY CONTAINED WITHIN SHIPMENT NUMBER 85-79 WAS INCORRECTLY LISTED DURING TRANSPORT. 10 CFR 20.31(D)(3), "TRANSFER FOR DISPOSAL AND MANIFESTS" REQUIRES LICENSEE WHO TRANSFERS RADIOACTIVE WASTE TO A LAND DISPOSAL FACILITY TO CONDUCT A QUALITY CONTROL PROGRAM TO ASSURE COMPLIANCE WITH 10 CFR 61.56. CONTRARY TO THE ABOVE, THE LICENSEE HAD TRANSFERRED SEVERAL RADIOACTIVE WASTE SHIPMENTS COMPOSED OF DEWATERED RESINS AND DIATOMACEOUT EARTH ABSORBERS TO A LAND DISPOSAL FACILITY DURING 1984, 1985 AND THE LICENSEE HAD NOT CONDUCTED A QUALITY CONTROL PROGRAM TO ASSURE THAT THE PACKAGES DID NOT CONTAIN FREE STANDING LIQUID GREATER THAN 1% OF THE WASTE VOLUME, AND THAT THE WASTE WAS STRUCTURALLY STABLE FOR

ENFORCEMENT SUMMARY

COMPLIANCE WITH 10 CFR 61.56.
(8601 4)

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-266 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: C. W. KRAUSE (414) 277-2001

4. Licensed Thermal Power (MWt): 1518

5. Nameplate Rating (Gross MWe): 582 X 0.9 = 524

6. Design Electrical Rating (Net MWe): 497

7. Maximum Dependable Capacity (Gross MWe): 509

8. Maximum Dependable Capacity (Net MWe): 485

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	137,927.0
13. Hours Reactor Critical	744.0	4,236.3	111,709.3
14. Rx Reserve Shtdwn Hrs	.0	11.0	645.4
15. Hrs Generator On-Line	714.8	4,1[illegible].5	109,028.3
16. Unit Reserve Shtdwn Hrs	.0	28.0	832.0
17. Gross Therm Ener (MWH)	1,070,864	6,062,657	149,230,601
18. Gross Elec Ener (MWH)	366,870	2,075,070	50,226,750
19. Net Elec Ener (MWH)	350,671	1,983,036	47,814,302
20. Unit Service Factor	96.1	81.0	79.0
21. Unit Avail Factor	96.1	81.6	79.7
22. Unit Cap Factor (MDC Net)	97.2	80.4	71.0*
23. Unit Cap Factor (DER Net)	94.8	78.4	69.8
24. Unit Forced Outage Rate	.0	.6	2.3
25. Forced Outage Hours	.0	24.0	2,437.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

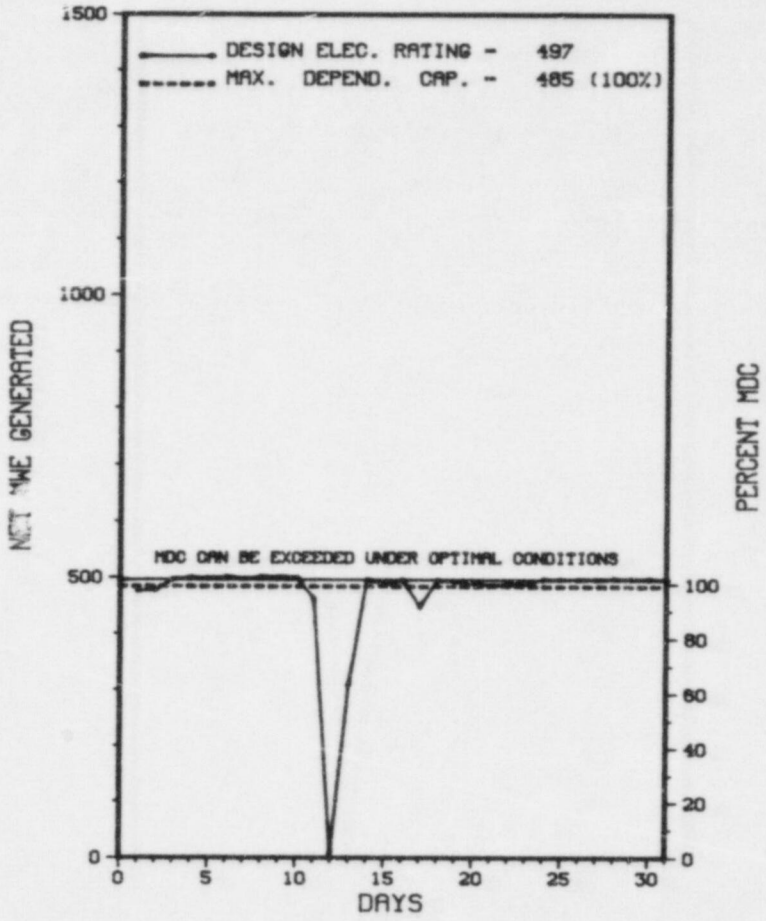
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* POINT BEACH 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

POINT BEACH 1



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

POINT BEACH 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
5	07/12/86	S	29.2	B	1		ZZ	ZZZZZ	SCHEDULED SHUTDOWN TO ALLOW REPAIRS TO THE 1A MOISTURE SEPARATOR REHEATER.

SUMMARY

POINT BEACH 1 HAD 1 SHUTDOWN IN JULY FOR MOISTURE SEPARATOR REHEATER REPAIRS.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* POINT BEACH 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....WISCONSIN

COUNTY.....MANITOWOC

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...15 MI N OF
MANITOWOC, WISC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...NOVEMBER 2, 1970

DATE ELEC ENER 1ST GENER...NOVEMBER 6, 1970

DATE COMMERCIAL OPERATE....DECEMBER 21, 1970

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....WISCONSIN ELECTRIC POWER COMPANY

CORPORATE ADDRESS.....231 WEST MICHIGAN STREET
MILWAUKEE, WISCONSIN 53201

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....R. HAGUE

LICENSING PROJ MANAGER.....T. COLBURN
DOCKET NUMBER.....50-266

LICENSE & DATE ISSUANCE....DPR-24, OCTOBER 5, 1970

PUBLIC DOCUMENT ROOM.....JOSEPH MANN PUBLIC LIBRARY
1516 16TH ST.
TWO RIVERS, WISCONSIN 54241

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON JUNE 16-19 AND 23-27 (86011): ROUTINE UNANNOUNCED INSPECTION BY ONE REGIONAL INSPECTOR OF QA IMPLEMENTATION; TESTS AND EXPERIMENTS; RECORDS; OFFSITE SUPPORT STAFF; AND NONROUTINE REPORTING. THE INSPECTION ADDRESSED SELECTED SECTIONS OF NRC INSPECTION PROCEDURES NO. 35701, 37703, 39701, 40703 AND 90714. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: 09/30/86

INSPECTION REPORT NO: 86015

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-03	06/03/86	07/03/86	REACTOR TRIP DUE TO LOSS OF POWER ON THE WHITE INSTRUMENT BUS
86-04	06/29/86	07/07/86	INDICATION OF ROD CONTROL CLUSTER ASSEMBLY MISALIGNMENT

1. Docket: 50-301 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: C. W. KRAUSE (414) 277-2001
4. Licensed Thermal Power (MWt): 1518
5. Nameplate Rating (Gross MWe): 582 X 0.9 = 524
6. Design Electrical Rating (Net MWe): 497
7. Maximum Dependable Capacity (Gross MWe): 509
8. Maximum Dependable Capacity (Net MWe): 485
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	122,712.0
13. Hours Reactor Critical	744.0	5,083.2	108,631.8
14. Rx Reserve Shtdwn Hrs	.0	.0	207.4
15. Hrs Generator On-Line	718.4	5,037.6	106,838.3
16. Unit Reserve Shtdwn Hrs	.0	9.9	244.2
17. Gross Therm Ener (MWH)	1,069,310	7,460,114	150,339,984
18. Gross Elec Ener (MWH)	362,010	2,523,670	50,939,520
19. Net Elec Ener (MWH)	345,048	2,410,766	48,531,485
20. Unit Service Factor	96.6	99.0	87.1
21. Unit Avail Factor	96.6	99.2	87.3
22. Unit Cap Factor (MDC Net)	95.6	97.7	80.6*
23. Unit Cap Factor (DER Net)	93.3	95.4	79.6
24. Unit Forced Outage Rate	.0	.5	1.3
25. Forced Outage Hours	.0	23.8	768.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

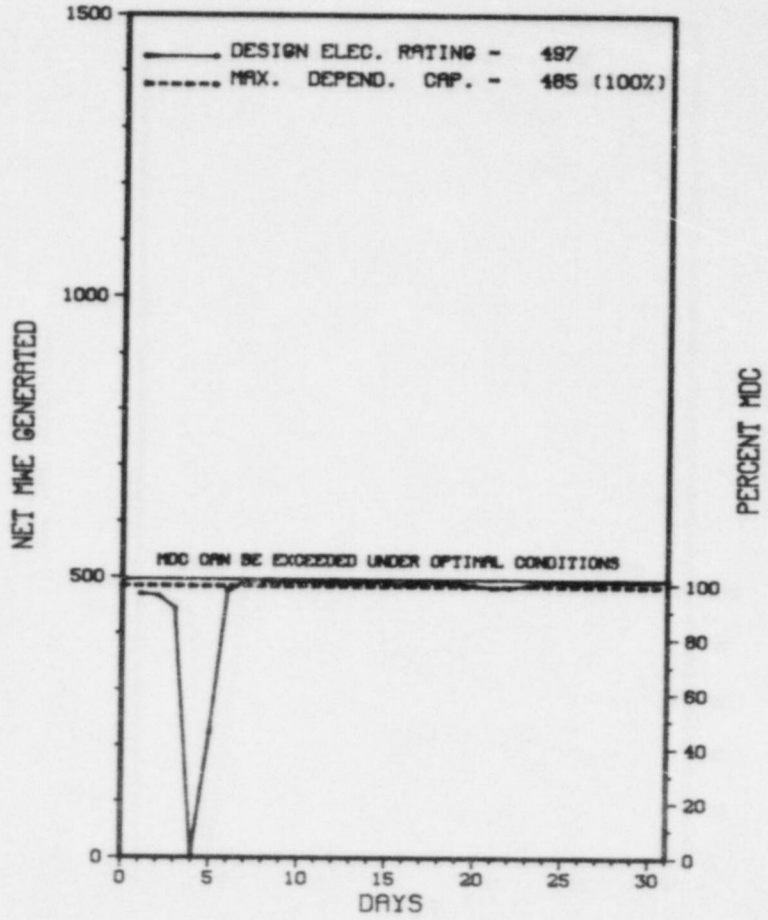
REFUELING OUTAGE: SEPTEMBER 27, 1986, 8 - WEEKS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* POINT BEACH 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

POINT BEACH 2



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

POINT BEACH 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
5	07/03/86	S	25.6	B	1		ZZ	ZZZZZZ	REPAIR LEAK ON "A" STEAM GENERATOR INBOARD MANWAY AND REPLACE A REACTOR COOLANT FLOW TRANSMITTER MANIFOLD VALVE.

SUMMARY

POINT BEACH 2 HAD 1 MAINTENANCE SHUTDOWN IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* POINT BEACH 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....WISCONSIN

COUNTY.....MANITOWOC

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...15 MI N OF
MANITOWOC, WISC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 30, 1972

DATE ELEC ENER 1ST GENER...AUGUST 2, 1972

DATE COMMERCIAL OPERATE....OCTOBER 1, 1972

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....WISCONSIN ELECTRIC POWER COMPANY

CORPORATE ADDRESS.....231 WEST MICHIGAN STREET
MILWAUKEE, WISCONSIN 53201

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....R. HAGUE

LICENSING PROJ MANAGER.....T. COLBURN
DOCKET NUMBER.....50-301

LICENSE & DATE ISSUANCE....DPR-27, MARCH 8, 1973

PUBLIC DOCUMENT ROOM.....JOSEPH MANN PUBLIC LIBRARY
1516 16TH ST.
TWO RIVERS, WISCONSIN 54241

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON JUNE 16-19 AND 23-27 (86011): ROUTINE UNANNOUNCED INSPECTION BY ONE REGIONAL INSPECTOR OF QA IMPLEMENTATION; TESTS AND EXPERIMENTS; RECORDS; OFFSITE SUPPORT STAFF; AND NONROUTINE REPORTING. THE INSPECTION ADDRESSED SELECTED SECTIONS OF NRC INSPECTION PROCEDURES NO. 35701, 37703, 39701, 40703 AND 90714. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: 09/30/86

INSPECTION REPORT NO: 86014

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
85-02	10/17/85	07/23/86	FAILURE OF CONTAINMENT ISOLATION VALVE 2-755A
86-03	06/03/86	07/03/86	RUNBACK DUE TO LOSS OF INSTRUMENT BUS

1. Docket: 50-282 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: DALE DUGSTAD (612) 388-1121
4. Licensed Thermal Power (MWt): 1650
5. Nameplate Rating (Gross MWe): 659 X 0.9 = 593
6. Design Electrical Rating (Net MWe): 530
7. Maximum Dependable Capacity (Gross MWe): 534
8. Maximum Dependable Capacity (Net MWe): 503
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	110,663.0
13. Hours Reactor Critical	744.0	4,239.4	91,596.9
14. Rx Reserve Shtdwn Hrs	.0	.0	5,571.1
15. Hrs Generator On-Line	744.0	4,216.5	90,219.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,222,014	6,264,415	141,633,028
18. Gross Elec Ener (MWH)	399,710	2,060,120	46,261,390
19. Net Elec Ener (MWH)	375,586	1,930,131	43,357,965
20. Unit Service Factor	100.0	82.9	81.5
21. Unit Avail Factor	100.0	82.9	81.5
22. Unit Cap Factor (MDC Net)	100.4	75.4	77.9
23. Unit Cap Factor (DER Net)	95.2	71.6	73.9
24. Unit Forced Outage Rate	.0	.1	7.3
25. Forced Outage Hours	.0	6.1	3,396.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

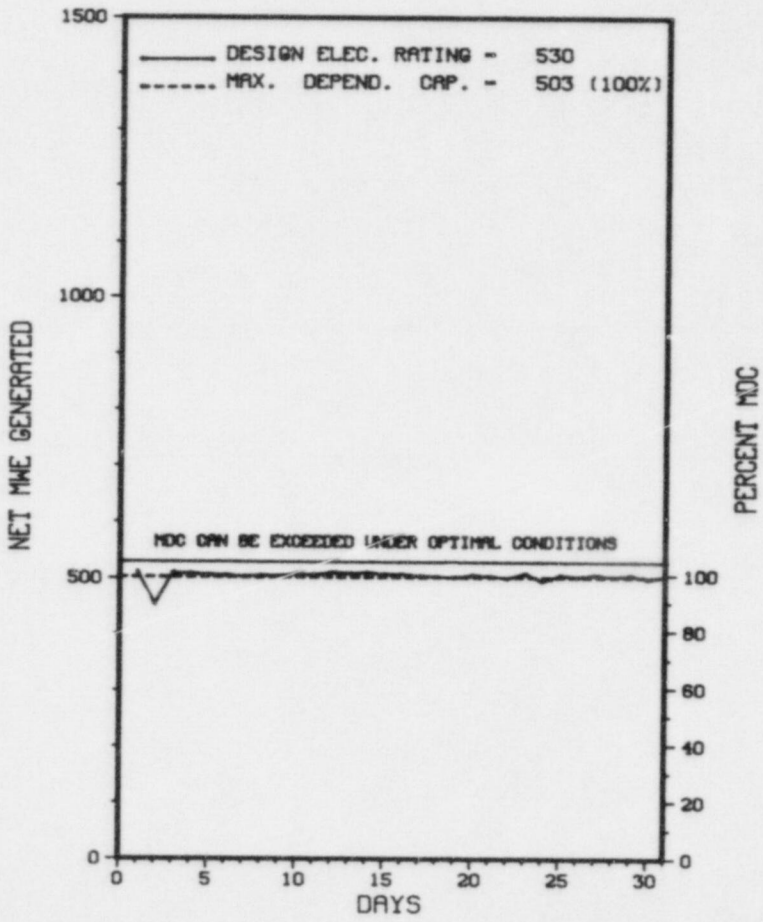
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* PRAIRIE ISLAND 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PRAIRIE ISLAND 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

PRAIRIE ISLAND 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	07/02/86	F	0.0	A	5				STEAM LEAK ON 12 FWP DISCHARGE PIPING.
2	07/02/86	S	0.0	B	5				TURBINE VALVES TEST.

SUMMARY

PRAIRIE ISLAND 1 OPERATED ROUTINELY IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* PRAIRIE ISLAND 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....MINNESOTA

COUNTY.....GOODHUE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...28 MI SE OF
MINNEAPOLIS, MINN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...DECEMBER 1, 1973

DATE ELEC ENER 1ST GENER...DECEMBER 4, 1973

DATE COMMERCIAL OPERATE....DECEMBER 16, 1973

CONDENSER COOLING METHOD...COOLING TOWERS

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-CONTINENT AREA
RELIABILITY COORDINATION
AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NORTHERN STATES POWER

CORPORATE ADDRESS.....414 NICOLLET MALL
MINNEAPOLIS, MINNESOTA 55401

CONTRACTOR
ARCHITECT/ENGINEER.....FLUOR PIONEER, INC.

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....NORTHERN STATES POWER COMPANY

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....J. HARD

LICENSING PROJ MANAGER.....D. DIIANNI
DOCKET NUMBER.....50-282

LICENSE & DATE ISSUANCE....DPR-42, APRIL 5, 1974

PUBLIC DOCUMENT ROOM.....ENVIRONMENTAL CONSERVATION LIBRARY
MINNEAPOLIS PUBLIC LIBRARY
300 NICOLLET MALL
MINNEAPOLIS, MINNESOTA 55401

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON JUNE 16-19 (86006; 86006): ROUTINE ANNOUNCED INSPECTION OF PRAIRIE ISLAND NUCLEAR GENERATING PLANT EMERGENCY PREPAREDNESS EXERCISE INVOLVING OBSERVATIONS BY SEVEN NRC REPRESENTATIVES OF KEY FUNCTIONS AND LOCATIONS DURING THE EXERCISE. THE INSPECTION WAS CONDUCTED BY FIVE NRC INSPECTORS AND TWO CONSULTANTS. NO VIOLATIONS, DEFICIENCIES OR DEVIATIONS WERE IDENTIFIED AS A RESULT OF THIS INSPECTION.

INSPECTION ON JUNE 30 (86008; 86010): ROUTINE UNANNOUNCED INSPECTION OF THE RESOLUTION OF AN IE BULLETIN. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY.

LAST IE SITE INSPECTION DATE: 08/16/86

INSPECTION REPORT NO: 86009

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-306 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: DALE DUGSTAD (612) 388-1121
4. Licensed Thermal Power (MWt): 1650
5. Nameplate Rating (Gross MWe): 659 X 0.9 = 593
6. Design Electrical Rating (Net MWe): 530
7. Maximum Dependable Capacity (Gross MWe): 531
8. Maximum Dependable Capacity (Net MWe): 500
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	101,781.0
13. Hours Reactor Critical	732.1	5,053.1	88,556.0
14. Rx Reserve Shtdwn Hrs	.0	.0	1,516.1
15. Hrs Generator On-Line	727.6	5,043.5	87,546.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,164,946	8,235,697	137,908,088
18. Gross Elec Ener (MWH)	381,080	2,732,220	44,791,660
19. Net Elec Ener (MWH)	358,511	2,586,704	42,076,021
20. Unit Service Factor	97.8	99.1	86.0
21. Unit Avail Factor	97.8	99.1	86.0
22. Unit Cap Factor (MDC Net)	96.4	101.7	82.7
23. Unit Cap Factor (DER Net)	90.9	95.9	78.0
24. Unit Forced Outage Rate	2.2	.9	3.5
25. Forced Outage Hours	16.4	43.5	3,359.0

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

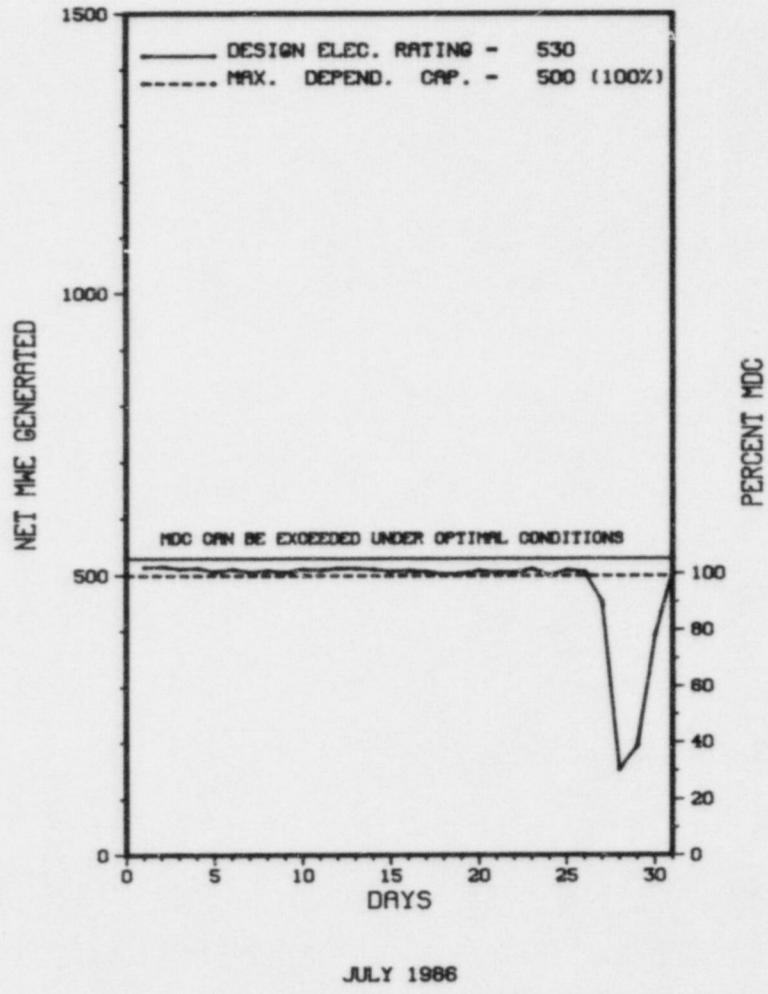
REFUELING-OCTOBER 16, 1986.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* PRAIRIE ISLAND 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

PRAIRIE ISLAND 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* PRAIRIE ISLAND 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	07/15/86	S	0.0	B	5				AXIAL OFFSET CALIBRATION.
2	07/27/86	S	0.0	B	5				TURBINE VALVES TESTING.
3	07/28/86	F	16.4	G	3	86-003			ERROR DURING SAFEGUARDS LOGIC TESTING.

****
* SUMMARY *
****

PRAIRIE ISLAND 2 OPERATED ROUTINELY IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* PRAIRIE ISLAND 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....MINNESOTA

COUNTY.....GOODHUE

DIST AND DIRECTION FROM NEAREST POPULATION CTR...28 MI SE OF MINNEAPOLIS, MINN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...DECEMBER 17, 1974

DATE ELEC ENER 1ST GENER...DECEMBER 21, 1974

DATE COMMERCIAL OPERATE....DECEMBER 21, 1974

CONDENSER COOLING METHOD...COOLING TOWERS

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY COUNCIL.....MID-CONTINENT AREA RELIABILITY COORDINATION AGREEMENT

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....NORTHERN STATES POWER

CORPORATE ADDRESS.....414 NICOLLET MALL
MINNEAPOLIS, MINNESOTA 55401

CONTRACTOR
ARCHITECT/ENGINEER.....FLUOR PIONEER, INC.

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....NORTHERN STATES POWER COMPANY

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....J. HARD

LICENSING PROJ MANAGER.....D. DIIANNI
DOCKET NUMBER.....50-306

LICENSE & DATE ISSUANCE....DPR-60, OCTOBER 29, 1974

PUBLIC DOCUMENT ROOM.....ENVIRONMENTAL CONSERVATION LIBRARY
MINNEAPOLIS PUBLIC LIBRARY
300 NICOLLET MALL
MINNEAPOLIS, MINNESOTA 55401

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON JUNE 16-19 (86006; 86006): ROUTINE ANNOUNCED INSPECTION OF PRAIRIE ISLAND NUCLEAR GENERATING PLANT EMERGENCY PREPAREDNESS EXERCISE INVOLVING OBSERVATIONS BY SEVEN NRC REPRESENTATIVES OF KEY FUNCTIONS AND LOCATIONS DURING THE EXERCISE. THE INSPECTION WAS CONDUCTED BY FIVE NRC INSPECTORS AND TWO CONSULTANTS. NO VIOLATIONS, DEFICIENCIES OR DEVIATIONS WERE IDENTIFIED AS A RESULT OF THIS INSPECTION.

INSPECTION ON JUNE 30 (86008; 86010): ROUTINE UNANNOUNCED INSPECTION OF THE RESOLUTION OF AN IE BULLETIN. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

UNIT IS OPERATING NORMALLY

LAST IE SITE INSPECTION DATE: 08/16/86

INSPECTION REPORT NO: 86011

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-254 O P E R A T I N G S T A T U S

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: CAROL KRONICH (309) 654-2241 X193

4. Licensed Thermal Power (MWt): 2511

5. Nameplate Rating (Gross MWe): 920 X 0.9 = 828

6. Design Electrical Rating (Net MWe): 789

7. Maximum Dependable Capacity (Gross MWe): 813

8. Maximum Dependable Capacity (Net MWe): 769

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	124,655.0
13. Hours Reactor Critical	702.3	2,858.6	99,520.1
14. Rx Reserve Shtdwn Hrs	.0	.0	3,421.9
15. Hrs Generator On-Line	690.5	2,800.5	96,079.2
16. Unit Reserve Shtdwn Hrs	.0	.0	909.2
17. Gross Therm Ener (MWH)	1,668,413	6,629,226	201,594,383
18. Gross Elec Ener (MWH)	539,577	2,151,347	65,272,834
19. Net Elec Ener (MWH)	515,725	2,048,352	61,075,666
20. Unit Service Factor	92.8	55.1	77.1
21. Unit Avail Factor	92.8	55.1	77.8
22. Unit Cap Factor (MDC Net)	90.1	52.4	63.7
23. Unit Cap Factor (DER Net)	87.9	51.0	62.1
24. Unit Forced Outage Rate	7.2	2.7	5.5
25. Forced Outage Hours	53.5	76.3	3,257.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

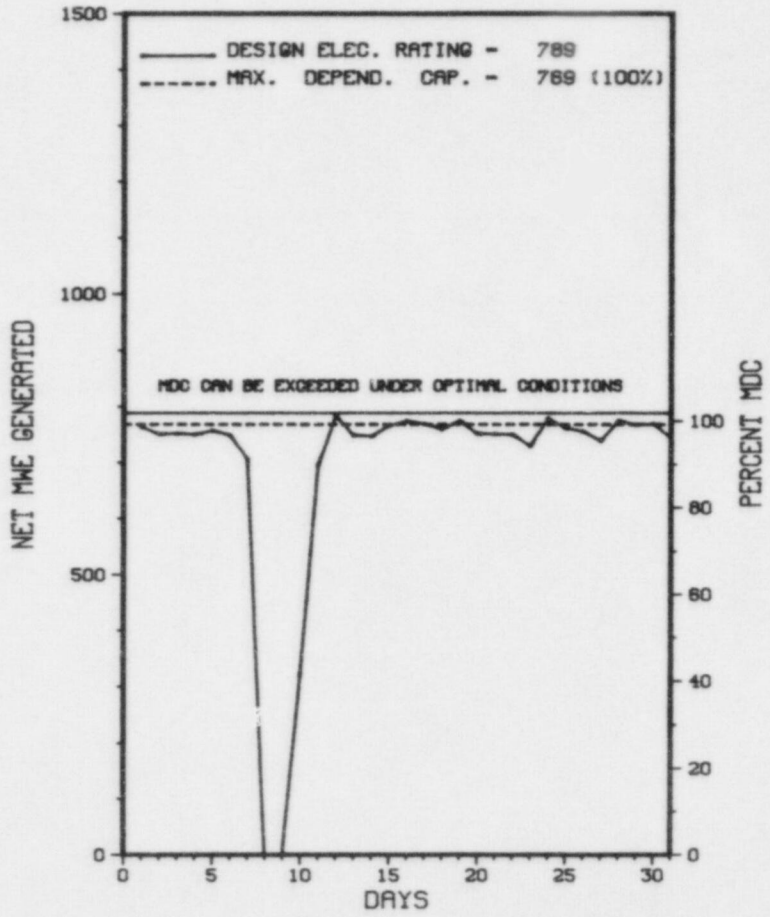
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* QUAD CITIES 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

QUAD CITIES 1



JULY 1986

Report Period JUL 1986

U N I T S H U T D O W N S / R E D U C T I O N S

****
* QUAD CITIES 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-20	07/05/86	S	0.0	B	5		HA	TURBIN	REDUCED LOAD TO 700 MWE FOR TURBINE SURVEILLANCES.
86-21	07/08/86	F	53.5	A	2		HC	HTEXCH	UNIT OUTAGE DUE TO HIGH CONDUCTIVITY -- CONDENSER TUBE LEAK.
86-22	07/19/86	S	0.0	H	5		HC	HTEXCH	REDUCED LOAD TO 700 MWE TO REVERSE CONDENSER FLOW.
86-23	07/22/86	S	0.0	H	5		ZZ	ZZZZZZ	REDUCED LOAD TO 700 MWE AS PER LOAD DISPATCHER.
86-24	07/26/86	S	0.0	B	5		HA	TURBIN	REDUCED LOAD TO 700 MWE FOR TURBINE SURVEILLANCE.

****
* SUMMARY *
****

QUAD CITIES 1 HAD 1 OUTAGE AND 4 POWER REDUCTIONS IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* QUAD CITIES 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....ROCK ISLAND

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...20 MI NE OF
MOLINE, ILL

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...OCTOBER 18, 1971

DATE ELEC ENER 1ST GENER...APRIL 12, 1972

DATE COMMERCIAL OPERATE....FEBRUARY 18, 1973

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....A. MADISON

LICENSING PROJ MANAGER.....T. ROTELLA
DOCKET NUMBER.....50-254

LICENSE & DATE ISSUANCE....DPR-29, DECEMBER 14, 1972

PUBLIC DOCUMENT ROOM.....MOLINE PUBLIC LIBRARY
504 17TH STREET
MOLINE, ILLINOIS 61265

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON MAY 28-30 AND JUNE 4-5 AND 17 (86012; 86011): ROUTINE, UNANNOUNCED INSPECTION OF GASEOUS AND LIQUID RADIOACTIVE EFFLUENTS INCLUDING: EFFLUENT RELEASES; RECORDS AND REPORTS OF EFFLUENTS; EFFLUENT CONTROL INSTRUMENTATION; PROCEDURES FOR CONTROLLING RELEASES; REACTOR COOLANT CHEMISTRY AND ACTIVITY; GASEOUS EFFLUENT FILTRATION; AND AUDITS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

OTHER ITEMS

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

H. TOREN, SITE EMERGENCY PLANNING COORDINATOR SINCE OCTOBER 1985, WILL BE LEAVING QUAD-CITIES NUCLEAR PLANT ON MARCH 21, 1986. HIS REPLACEMENT HAS NOT YET BEEN SELECTED BY MANAGEMENT.

PLANT STATUS:

OPERATING ROUTINELY

LAST IE SITE INSPECTION DATE: 10/04/86

INSPECTION REPORT NO: 86014

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-01	01/06/86	07/09/86	LEAK RATE FROM ALL VALVES AND PENETRATIONS ON UNIT ONE IN EXCESS OF TECHNICAL SPECIFICATION LIMIT
86-02	01/07/86	06/11/86	LEAK RATE FOR MSIV (MAIN STEAM ISOLATION VALVE) IN EXCESS OF TECHNICAL SPECIFICATION LIMIT

1. Docket: 50-265 O P E R A T I N G S T A T U S

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: CAROL KRONICH (309) 654-2241 X193

4. Licensed Thermal Power (MWt): 2511

5. Nameplate Rating (Gross MWe): 920 X 0.9 = 828

6. Design Electrical Rating (Net MWe): 789

7. Maximum Dependable Capacity (Gross MWe): 813

8. Maximum Dependable Capacity (Net MWe): 769

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	123,765.0
13. Hours Reactor Critical	744.0	4,757.7	96,025.7
14. Rx Reserve Shtdwn Hrs	.0	.0	2,985.8
15. Hrs Generator On-Line	744.0	4,715.3	93,013.0
16. Unit Reserve Shtdwn Hrs	.0	.0	702.9
17. Gross Therm Ener (MWH)	1,805,766	11,234,663	197,343,686
18. Gross Elec Ener (MWH)	571,627	3,655,476	63,072,248
19. Net Elec Ener (MWH)	546,418	3,498,990	59,374,655
20. Unit Service Factor	100.0	92.7	75.2
21. Unit Avail Factor	100.0	92.7	75.7
22. Unit Cap Factor (MDC Net)	95.5	89.4	62.4
23. Unit Cap Factor (DER Net)	93.1	87.2	60.8
24. Unit Forced Outage Rate	.0	.8	7.7
25. Forced Outage Hours	.0	39.4	3,857.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

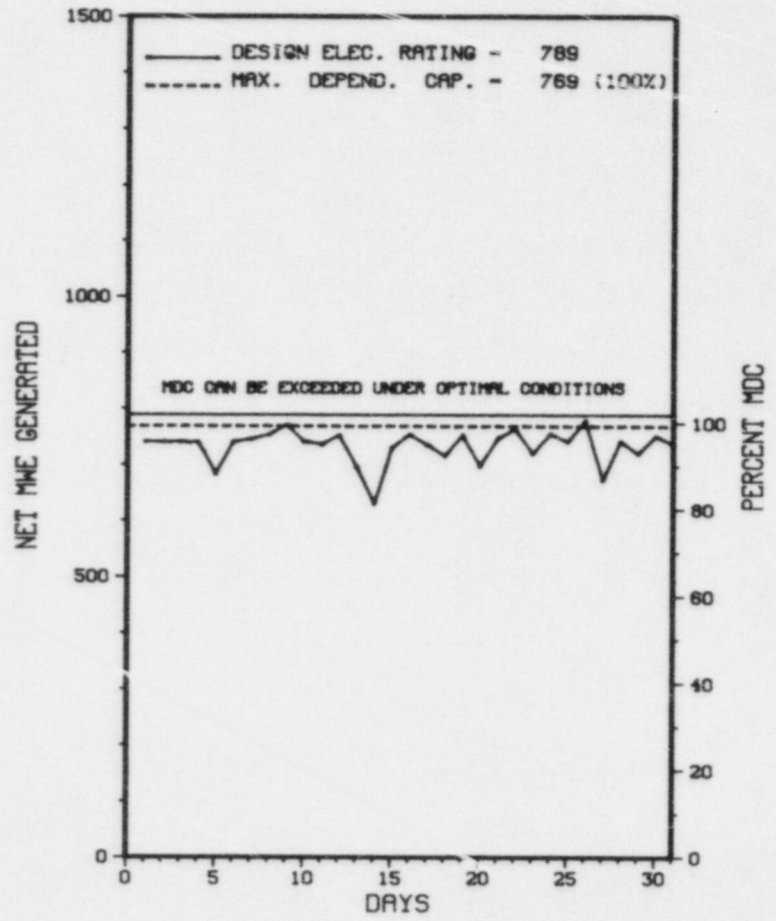
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* QUAD CITIES 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

QUAD CITIES 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

QUAD CITIES 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-22	07/05/86	S	0.0	B	5		HH	DEMINX	REDUCED LOAD TO 650 MWE -- CONDENSATE DEMINERALIZER MAINTENANCE.
86-23	07/13/86	S	0.0	H	5		RC	CONROD	REDUCED LOAD TO 650 MWE FOR A CONTROL ROD PATTERN ADJUSTMENT.
86-24	07/14/86	F	0.0	H	5		CB	MOTORX	REDUCED LOAD TO 420 MWE DUE TO RECIRCULATION MOTOR-GENERATOR SET PROBLEMS.
86-25	07/17/86	S	0.0	H	5		HF	PUMPXX	REDUCED LOAD TO 715 MWE TO TAKE 2A CIRCULATION WATER PUMP OUT OF SERVICE.
86-26	07/19/86	S	0.0	B	5		CH	PUMPXX	REDUCED LOAD TO 600 MWE TO CHANGE OVER REACTOR FEED PUMPS.
86-27	07/22/86	S	0.0	H	5		ZZ	ZZZZZZ	REDUCED LOAD TO 700 MWE AS PER LOAD DISPATCHER.
86-28	07/24/86	S	0.0	B	5		HH	DEMINX	REDUCED LOAD TO 715 MWE TO REMOVE 2E CONDENSATE DEMINERALIZER FROM SERVICE.
86-29	07/26/86	S	0.0	B	5		HA	TURBIN	REDUCED LOAD TO 735 MWE FOR TRUBINE SURVEILLANCES.

SUMMARY

QUAD CITIES 2 HAD NO OUTAGES AND SEVERAL POWER REDUCTIONS IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* QUAD CITIES 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....ROCK ISLAND

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...20 MI NE OF
MOLINE, ILL

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...APRIL 26, 1972

DATE ELEC ENER 1ST GENER...MAY 23, 1972

DATE COMMERCIAL OPERATE....MARCH 10, 1973

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....A. MADISON

LICENSING PROJ MANAGER.....T. ROTELLA
DOCKET NUMBER.....50-265

LICENSE & DATE ISSUANCE....DPR-30, DECEMBER 14, 1972

PUBLIC DOCUMENT ROOM.....MOLINE PUBLIC LIBRARY
504 17TH STREET
MOLINE, ILLINOIS 61265

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON MAY 28-30 AND JUNE 4-5 AND 17 (86012; 86011): ROUTINE, UNANNOUNCED INSPECTION OF GASEOUS AND LIQUID RADIOACTIVE EFFLUENTS INCLUDING: EFFLUENT RELEASES; RECORDS AND REPORTS OF EFFLUENTS; EFFLUENT CONTROL INSTRUMENTATION; PROCEDURES FOR CONTROLLING RELEASES; REACTOR COOLANT CHEMISTRY AND ACTIVITY; GASEOUS EFFLUENT FILTRATION; AND AUDITS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

OTHER ITEMS

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

H. TOREN, SITE EMERGENCY PLANNING COORDINATOR SINCE OCTOBER 1985, WILL BE LEAVING QUAD-CITIES NUCLEAR PLANT ON MARCH 21, 1986. HIS REPLACEMENT HAS NOT YET BEEN SELECTED BY MANAGEMENT.

PLANT STATUS:

OPERATING ROUTINELY

LAST IE SITE INSPECTION DATE: 10/04/86

INSPECTION REPORT NO: 86013

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-09	06/26/86	07/16/86	2B CORE SPRAY ROOM COOLER INOPERABLE DUE TO DRIVE BELT FAILURE
86-10	06/27/86	07/22/86	UNIT 2 SPURIOUS CLEAN-UP SYSTEM ISOLATION CAUSED BY ELECTRICAL SPIKE DUE TO PERSONNEL ERROR

1. Docket: 50-312 O P E R A T I N G S T A T U S

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: RON COLOMBO (916) 452-3211

4. Licensed Thermal Power (MWt): 2772

5. Nameplate Rating (Gross MWe): 1070 X 0.9 = 963

6. Design Electrical Rating (Net MWe): 918

7. Maximum Dependable Capacity (Gross MWe): 917

8. Maximum Dependable Capacity (Net MWe): 873

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	98,952.0
13. Hours Reactor Critical	.0	.0	52,565.0
14. Rx Reserve Shtdwn Hrs	.0	.0	10,647.7
15. Hrs Generator On-Line	.0	.0	50,363.8
16. Unit Reserve Shtdwn Hrs	.0	.0	1,210.2
17. Gross Therm Ener (MWH)	0	0	124,228,535
18. Gross Elec Ener (MWH)	0	0	41,528,149
19. Net Elec Ener (MWH)	-3,010	-14,641	39,063,470
20. Unit Service Factor	.0	.0	50.9
21. Unit Avail Factor	.0	.0	52.1
22. Unit Cap Factor (MDC Net)	.0	.0	45.2
23. Unit Cap Factor (DER Net)	.0	.0	43.0
24. Unit Forced Outage Rate	100.0	100.0	34.5
25. Forced Outage Hours	744.0	5,087.0	26,472.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

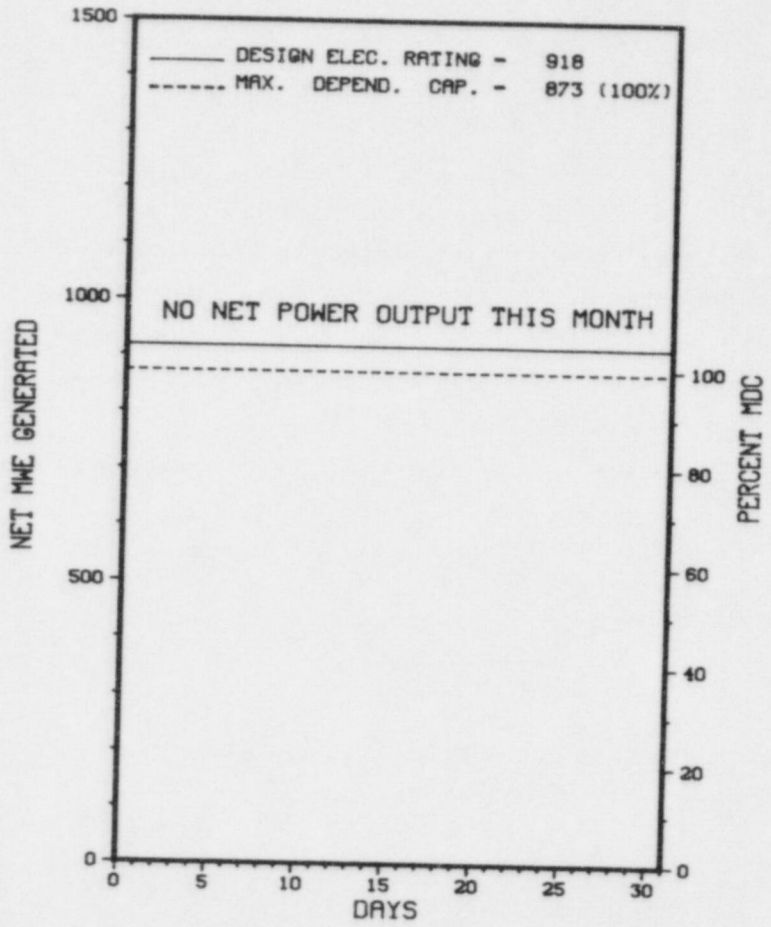
NONE

27. If Currently Shutdown Estimated Startup Date: 12/25/86

****
* RANCHO SECO 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

RANCHO SECO 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

RANCHO SECO 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	12/26/86	F	744.0	A	4	85-25	CB	INSTRU	REACTOR TRIP ON HIGH PRESSURE PRECEDED BY TOTAL LOSS OF ICS POWER. CORRECTIVE ACTIONS BEING IMPLEMENTED.

SUMMARY

RANCHO SECO 1 REMAINS SHUT DOWN FOLLOWING A REACTOR TRIP ON DECEMBER 26, 1985.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report (LER) File (NUREG-0161)
			9-Other	

****
* RANCHO SECO 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....CALIFORNIA

COUNTY.....SACRAMENTO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI SE OF
SACRAMENTO, CA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...SEPTEMBER 16, 1974

DATE ELEC ENER 1ST GENER...OCTOBER 13, 1974

DATE COMMERCIAL OPERATE....APRIL 17, 1975

CONDENSER COOLING METHOD...COOLING TOWERS

CONDENSER COOLING WATER....FOLSOM CANAL

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....SACRAMENTO MUN. UTIL. DISTRICT

CORPORATE ADDRESS.....6201 S STREET P.O. BOX 15830
SACRAMENTO, CALIFORNIA 95813

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....J. ECKHARD

LICENSING PROJ MANAGER.....S. MINER
DOCKET NUMBER.....50-312

LICENSE & DATE ISSUANCE....DPR-54, AUGUST 16, 1974

PUBLIC DOCUMENT ROOM.....BUSINESS AND MUNICIPAL DEPARTMENT
SACRAMENTO LIBRARY
828 I STREET
SACRAMENTO, CALIFORNIA 95814

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON MAY 27 - JULY 9, 1986 (REPORT NO. 50-312/86-19) AREAS INSPECTED: SECURITY PLAN AND IMPLEMENTING PROCEDURES; PHYSICAL BARRIERS; AND LIGHTING. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JUNE 2 - JULY 11, 1986 (REPORT NO. 50-312/86-21) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JUNE 10 - JULY 11, 1986 (REPORT NO.50-312/86-22) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ ENFORCEMENT CONFERENCE ON JUNE 20, 1986 (REPORT NO. 50-312/86-23) SUMMARY: THE FOLLOWING TOPICS WERE DISCUSSED: 1. APPARENT VIOLATIONS IDENTIFIED DURING INSPECTIONS OF THE EMERGENCY PREPAREDNESS PROGRAM AND RADIOACTIVE EFFLUENT MANAGEMENT (INSPECTION REPORT NOS. 50-312/86-14 AND 50-312/86-15); 2. MATTERS OF CONCERN TO THE NRC.

+ INSPECTION ON JUNE 24 - JULY 17, 1986 (REPORT NO. 50-312/86-24) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JULY 7-11, 1986 (REPORT NO. 50-312/86-25) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION BY REGIONALLY BASED INSPECTOR OF LICENSEE ACTION ON INSPECTOR-IDENTIFIED ITEMS AND IMPLEMENTATION OF QUALITY ASSURANCE PROGRAM REQUIREMENTS. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

INSPECTION SUMMARY

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JULY 14 - AUGUST 6, 1986 (REPORT NO. 50-312/86-27) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON AUGUST 4-15, 1986 (REPORT NO. 50-312/86-28) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JUNE 1, 1985 - JUNE 30, 1986 (REPORT NO. 50-312/86-29) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JULY 14 - AUGUST 15, 1986 (REPORT NO. 50-312/86-30) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JULY 30-31, 1986 (REPORT NO. 50-312/86-31) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

RANCHO SECO T.S. 6.8.1.A REQUIRES THAT THE APPLICABLE PROCEDURES RECOMMENDED IN APP. A OF REGULATORY GUIDE 1.33 (NOVEMBER, 1972) SHALL BE ESTABLISHED. SECTION C OF APP. A, REGULATORY GUIDE 1.33, RECOMMENDS PROCEDURES FOR STARTUP, OPERATION, AND SHUTDOWN OF SAFETY-RELATED PWR SYSTEMS INCLUDING INSTRUCTIONS FOR CHANGING MODES OF OPERATION. SECTION F OF APP. A, REGULATORY GUIDE 1.33 (NOVEMBER 1972) RECOMMENDS PROCEDURES FOR COMBATING EMERGENCIES AND OTHER SIGNIFICANT EVENTS. CONTRARY TO THE REQUIREMENT: 1) ON DECEMBER 26, 1985, NO WRITTEN PROCEDURES EXISTED FOR THE EXERCISE OF THE MANUALLY OPERATED AUXILIARY FEEDWATER SYSTEM VALVE FWS-063, WHICH WAS NORMALLY IDLE; 2) THE LICENSEE GUIDANCE FOR LOSS OF 120 VAC AND 125 VDC POWER SUPPLIES, LOSS OF FLUX INDICATION, TORNADO, DAM FAILURE, AND IRRADIATED FUEL DAMAGE WHILE REFUELING DID NOT HAVE THE ELEMENTS CONTAINED IN SECTION 5.3.8.1 OF ANSI 18.7-1972 WHICH IS REFERENCED BY REGULATORY GUIDE 1.33; 3) NO WRITTEN PROCEDURES FOR MANUAL EMERGENCY OPERATION OF AUXILIARY FEEDWATER CONTROL VALVE FV20527 EXISTED DURING THE DECEMBER 26, 1985 LOSS OF ICS POWER EVENT. CONSEQUENTLY THE VALVE WAS OPERATED USING EXCESSIVE LEVERAGE ON THE MANUAL OPERATOR; 4) DURING THE COOLDOWN TRANSIENT OF DECEMBER 26, 1985, AFW PUMP P-318 WAS NOT STOPPED WHEN THE LEVEL IN 'A' OTSG REACHED 95%. ALSO, THE AFW FLOW WAS NOT STOPPED TO THE STEAM GENERATORS TO PREVENT EXCESSIVE HEAT TRANSFER. CONSEQUENTLY, WATER OVERFLOWED INTO THE MAIN STEAM LINES OF THE 'A' OTSG AND THE REACTOR COOLANT SYSTEM WAS EXCESSIVELY COOLED; 5) NO WRITTEN PROCEDURES HAD BEEN ESTABLISHED FOR SECURING THE SAFETY-RELATED HIGH PRESSURE INJECTION SYSTEM FOLLOWING SAFETY FEATURES ACTUATION. CONSEQUENTLY, DURING THE DECEMBER 26TH EVENT, THE MAKEUP AUXILIARY PUMP (P-236) CONTINUED TO OPERATE WITHOUT A SOURCE OF COOLANT, WAS DAMAGED AND PRIMARY COOLANT WAS RELEASED TO THE AUXILIARY BUILDING; 6) NO WRITTEN PROCEDURES FOR THE LOSS OF POWER TO THE INTEGRATED CONTROL SYSTEM HAD BEEN ESTABLISHED FOR USE AT RANCHO SECO ON DECEMBER 26, 1985, WHEN LOSS OF DC POWER SUPPLY TO THE INTEGRATED CONTROL SYSTEM OCCURRED. THE LOSS OF POWER RESULTED IN A SIGNIFICANT PLANT TRANSIENT THAT INVOLVED EXCESSIVE COOLDOWN OF THE PRIMARY COOLANT SYSTEM. TECHNICAL SPECIFICATION 3.13.1 REQUIRES THE CONTROL ROOM/TECHNICAL SUPPORT CENTER (CR/TSC) EMERGENCY FILTER SYSTEM TO BE OPERABLE AT ALL TIMES WHEN CONTAINMENT INTEGRITY IS REQUIRED. TECHNICAL SPECIFICATION 4.10 REQUIRES A SYSTEM AIR FLOW OF 3200 CFM (+/-10% DURING PERIODIC TESTING OF THE CONTROL ROOM/TECHNICAL SUPPORT CENTER EMERGENCY FILTERING SYSTEM TO DEMONSTRATE THAT THE SYSTEM IS OPERABLE. CONTRARY TO THE REQUIREMENT, ON OCTOBER 1, 1985, OPERATING AIRFLOW RATES THROUGH THE CR/TSC EMERGENCY FILTRATION SYSTEM WERE MEASURED TO RANGE FROM 3581 TO 4170 CFM DURING THE ENTIRE DURATION OF THE 24 MINUTE SURVEILLANCE TEST. FURTHERMORE, DURING ACTUAL EMERGENCY OPERATION OF THE 'B' TRAIN ON DECEMBER 26, 1985, WHILE CONTAINMENT INTEGRITY WAS REQUIRED AIR FLOW RATES RANGED FROM 4071 TO 4575 CFM DURING EMERGENCY OPERATION FOR 8 MINUTES. NO CORRECTIVE ACTIONS WERE TAKEN TO CORRECT THE EXCESSIVE AIR FLOW RATES AFTER THE OCTOBER 1, 1985, SURVEILLANCE TEST.
(8600 3)

RANCHO SECO TECHNICAL SPECIFICATION 6.8.1.A REQUIRES THAT THE APPLICABLE PROCEDURES RECOMMENDED IN APPENDIX A OF REGULATORY GUIDE 1.33 (NOVEMBER, 1972) SHALL BE ESTABLISHED. SECTION H OF APPENDIX A, REGULATORY GUIDE 1.33 RECOMMENDS THAT "PROCEDURES OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES SHOULD BE PROVIDED TO ASSURE THAT TOOLS, GAUGES, INSTRUMENTS, CONTROLS, AND OTHER MEASURING AND TESTING DEVICES ARE PROPERLY CONTROLLED, CALIBRATED, AND ADJUSTED AT SPECIFIED PERIODS TO MAINTAIN ACCURACY." CONTRARY TO THE REQUIREMENT, ON APRIL 11, 1986, NO WRITTEN PROCEDURES EXISTED SPECIFICALLY ADDRESSING CRIMPING TOOL CALIBRATION AND CONTROL.
(8600 4)

****
* RANCHO SECO 1 *
****

ENFORCEMENT SUMMARY

RANCHO SECO TECHNICAL SPECIFICATION 6.5.1.6.D STATES THAT THE PRC SHALL BE RESPONSIBLE FOR REVIEW OF ALL PROPOSED CHANGES OR MODIFICATIONS TO PLANT SYSTEMS OR EQUIPMENT THAT AFFECT NUCLEAR SAFETY. CONTRARY TO THE REQUIREMENT, THE PRC DID NOT REVIEW NRC #5218, WHICH MODIFIED A REACTOR COOLANT SYSTEM PRESSURE BOUNDARY VALVE. (8601 4)

10 CFR 50.59(B) STATES, IN PART, "...THE LICENSEE SHALL FURNISH ... ANNUALLY OR AT SUCH SHORTER INTERVALS AS MAY BE SPECIFIED IN THE LICENSE, A REPORT CONTAINING A BRIEF DESCRIPTION OF SUCH CHANGES ... INCLUDING A SUMMARY OF THE SAFETY EVALUATION OF EACH." PARAGRAPH 6.9.3 OF THE RANCHO SECO TECHNICAL SPECIFICATIONS SPECIFIES, IN PART, "... TABULATION OF FACILITY CHANGES, TESTS OR EXPERIMENTS REQUIRED PURSUANT TO 10 CFR 50.59(B), SHALL BE SUBMITTED ON A MONTHLY BASIS ..." CONTRARY TO THE REQUIREMENT, THE LICENSEE'S MONTHLY REPORT TABULATIONS FOR 10 CFR 50.59 MODIFICATIONS DO NOT MEET, IN MANY CASES, THE REPORTING REQUIREMENTS OF ANNUAL OR SHORTER INTERVAL; NOR DO THEY MEET THE REQUIREMENT FOR A SUMMARY SAFETY EVALUATION TO BE INCLUDED WITH EACH 10 CFR 50.59 CHANGE DESCRIPTION. RANCHO SECO TECHNICAL SPECIFICATION 6.8.1 STATES IN PART: "WRITTEN PROCEDURES SHOULD BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING ... THE APPLICABLE PROCEDURES RECOMMENDED IN APP. A OF REG. GUIDE 1.33, NOVEMBER 1972." PARAGRAPH A.3 RECOMMENDS ADMINISTRATIVE PROCEDURES FOR EQUIPMENT CONTROL. CONTRARY TO THE REQUIREMENT, ON DECEMBER 8, 1985, NO ABNORMAL TAG WAS HUNG ON VALVE SFV-22006 AFTER BEING TEMPORARILY MODIFIED AND RETURNED TO SERVICE UNDER NCR #5218. (8601 5)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

INTEGRATED CONTROL SYSTEM UNDER REVIEW FOLLOWING EXCESSIVE COOLDOWN RATE PRODUCED WHEN SYSTEM LOST DC POWER AT 100% REACTOR POWER.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

RESTART ACTION PLAN IMPLEMENTATION IS BEING REVIEWED BY NRR AND REGION V PRIOR TO RETURN TO OPERATION.

ENFORCEMENT CONFERENCE WAS HELD ON MAY 16, 1986, ON VIOLATIONS RELATED TO THE DECEMBER 26, 1985, EVENT AND FOLLOWUP ACTIVITIES.

+ NRC SALP BOARD MEETING WAS HELD ON AUGUST 12, 1986.

PLANT STATUS:

PLANT REMAINS IN COLD SHUTDOWN FOLLOWING PLANT TRIP FROM 100% DUE TO LOSS OF ICS DC POWER ON DECEMBER 26, 1985.

LAST IE SITE INSPECTION DATE: 07/14-08/15/86+

INSPECTION REPORT NO: 50-312/86-30+

REPORTS FROM LICENSEE

RANCHO SECO 1

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-01-LO	12-18-85	01-14-86	DIESEL GENERATORS 'A' AND 'B' DECLARED INOPERABLE DURING COLD SHUTDOWN
86-06-LO	04-14-86	05-12-86	FIRE PROTECTION SYSTEMS OVERDUE FOR SURVEILLANCE
86-07-LO	04-14-86	05-19-86	CONTROL ROOM EMERGENCY HEATING, VENTILATING, AND AIR CONDITIONING HIGH FLOW CONDITION
86-08-LO	04-29-86	05-29-86	FAILURE TO INSTITUTE FIRE WATCH FOR BREACHED PENETRATION
86-09-LO	05-09-86	06-09-86	INADVERTENT ENGINEERED SAFETY FEATURES SYSTEM ACTUATIONS
86-10-LO	05-22-86	06-19-86	REDUNDANT CABLING IN THE SAME FIRE AREA

1. Docket: 50-458 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: J. H. SIMMONS (504) 635-6094

4. Licensed Thermal Power (MWt): 2894

5. Nameplate Rating (Gross MWe): 2894

6. Design Electrical Rating (Net MWe): 936

7. Maximum Dependable Capacity (Gross MWe): 936

8. Maximum Dependable Capacity (Net MWe): 936

9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	4,343.0	4,343.0
13. Hours Reactor Critical	469.5	2,755.4	2,755.4
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	400.3	2,338.7	2,338.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,076,579	4,769,129	4,769,129
18. Gross Elec Ener (MWH)	353,301	1,506,369	1,506,369
19. Net Elec Ener (MWH)	329,568	1,378,481	1,378,481
20. Unit Service Factor			
21. Unit Avail Factor		NOT IN	
22. Unit Cap Factor (MDC Net)		COMMERCIAL	
23. Unit Cap Factor (DER Net)		OPERATION	
24. Unit Forced Outage Rate			
25. Forced Outage Hours	343.7	1,013.1	1,013.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

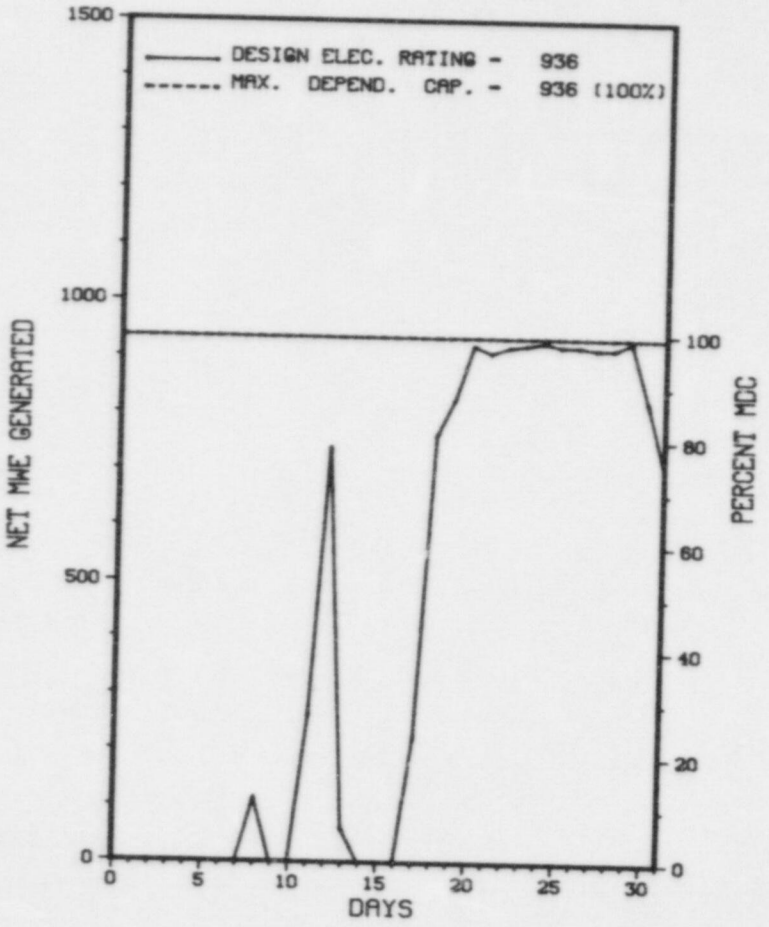
TURBINE TORSIONAL VIBRATION TEST: AUGUST 15, 1986, 12 DAYS

27. If Currently Shutdown Estimated Startup Date: N/A

****
* RIVER BEND 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

RIVER BEND 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

RIVER BEND 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
1	07/01/86	F	168.9	A	1				CONTINUED FROM JULY OUTAGE TO REPAIR EHC LINE LEAK AND TESTABLE CHECK VALVE LEAK IN DRYWELL.
2	07/08/86	F	71.9	A	3				MSR DRAIN TANK MANWAY GASKET BLEW OUT. TURBINE BUILDING HIGH AREA TEMPERATURE/MSIV ISOLATION AND REACTOR SCRAM.
3	07/13/86	F	102.9	A	3				STEAM SEAL EVAPORATOR LEVEL CONTROL FAILED TO MAINTAIN LEVEL. LOSS OF STEAM SEALS TO MAIN TURBINE RESULTED IN LOSS OF VACUUM, TURBINE TRIP AND REACTOR SCRAM.

SUMMARY

RIVER BEND 1 INCURRED 3 SHUTDOWNS IN JULY AS DISCUSSED ABOVE.

Type

F-Forced
S-Sched

Reason

A-Equip Failure
B-Maint or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Admin
G-Oper Error
H-Other

Method

1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other

System & Component

Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* RIVER BEND 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....LOUISIANA

COUNTY.....WEST FELICIANA

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...24 MI NNW OF
BATON ROUGE, LA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...OCTOBER 31, 1985

DATE ELEC ENER 1ST GENER...DECEMBER 3, 1985

DATE COMMERCIAL OPERATE....**********

CONDENSER COOLING METHOD...MDCT

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHWEST POWER POOL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....GULF STATES UTILITIES

CORPORATE ADDRESS.....P.O. BOX 2951
BEAUMONT, LOUISIANA 77704

CONTRACTOR
ARCHITECT/ENGINEER..... STONE & WEBSTER

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....D. CHAMBERLAIN

LICENSING PROJ MANAGER.....S. STERN
DOCKET NUMBER.....50-458

LICENSE & DATE ISSUANCE....NPF-47, NOVEMBER 20, 1985

PUBLIC DOCUMENT ROOM.....GOVERNMENT DOCUMENTS DEPARTMENT
TROY H. MIDDLETON LIBRARY
LOUISIANA STATE UNIVERSITY
BATON ROUGE, LOUISIANA 70803

INSPECTION STATUS

INSPECTION SUMMARY

INFO. NOT SUPPLIED BY REGION

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

INFO. NOT SUPPLIED BY REGION

FACILITY ITEMS (PLANS AND PROCEDURES):

INFO. NOT SUPPLIED BY REGION

MANAGERIAL ITEMS:

INFO. NOT SUPPLIED BY REGION

INSPECTION STATUS - (CONTINUED)

****
* RIVER BEND 1 *
****

PLANT STATUS:

INFO. NOT SUPPLIED BY REGION

LAST IE SITE INSPECTION DATE: INFO. NOT SUPPLIED BY REGION

INSPECTION REPORT NO: INFO. NOT SUPPLIED BY REGION

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
INFO. NOT SUPPLIED BY REGION			

1. Docket: 50-261 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: ANITA E. SCOTT (803) 383-4524

4. Licensed Thermal Power (MWt): 2300

5. Nameplate Rating (Gross MWe): 854 X 0.9 = 769

6. Design Electrical Rating (Net MWe): 700

7. Maximum Dependable Capacity (Gross MWe): 700

8. Maximum Dependable Capacity (Net MWe): 665

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	135,077.0
13. Hours Reactor Critical	744.0	3,708.1	95,764.7
14. Rx Reserve Shtdwn Hrs	.0	280.8	2,936.4
15. Hrs Generator On-Line	744.0	3,624.0	93,391.9
16. Unit Reserve Shtdwn Hrs	.0	.0	23.2
17. Gross Therm Ener (MWH)	1,693,508	7,653,284	187,145,197
18. Gross Elec Ener (MWH)	553,076	2,537,891	60,397,382
19. Net Elec Ener (MWH)	527,016	2,403,303	57,052,877
20. Unit Service Factor	100.0	71.2	69.1
21. Unit Avail Factor	100.0	71.2	69.2
22. Unit Cap Factor (MDC Net)	106.5	71.0	63.5
23. Unit Cap Factor (DER Net)	101.2	67.5	60.3
24. Unit Forced Outage Rate	.0	7.1	13.9
25. Forced Outage Hours	.0	278.8	9,323.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

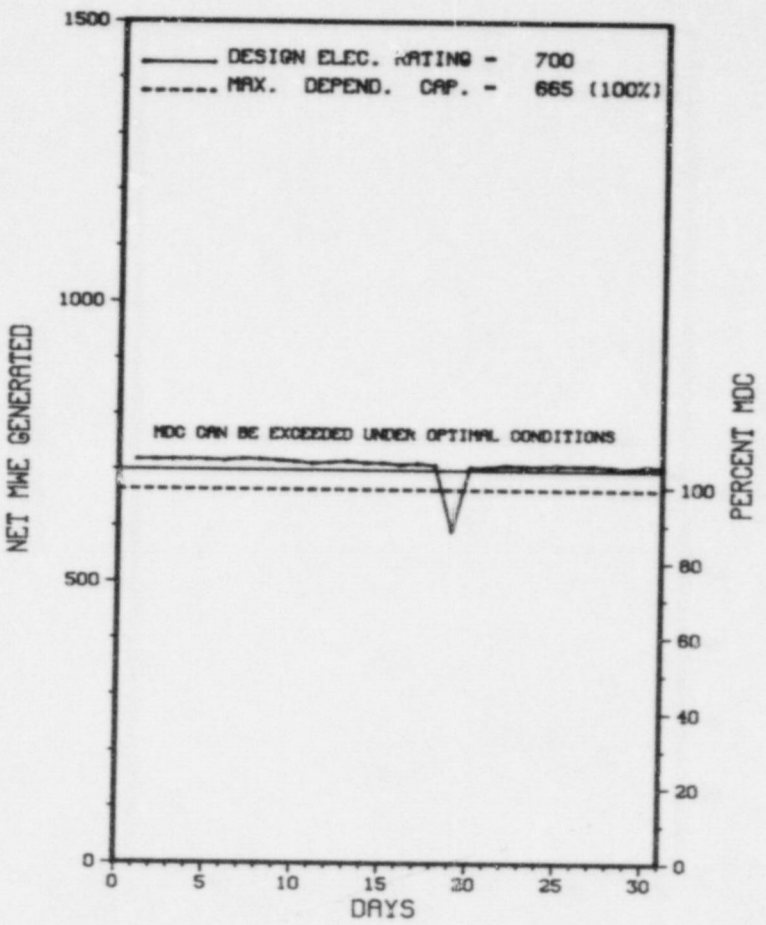
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* ROBINSON 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ROBINSON 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* ROBINSON 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

****
* SUMMARY *
****

ROBINSON 2 OPERATED ROUTINELY IN JULY WITH NO OUTAGES OR POWER REDUCTIONS REPORTED.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* ROBINSON 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....SOUTH CAROLINA

COUNTY.....DARLINGTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI NW OF
HARTSVILLE, SC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...SEPTEMBER 20, 1970

DATE ELEC ENER 1ST GENER...SEPTEMBER 26, 1970

DATE COMMERCIAL OPERATE....MARCH 7, 1971

CONDENSER COOLING METHOD...RECIRCULATION

CONDENSER COOLING WATER....ROBINSON IMPOUNDMENT

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....CAROLINA POWER & LIGHT

CORPORATE ADDRESS.....411 FAYETTEVILLE STREET
RALEIGH, NORTH CAROLINA 27601

CONTRACTOR
ARCHITECT/ENGINEER.....EBASCO

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....EBASCO

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....P. KRUG

LICENSING PROJ MANAGER.....G. REQUA
DOCKET NUMBER.....50-261

LICENSE & DATE ISSUANCE....DPR-23, SEPTEMBER 23, 1970

PUBLIC DOCUMENT ROOM.....HARTSVILLE MEMORIAL LIBRARY
220 N. FIFTH ST.
HARTSVILLE, SOUTH CAROLINA 29550

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION APRIL 28 - MAY 2 (86-11): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED ON SITE IN THE AREAS OF LICENSED OPERATOR TRAINING AND TRAINING OF NON-LICENSED PERSONNEL. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 11 - JULY 10 (86-15): THIS ROUTINE, ANNOUNCED INSPECTION WAS CONDUCTED IN THE AREAS OF TECHNICAL SPECIFICATION (TS) COMPLIANCE, PLANT TOUR, OPERATIONS PERFORMANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, SITE SECURITY, SURVEILLANCE ACTIVITIES, MAINTENANCE ACTIVITIES, QUALITY ASSURANCE PRACTICES, RADIATION CONTROL ACTIVITIES, OUTSTANDING ITEMS REVIEW, IE BULLETIN AND IE NOTICE FOLLOWUP, ORGANIZATION AND ADMINISTRATION, INDEPENDENT INSPECTION, SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP) AND ENFORCEMENT ACTION FOLLOWUP. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED WITHIN THE AREAS INSPECTED.

INSPECTION JUNE 9-13 (86-16): THIS ROUTINE, UNANNOUNCED INSPECTION WAS CONDUCTED IN THE AREAS OF FIRE PREVENTION/PROTECTION AND FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 9-13, 1986 +

INSPECTION REPORT NO: 50-261/86-16 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-272 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: PELL WHITE (609) 935-6000 X4451
4. Licensed Thermal Power (MWt): 3338
5. Nameplate Rating (Gross MWe): 1300 X 0.9 = 1170
6. Design Electrical Rating (Net MWe): 1090
7. Maximum Dependable Capacity (Gross MWe): 1124
8. Maximum Dependable Capacity (Net MWe): 1079
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	79,656.0
13. Hours Reactor Critical	595.4	3,741.7	47,927.1
14. Rx Reserve Shtdwn Hrs	.0	.0	3,088.4
15. Hrs Generator On-Line	589.0	3,585.1	46,089.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,897,838	11,421,037	141,907,272
18. Gross Elec Ener (MWH)	633,030	3,715,970	47,009,778
19. Net Elec Ener (MWH)	603,687	3,538,662	44,644,654
20. Unit Service Factor	79.2	70.5	57.9
21. Unit Avail Factor	79.2	70.5	57.9
22. Unit Cap Factor (MDC Net)	75.2	64.5	51.9
23. Unit Cap Factor (DER Net)	74.4	63.8	51.4
24. Unit Forced Outage Rate	20.8	10.1	28.8
25. Forced Outage Hours	155.0	402.9	18,891.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

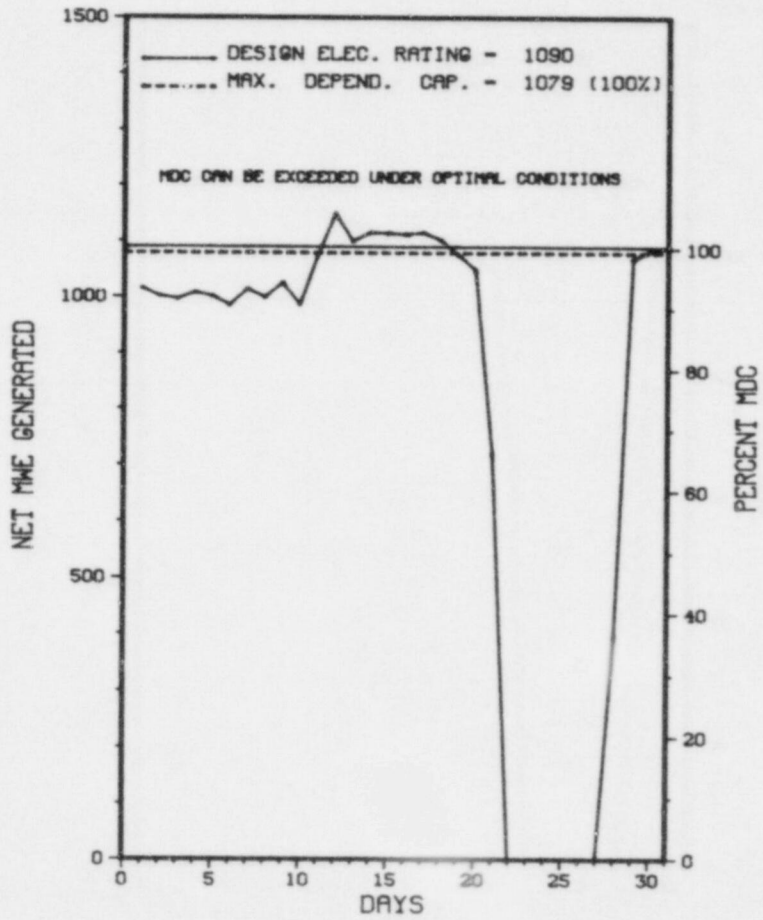
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* SALEM 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SALEM 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

SALEM 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
594	07/21/86	F	155.0	A	1		ZZ	GENERA	H2 LEAK ON GENERATOR.

SUMMARY

SALEM 1 HAD 1 OUTAGE IN JULY BECAUSE OF A GENERATOR HYDROGEN LEAK.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* SALEM 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....NEW JERSEY

COUNTY.....SALEM

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...20 MI S OF
WILMINGTON, DEL

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...DECEMBER 11, 1976

DATE ELEC ENER 1ST GENER...DECEMBER 25, 1976

DATE COMMERCIAL OPERATE....JUNE 30, 1977

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....DELAWARE RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PUBLIC SERVICE ELECTRIC & GAS

CORPORATE ADDRESS.....80 PARK PLACE
NEWARK, NEW JERSEY 07101

CONTRACTOR
ARCHITECT/ENGINEER.....PUBLIC SERVICES & GAS CO.

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....T. LINVILLE

LICENSING PROJ MANAGER.....D. FISCHER
DOCKET NUMBER.....50-272

LICENSE & DATE ISSUANCE....DPR-70, DECEMBER 1, 1976

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INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

REGULATORY GUIDE 1.33, NOVEMBER 1972, APPENDIX "A", PART J STATES IN PART "CHEMICAL AND RADIOCHEMICAL CONTROL PROCEDURES SHOULD BE WRITTEN TO PRESCRIBE THE NATURE AND FREQUENCY OF SAMPLING AND ANALYSIS, INSTRUCTIONS MAINTAINING WATER CHEMISTRY WITHIN PRESCRIBED LIMITS...AND THAT THESE PROCEDURES SHOULD SPECIFY LABORATORY INSTRUCTIONS AND CALIBRATION OF LABORATORY EQUIPMENT. TECHNICAL SPECIFICATIONS 6.8.1 STATES: WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING THE ACTIVITY REFERENCED BELOW: THE APPLICABLE PROCEDURES RECOMMENDED IN APPENDIX "A" OF REGULATORY GUIDE 1.33, NOVEMBER 1972. PSE&G CHEMISTRY ORGANIZATION MANUAL, CHAPTER 7, "CHEMISTRY QUALITY CONTROL," PART 4, "SELECTION AND CONTROL OF REAGENTS," STATES IN PART "CHEMICALS AND REAGENTS SHOULD HAVE EXPIRATION DATE STICKERS ON THEIR CONTAINERS. THE CHEMICALS SHOULD NOT BE USED AFTER THEIR EXPIRATION DATE." PART 5, "SELECTION AND CONTROL OF STANDARDS" STATES IN PART "SHELF LIFE SHALL BE AFFIXED TO STANDARDS AS WELL AS REAGENTS." CONTRARY TO THE ABOVE, ON MARCH 11, 1986 CHEMICALS THAT WERE OUT OF DATE WERE IDENTIFIED, BY THE INSPECTOR, ON THE CHEMISTRY LABORATORY SHELVES. FURTHER INVESTIGATION IDENTIFIED INSTANCES WHERE OUT OF DATE CHEMICAL WERE USED IN THE PRODUCTION OF STANDARDS WHICH WERE USED IN CHEMICAL ANALYSIS.

(8600 5)

ENFORCEMENT SUMMARY

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-311 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: PELL WHITE (609) 935-6000 X4451
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1162
6. Design Electrical Rating (Net MWe): 1115
7. Maximum Dependable Capacity (Gross MWe): 1149
8. Maximum Dependable Capacity (Net MWe): 1106
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	42,072.0
13. Hours Reactor Critical	673.2	4,536.5	24,862.2
14. Rx Reserve Shtdwn Hrs	.0	.0	3,533.6
15. Hrs Generator On-Line	655.3	4,478.3	24,015.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,147,440	14,810,717	74,556,415
18. Gross Elec Ener (MWH)	682,460	4,823,890	24,400,370
19. Net Elec Ener (MWH)	648,747	4,614,063	23,148,918
20. Unit Service Factor	88.1	88.0	57.1
21. Unit Avail Factor	88.1	88.0	57.1
22. Unit Cap Factor (MDC Net)	78.8	82.0	49.7
23. Unit Cap Factor (DER Net)	78.2	81.3	49.3
24. Unit Forced Outage Rate	11.9	11.8	36.7
25. Forced Outage Hours	88.7	600.0	13,895.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

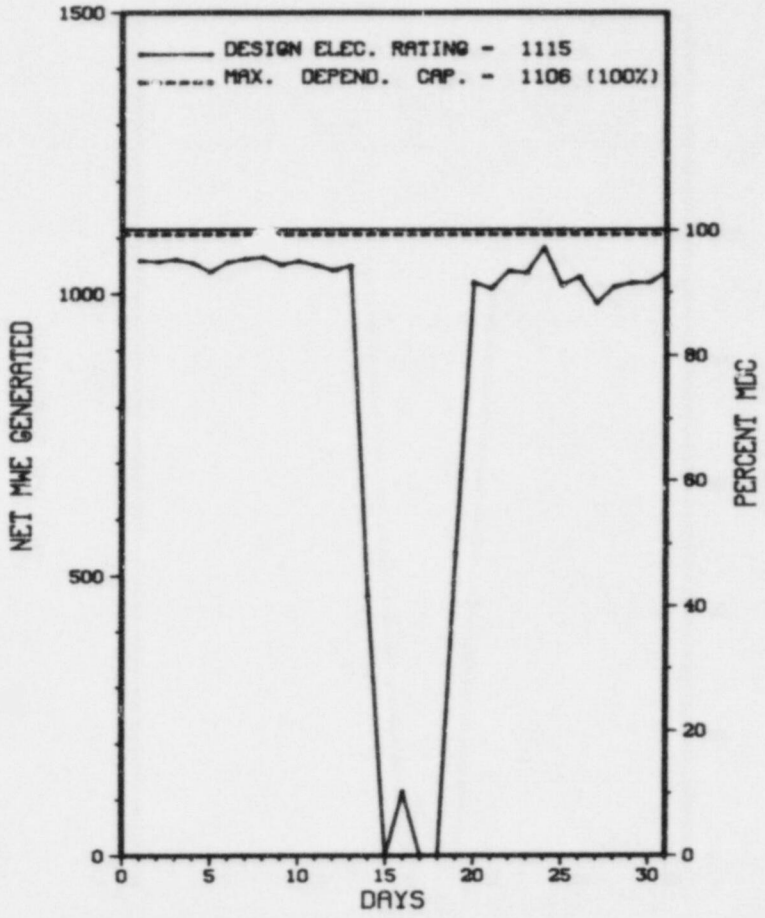
SCHEDULED REFUELING: 10/4/86 - 11/26/86.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* SALEM 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SALEM 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* SALEM 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
442	07/14/86	F	38.0	A	3		ED	GENERA	CONDUCTORS & BUSES AC INSTRUMENT POWER.
444	07/16/86	F	50.7	A	3		CC	PUMPXX	FEEDWATER PUMP/DRIVE LUBE OIL SYSTEM.

****
* SUMMARY *
****

SALEM 2 HAD 2 SHUTDOWNS IN JULY AS DESCRIBED ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Trainirg		5-Reduced Load	Licensee Event Report
	& License Examination		9-Other	(LER) File (NUREG-0161)

****
* SALEM 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....NEW JERSEY

COUNTY.....SALEM

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...20 MI S OF
WILMINGTON, DEL

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 8, 1980

DATE ELEC ENER 1ST GENER...JUNE 3, 1981

DATE COMMERCIAL OPERATE....OCTOBER 13, 1981

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....DELAWARE RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PUBLIC SERVICE ELECTRIC & GAS

CORPORATE ADDRESS.....80 PARK PLACE
NEWARK, NEW JERSEY 07101

CONTRACTOR
ARCHITECT/ENGINEER.....PUBLIC SERVICES & GAS CO.

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....T. LINVILLE

LICENSING PROJ MANAGER.....D. FISCHER
DOCKET NUMBER.....50-311

LICENSE & DATE ISSUANCE....DPR-75, MAY 20, 1981

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INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-206 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: E. R. SIACOR (714) 368-6223
4. Licensed Thermal Power (MWt): 1347
5. Nameplate Rating (Gross MWe): 500 X 0.9 = 450
6. Design Electrical Rating (Net MWe): 436
7. Maximum Dependable Capacity (Gross MWe): 456
8. Maximum Dependable Capacity (Net MWe): 436
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe): 390
11. Reasons for Restrictions, If Any:

STEAM GENERATOR TUBE CORROSION.

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	167,671.0
13. Hours Reactor Critical	271.5	271.5	96,384.7
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	125.2	125.2	92,500.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	75,740	75,740	117,386,864
18. Gross Elec Ener (MWH)	21,913	21,913	39,851,547
19. Net Elec Ener (MWH)	10,997	3,696	37,664,773
20. Unit Service Factor	16.8	2.5	55.2
21. Unit Avail Factor	16.8	2.5	55.2
22. Unit Cap Factor (MDC Net)	3.4	.2	51.5
23. Unit Cap Factor (DER Net)	3.4	.2	51.5
24. Unit Forced Outage Rate	.0	.0	21.1
25. Forced Outage Hours	.0	.0	12,129.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

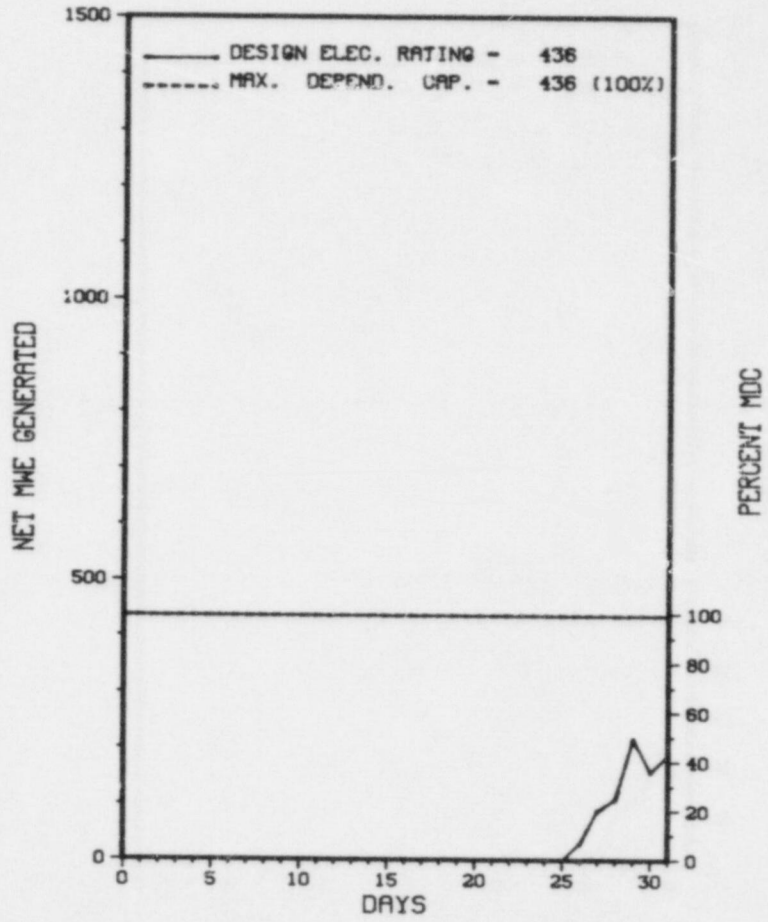
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* SAN ONOFRE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SAN ONOFRE 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

SAN ONOFRE 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
93	11/21/85	S	618.8	C	4				REFUELING/BACXFIT WORK.
94	07/29/86	S	0.0	A	5	86-007	BA	PT	POWER REDUCTION DUE TO FAILURE OF STEAM DENSITY COMPENSATOR PRESSURE TRANSMITTER PT-459. PT-459 HAS BEEN REPLACED AND RETURNED TO SERVICE.

SUMMARY

SAN ONOFRE 1 COMPLETED A REFUELING OUTAGE ON JULY 26.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* SAN ONOFRE 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....CALIFORNIA

COUNTY.....SAN DIEGO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI S OF
SAN CLEMENTE, CA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JUNE 14, 1967

DATE ELEC ENER 1ST GENER...JULY 16, 1967

DATE COMMERCIAL OPERATE....JANUARY 1, 1968

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....PACIFIC OCEAN

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....SOUTHERN CALIFORNIA EDISON

CORPORATE ADDRESS.....2244 WALNUT GROVE AVENUE
ROSEMEAD, CALIFORNIA 91770

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....A. DANGELO

LICENSING PROJ MANAGER.....R. DUDLEY
DOCKET NUMBER.....50-206

LICENSE & DATE ISSUANCE....DPR-13, MARCH 27, 1967

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INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON OCTOBER 1, 1984 - JUNE 18, 1986 (REPORT NO. 50-206/86-10) YEARLY SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE. LETTER ISSUED ON AUGUST 7, 1986.

+ INSPECTION ON JULY 22-25, 1986 (REPORT NO. 50-206/86-23) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON AUGUST 4-8, 1986 (REPORT NO. 50-206/86-32) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JUNE 28 - JULY 25, 1986 (REPORT NO. 50-206/86-34) AREAS INSPECTED: ROUTINE, RESIDENT INSPECTION OF OPERATIONS PROGRAM INCLUDING THE FOLLOWING AREAS: OPERATIONAL SAFETY VERIFICATION, EVALUATION OF PLANT TRIPS AND EVENTS, MONTHLY SURVEILLANCE ACTIVITIES, MONTHLY MAINTENANCE ACTIVITIES, PLANT STARTUP FROM REFUELING AND INDEPENDENT INSPECTION. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JULY 28 - AUGUST 15, 1986 (REPORT NO. 50-206/86-35) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

FAILURE TO MARK WELD DATUM AND REFERENCE POINTS FOR ISI UT EXAMS.
(8601 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

+ THE UNIT EXPERIENCED A FAILURE OF PRESSURE TRANSMITTER, PT-459, ON JULY 30, WHICH RESULTED IN ALL THREE STEAM FLOW/FEED FLOW MISMATCH TRIP CIRCUITS BECOMING INOPERABLE. BECAUSE OF THIS, AN UNUSUAL EVENT WAS DECLARED AND A TECHNICAL SPECIFICATION-REQUIRED SHUTDOWN WAS COMMENCED FROM 65% POWER. THE CAUSE OF THE FAILURE WAS DETERMINED TO BE A FAILED ELECTRONIC CARD IN THE AMPLIFIER FOR THE TRANSMITTER. THE AMPLIFIER WAS REPLACED, THE LICENSEE SECURED FROM THE UNUSUAL EVENT, AND THE SHUTDOWN WAS TERMINATED AT 31% POWER.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

+ THE UNIT, WHICH HAD BEEN IN ITS 8TH REFUELING AND MODIFICATION OUTAGE SINCE NOVEMBER, 1985, ENTERED MODE 3 ON JULY 6, AND RESUMED POWER OPERATION ON JULY 15. THE UNIT EXPERIENCED A TRIP DURING ITS RETURN TO FULL POWER ON JULY 17 WHEN THE OPERATORS PERFORMED A MANUAL TRANSFER OF VITAL BUS 4 POWER FROM THE BACKUP TO THE NORMAL SUPPLY. A VOLTAGE TRANSIENT RESULTED FROM THE BREAK BEFORE MAKE NATURE OF THE TRANSFER SWITCH DESIGN.

LAST IE SITE INSPECTION DATE: 07/28-08/15/86+

INSPECTION REPORT NO: 50-206/86-35+

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-361 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: E. R. SIACOR (714) 368-6223
4. Licensed Thermal Power (MWt): 3410
5. Nameplate Rating (Gross MWe): 1127
6. Design Electrical Rating (Net MWe): 1070
7. Maximum Dependable Capacity (Gross MWe): 1127
8. Maximum Dependable Capacity (Net MWe): 1070
9. If Changes Occur Above Since Last Report, Give Reasons:

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10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	26,136.0
13. Hours Reactor Critical	655.1	2,928.6	16,049.5
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	646.2	2,741.0	15,590.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,127,171	8,467,793	50,003,755
18. Gross Elec Ener (MWH)	721,383	2,816,636	16,786,836
19. Net Elec Ener (MWH)	682,756	2,642,145	15,838,784
20. Unit Service Factor	86.9	53.9	59.7
21. Unit Avail Factor	86.9	53.9	59.7
22. Unit Cap Factor (MDC Net)	85.8	48.5	56.6
23. Unit Cap Factor (DER Net)	85.8	48.5	56.6
24. Unit Forced Outage Rate	13.1	6.9	5.6
25. Forced Outage Hours	97.8	203.6	917.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

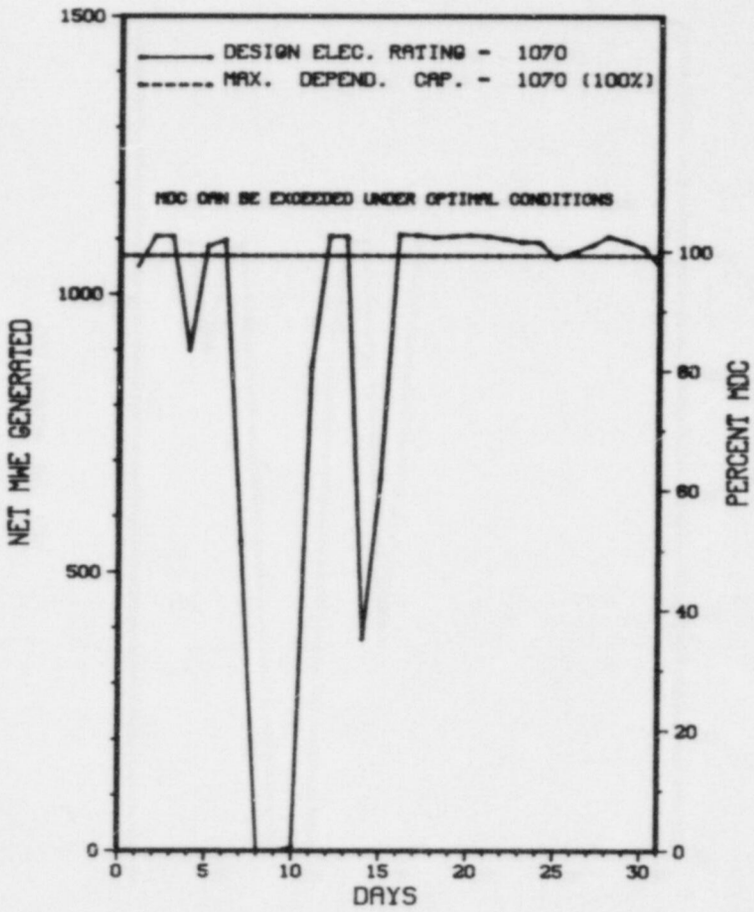
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* SAN ONOFRE 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SAN ONOFRE 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

SAN ONOFRE 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
27	07/07/86	F	77.5	A	3	86-018	AA	CHU	THE REACTOR TRIPPED FROM 47% POWER ON DEPARTURE FROM NUCLEATE BOILING RATIO/LOCAL POWER DENSITY (DNBR/LPD) AS A RESULT OF MISALIGNMENT PENALTY FACTORS GENERATED BY THE CORE PROTECTION CALCULATORS (CPC) WHEN CONTROL ELEMENT ASSEMBLY (CEA) 49 DROPPED TO ITS FULLY INSERTED POSITION IN ADDITION TO CEA 55 THAT HAD DROPPED PREVIOUSLY.
28	07/14/86	F	20.3	A	3	86-019	JM	RLY	A REACTOR TRIP FROM 100% POWER OCCURRED WHEN THE REACTOR COOLANT SYSTEM (RCS) PRESSURE REACHED THE CORE PROTECTION CALCULATORS (CPC) AUXILIARY TRIP SETPOINT.

SUMMARY

SAN ONOFRE 2 INCURRED 2 SHUTDOWNS IN JULY AS DISCUSSED ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SAN ONOFRE 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....CALIFORNIA

COUNTY.....SAN DIEGO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI S OF
SAN CLEMENTE, CA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JULY 26, 1982

DATE ELEC ENER 1ST GENER...SEPTEMBER 20, 1982

DATE COMMERCIAL OPERATE....AUGUST 8, 1983

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....PACIFIC OCEAN

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....SOUTHERN CALIFORNIA EDISON

CORPORATE ADDRESS.....P.O. BOX 800
ROSEMEAD, CALIFORNIA 91770

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC COM (ENG VERSION)

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....R. HUEY

LICENSING PROJ MANAGER.....H. ROOD
DOCKET NUMBER.....50-361

LICENSE & DATE ISSUANCE....NPF-10, SEPTEMBER 7, 1982

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INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON OCTOBER 1, 1984 - JUNE 18, 1986 (REPORT NO. 50-361/86-09) YEARLY SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE. LETTER ISSUED ON AUGUST 7, 1986.

+ INSPECTION ON MAY 1 - JULY 5, 1986 (REPORT NO. 50-361/86-19) AREAS INSPECTED: ROUTINE, RESIDENT INSPECTION OF OPERATIONS PROGRAM INCLUDING THE FOLLOWING AREAS: OPERATIONAL SAFETY VERIFICATION, EVALUATION OF PLANT TRIPS AND EVENTS, MONTHLY SURVEILLANCE ACTIVITIES, MONTHLY MAINTENANCE ACTIVITIES, REFUELING ACTIVITIES, INDEPENDENT INSPECTION, LICENSEE EVENTS REPORT REVIEW, AND FOLLOWUP OF PREVIOUSLY IDENTIFIED ITEMS. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON AUGUST 4-8, 1986 (REPORT NO. 50-361/86-22) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ ENFORCEMENT CONFERENCE ON JULY 26, 1986 (REPORT NO. 50-361/86-23) 1. DESCRIPTION OF THE VIOLATIONS ASSOCIATED WITH THE APRIL 21 - JUNE 2, 1986, INSPECTION AND DOCUMENTED IN INSPECTION REPORT NO. 50-206/86-17, 50-361/86-12, 50-362/86-12 (IE-V-748), DATED JUNE 12, 1986; 2. NRC CONCERNS; 3. LICENSEE CORRECTIVE ACTIONS PLANNED AND IMPLEMENTED.

+ INSPECTION ON JUNE 28 - AUGUST 15, 1986 (REPORT NO. 50-361/86-24) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON SEPTEMBER 15-26, 1986 (REPORT NO. 50-361/86-25) INSPECTION TO BE COMPLETED IN SEPTEMBER, 1986.

****
* SAN ONOFRE 2 *
****

INSPECTION SUMMARY

+ INSPECTION ON JULY 28 - AUGUST 15, 1986 (REPORT NO. 50-361/86-26) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

+ THE UNIT EXPERIENCED SEVERAL CONTROL ELEMENT ASSEMBLY (CEA) DROPS DURING JULY. THE FIRST CEA DROP OCCURRED ON JULY 25, AND POWER WAS REDUCED FROM 98% TO 87%. THE SECOND CEA DROP OCCURRED ON JULY 29, AND POWER WAS REDUCED FROM 100% TO 82%. NORMAL CEA RECOVERIES WERE PERFORMED IN BOTH CASES.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

+ THE UNIT CONTINUED FULL POWER OPERATIONS IN JULY. A REACTOR TRIP OCCURRED ON JULY 14 FROM A CONTAINMENT ISOLATION ACTUATION SYSTEM SIGNAL DUE TO A FAILED RELAY. THE RELAY WAS REPLACED AND NORMAL POWER OPERATIONS WERE CONTINUED.

LAST IE SITE INSPECTION DATE: 09/15-26/86+

INSPECTION REPORT NO: 50-361/86-25+

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-11-L0	06-07-86	06-27-86	SPURIOUS CPIS
86-12-L0	06-03-86	06-27-86	PPS ACTUATION ON LOW REACTOR COOLANT SYSTEM FLOW
86-13-L0	06-10-86	07-08-86	REACTOR TRIP ON HIGH STEAM GENERATOR LEVEL
86-14-L0	06-13-86	06-30-86	125 VDC BATTERY SURVEILLANCE

1. Docket: 50-362 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: E. R. SIACOR (714) 368-6223
4. Licensed Thermal Power (MWt): 3390
5. Nameplate Rating (Gross MWe): 1127
6. Design Electrical Rating (Net MWe): 1080
7. Maximum Dependable Capacity (Gross MWe): 1127
8. Maximum Dependable Capacity (Net MWe): 1080
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

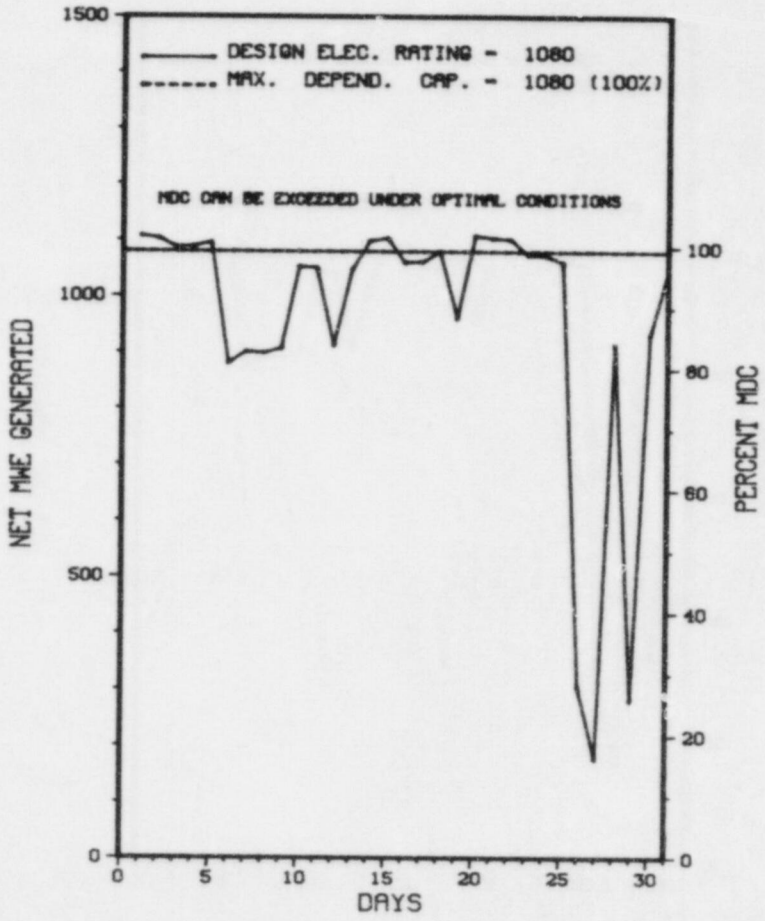
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	20,447.0
13. Hours Reactor Critical	726.6	4,280.9	13,466.0
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	714.7	3,959.8	12,775.1
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,129,940	12,518,254	37,517,818
18. Gross Elec Ener (MWH)	751,978	4,247,764	12,619,166
19. Net Elec Ener (MWH)	712,797	3,996,075	11,803,422
20. Unit Service Factor	96.1	77.8	62.5
21. Unit Avail Factor	96.1	77.8	62.5
22. Unit Cap Factor (MDC Net)	88.7	72.7	53.5
23. Unit Cap Factor (DER Net)	88.7	72.7	53.5
24. Unit Forced Outage Rate	3.9	16.8	15.0
25. Forced Outage Hours	29.3	801.5	2,250.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
CYCLE 3 REFUELING OUTAGE: NOVEMBER 28, 1986.
27. If Currently Shutdown Estimated Startup Date: N/A

****
* SAN ONOFRE 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SAN ONOFRE 3



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* SAN ONOFRE 3 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
28	07/29/86	F	29.3	A	2	86-010	SJ	P	THE REACTOR WAS MANUALLY TRIPPED DUE TO LOSS OF MAIN FEEDWATER. INVESTIGATION INTO THE CAUSE IS CONTINUING. THE RESULTS OF THE INVESTIGATION AND APPROPRIATE CORRECTIVE ACTIONS TO BE TAKEN WILL BE ADDRESSED IN LER 86-010.
29	07/29/86	S	0.0	A	5		SJ	SHV	POWER REDUCTION TO REMOVE MAIN FEEDWATER BLOCK VALVE 3HV-4051 FROM SERVICE FOR N2 LEAK REPAIR OF HYDRAULIC DUMP VALVE.

****
* SUMMARY *
****

SAN ONOFRE 3 HAD 1 OUTAGE AND 1 POWER REDUCTION IN JULY AS DISCUSSED ABOVE.

Type

F-Forced
S-Sched

Reason

A-Equip Failure
B-Maint or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Admin
G-Oper Error
H-Other

Method

1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other

System & Component

Exhibit F & H
Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* SAN ONOFRE 3 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....CALIFORNIA

COUNTY.....SAN DIEGO

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI S OF
SAN CLEMENTE, CA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 29, 1983

DATE ELEC ENER 1ST GENER...SEPTEMBER 25, 1983

DATE COMMERCIAL OPERATE....APRIL 1, 1984

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....PACIFIC OCEAN

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....SOUTHERN CALIFORNIA EDISON

CORPORATE ADDRESS.....P.O. BOX 800
ROSEMEAD, CALIFORNIA 91770

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC COM (ENG VERSION)

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....R. HUEY

LICENSING PROJ MANAGER.....H. ROOD
DOCKET NUMBER.....50-362

LICENSE & DATE ISSUANCE....NPF-15, NOVEMBER 15, 1982

PUBLIC DOCUMENT ROOM.....UNIVERSITY OF CALIFORNIA
GENERAL LIBRARY
IRVINE, CA. 92713

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON OCTOBER 1, 1984 - JUNE 18, 1986 (REPORT NO. 50-362/86-09) YEARLY SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE. LETTER ISSUED ON AUGUST 7, 1986.

+ INSPECTION ON MAY 1 - JULY 5, 1986 (REPORT NO. 50-362/86-19) AREAS INSPECTED: ROUTINE, RESIDENT INSPECTION OF OPERATIONS PROGRAM INCLUDING THE FOLLOWING AREAS: OPERATIONAL SAFETY VERIFICATION, EVALUATION OF PLANT TRIPS AND EVENTS, MONTHLY SURVEILLANCE ACTIVITIES, MONTHLY MAINTENANCE ACTIVITIES, REFUELING ACTIVITIES, INDEPENDENT INSPECTION, LICENSEE EVENTS REPORT REVIEW, AND FOLLOWUP OF PREVIOUSLY IDENTIFIED ITEMS. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON JUNE 16-27, 1986 (REPORT NO. 50-362/86-21) REPORT CANCELLED.

+ INSPECTION ON AUGUST 4-8, 1986 (REPORT NO. 50-362/86-22) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JUNE 28 - AUGUST 15, 1986 (REPORT NO/ 50-362/86-23) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JULY 28 - AUGUST 15, 1986 (REPORT NO. 50-362/86-24) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

+ THE UNIT WAS OPERATED AT VARIOUS POWER LEVELS TO FACILITATE CLEANING OF CONDENSER WATER BOXES. IN ADDITION, A POWER REDUCTION TO 18% OCCURRED ON JULY 28, IN ORDER TO PERFORM MAIN FEEDWATER BLOCK VALVE REPAIRS.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

+ THE UNIT CONTINUED FULL POWER OPERATIONS IN JULY. ONE MANUAL REACTOR TRIP WAS INITIATED ON JULY 26, AS A RESULT OF A CONDENSATE TRIP WHICH RESULTED IN A LOSS OF MAIN FEED PUMP DUE TO LOW SUCTION PRESSURE.

LAST IE SITE INSPECTION DATE: 07/28-08/15/86+

INSPECTION REPORT NO: 50-362/86-24+

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-327 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 1488.0
3. Utility Contact: DAVID DUPREE (615) 870-6544
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1220
6. Design Electrical Rating (Net MWe): 1148
7. Maximum Dependable Capacity (Gross MWe): 1183
8. Maximum Dependable Capacity (Net MWe): 1148
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	44,568.0
13. Hours Reactor Critical	.0	.0	24,444.7
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	.0	23,871.0
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	77,060,921
18. Gross Elec Ener (MWH)	0	0	25,978,386
19. Net Elec Ener (MWH)	-3,653	-24,034	24,918,703
20. Unit Service Factor	.0	.0	53.6
21. Unit Avail Factor	.0	.0	53.6
22. Unit Cap Factor (MDC Net)	.0	.0	48.7
23. Unit Cap Factor (DER Net)	.0	.0	48.7
24. Unit Forced Outage Rate	100.0	100.0	32.4
25. Forced Outage Hours	488.0	5,831.0	11,458.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

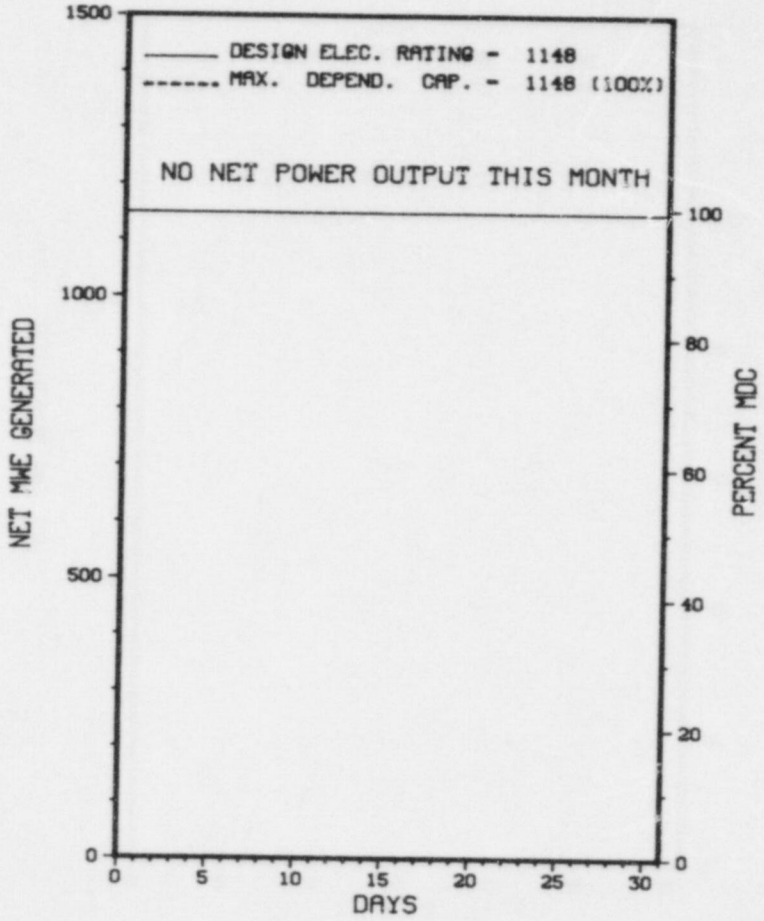
NONE

27. If Currently Shutdown Estimated Startup Date: 12/25/86

****
* SEQUOYAH 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SEQUOYAH 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

SEQUOYAH 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
7	12/20/85	F	744.0	F	4				DESIGN CONTROL, CONFIGURATION UPDATING, AND EMPLOYEE CONCERNS.

SUMMARY

SEQUOYAH 1 REMAINS IN AN EXTENDED ADMINISTRATIVE SHUTDOWN.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* SEQUOYAH 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....TENNESSEE

COUNTY.....HAMILTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...9.5 MI NE OF
CHATTANOOGA, TN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JULY 5, 1980

DATE ELEC ENER 1ST GENER...JULY 22, 1980

DATE COMMERCIAL OPERATE....JULY 1, 1981

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CHICKAMAUGA LAKE

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....TENNESSEE VALLEY AUTHORITY

CORPORATE ADDRESS.....6 NORTH 38A LOOKOUT PLACE
CHATTANOOGA, TENNESSEE 37401

CONTRACTOR
ARCHITECT/ENGINEER.....TENNESSEE VALLEY AUTHORITY

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....TENNESSEE VALLEY AUTHORITY

TURBINE SUPPLIER.....WESTINCHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....E. FORD

LICENSING PROJ MANAGER.....C. STAHLE
DOCKET NUMBER.....50-327

LICENSE & DATE ISSUANCE....DPR-77, SEPTEMBER 17, 1980

PUBLIC DOCUMENT ROOM.....CHATTANOOGA - HAMILTON BICENTENNIAL LIBRARY
1001 BROAD STREET
CHATTANOOGA, TENNESSEE 37402

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JUNE 16-20 (86-36): THIS WAS A SPECIAL, ANNOUNCED INSPECTION IN THE AREAS OF PREVIOUS ENFORCEMENT MATTERS; HEALTH PHYSICS RESTART ISSUES; FOLLOW-UP ON ALLEGATIONS; ORGANIZATION AND MANAGEMENT CONTROLS; EXTERNAL EXPOSURE CONTROL; INTERNAL EXPOSURE CONTROL; CONTROL OF RADIOACTIVE MATERIAL; FACILITIES AND EQUIPMENT; LICENSEE'S PROGRAM FOR MAINTAINING OCCUPATIONAL RADIATION EXPOSURES AS LOW AS REASONABLY ACHIEVABLE (ALARA); SOLID WASTES; TRANSPORTATION; FOLLOW-UP ON PREVIOUS INSPECTOR IDENTIFIED ITEMS; AND IE INFORMATION NOTICES. ONE VIOLATION WAS IDENTIFIED FOR FAILURE TO PERFORM RADIATION SURVEYS ON THE TOP SURFACES OF EXCLUSIVE USE CLOSED TRANSPORT VEHICLES.

ENFORCEMENT SUMMARY

CONTRARY TO TS 6.8.1.C, A WRITTEN PROCEDURE WAS NOT ESTABLISHED FOR SURVEILLANCE REQUIREMENT OF TS 4.3.1.1.1, TABLE 4.3-1, ITEM 19, NOTE (4), WHICH REQUIRES A MANUAL ENGINEERED SAFETY FEATURE (ESF) FUNCTIONAL INPUT CHECK OF THE REACTOR TRIP SYSTEM EVERY 18 MONTHS. THIS INVOLVED ONE MISSED RELAY ON EACH OF TWO MULTICONTACT SWITCHES. AS SUCH, A FUNCTIONAL CHECK OF THESE RELAYS WAS NOT CONDUCTED. CONTRARY TO TS 4.6.1.1.A, IN UNIT 1, 20 VENT, DRAIN AND TEST CONTAINMENT ISOLATION VALVES WERE NOT VERIFIED TO BE IN THE PROPER POSITION WHEN SI-14, VERIFICATION OF CONTAINMENT INTEGRITY - UNIT 1, WAS PERFORMED ON MAY 20, 1985 PRIOR TO HEAT UP TO MODE 4 AND WHILE IN MODE 4 ON MAY 17, 1985; JUNE 3, 6, 19 AND 27, 1985; AND JULY 25, 1985. THIS INVOLVED CONTAINMENT ISOLATION VALVES 62-707, 70-735, 70-737, 70-678B, 70-691B, 70-702B, C, E AND F, 70-703, 70-760, 70-762, 70-763, 72-546, 72-554, 70-543, 72-545, 78-226A, 78-228A AND 87-523.

ENFORCEMENT SUMMARY

(8602 4)

CONTRARY TO TECHNICAL SPECIFICATION 6.5.1.2, THE SEQUOYAH PROCEDURE IMPLEMENTING THE ABOVE REQUIREMENTS, SQA21, PROVIDED FOR A PORC COMPOSITION OF THE CHAIRMAN AND EIGHT MEMBERS. THE TWO "EXTRA MEMBERS", BEYOND THE TS REQUIRED COMPOSITION, COULD LEGALLY FUNCTION AS ALTERNATES, BUT WERE RECOGNIZED BY THE LICENSEE AS MEMBERS. THE TWO "EXTRA MEMBERS" RESULTED IN THE QUORUM REQUIREMENTS BEING VIOLATED WITH RESPECT TO PORC MEMBERS PRESENT DURING MEETINGS 3351, 3364, 3367, 3378, 3381, 3389 IN MAY, 1985). THE PERSONS SERVING AS THE "EXTRA MEMBERS" DO MEET ANSI N18.1 QUALIFICATIONS.
(8602 5)

CONTRARY TO 10 CFR 71.5(A), 49 CFR 173.475(I) AND 49 CFR 173.441(B)(2), ON MAY 2 AND JUNE 5, 1986, THE LICENSEE FAILED TO COMPLY WITH DOT REQUIREMENTS FOR RADIOACTIVE MATERIAL SHIPMENTS SNP-428 AND SNP-430, TRANSFERRED TO THE CHEM-NUCLEAR SYSTEMS, INC. LOW LEVEL WASTE DISPOSAL SITE NEAR BARNWELL, S.C. IN THAT, PRIOR TO THE SHIPMENT, NO RADIATION LEVEL MEASUREMENTS WERE PERFORMED ON THE TOP SURFACES OF THE EXCLUSIVE USE CLOSED TRANSPORT VEHICLES.
(8603 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

ENVIRONMENTAL QUALIFICATION OF EQUIPMENT.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

MODE 5.

LAST IE SITE INSPECTION DATE: JUNE 16-20, 1986 +

INSPECTION REPORT NO: 50-327/86-36 +

REPORTS FROM LICENSEE

SEQUOYAH 1

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-012	04/15/86	05/14/86	FAILURE TO PROPERLY STROKE TIME CONTAINMENT ISOLATION VALVES.
86-017	04/19/86	05/19/86	FAILURE TO TEST RADIATION MONITORS DURING CORE ALTERATIONS; CAUSE - INADEQUATE PROCEDURE.
86-018	04/21/86	05/20/86	FAILURE TO VERIFY INDICATED POWER AVAILABILITY DUE TO INADEQUATE PROCEDURE.
86-022	05/15/86	06/13/86	INADVERTENT CONTAINMENT VENTILATION ISOLATION CAUSED BY RADIATION MONITOR SPIKES.
86-023	05/08/86	06/06/86	CHANNEL CALIBRATION OF FLOW INDICATOR NOT PERFORMED AS REQUIRED.
86-024	05/25/86	06/20/86	SECURITY COMPUTER MALFUNCTION CAUSES MISSED FIRE WATCH; CAUSE - LACK OF A PM PROGRAM ON THE BACKUP BATTERY FOR THE MAC-540 MEMORY.

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1. Docket: 50-328 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: DAVID DUPREE (615) 870-6544

4. Licensed Thermal Power (MWt): 3411

5. Nameplate Rating (Gross MWe): 1220

6. Design Electrical Rating (Net MWe): 1148

7. Maximum Dependable Capacity (Gross MWe): 1183

8. Maximum Dependable Capacity (Net MWe): 1148

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	36,528.0
13. Hours Reactor Critical	.0	.0	21,984.5
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	.0	.0	21,494.4
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	0	0	69,127,974
18. Gross Elec Ener (MWH)	0	0	23,536,780
19. Net Elec Ener (MWH)	-5,769	-34,539	22,597,419
20. Unit Service Factor	.0	.0	58.8
21. Unit Avail Factor	.0	.0	58.8
22. Unit Cap Factor (MDC Net)	.0	.0	53.9
23. Unit Cap Factor (DER Net)	.0	.0	53.9
24. Unit Forced Outage Rate	100.0	100.0	32.2
25. Forced Outage Hours	744.0	5,087.0	10,210.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

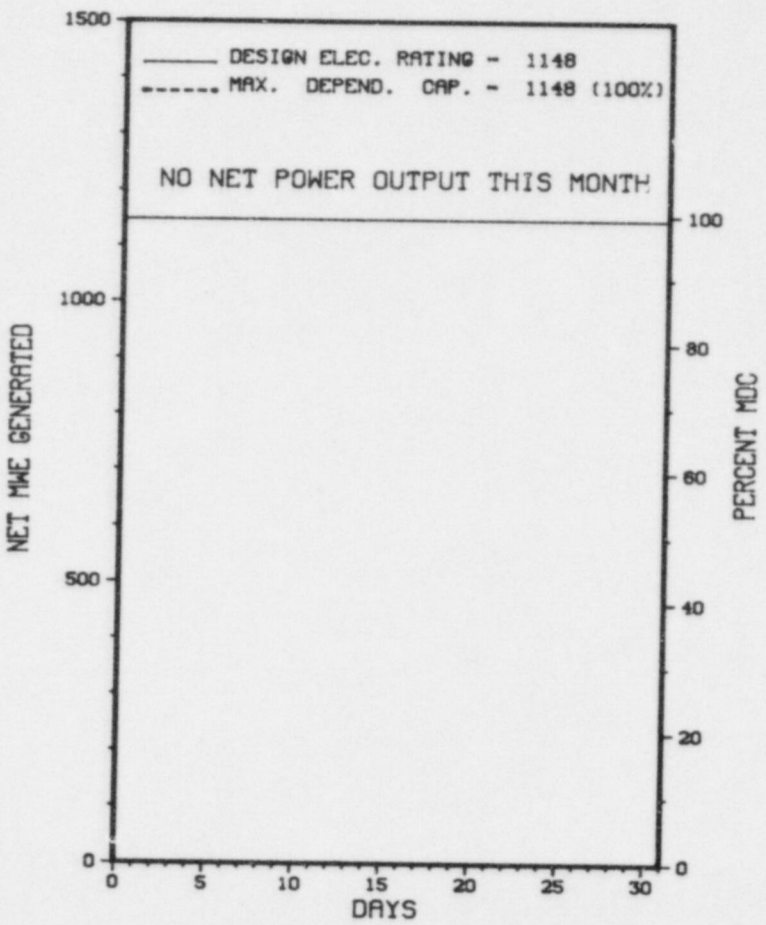
NONE

27. If Currently Shutdown Estimated Startup Date: 01/15/87

****
* SEQUOYAH 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SEQUOYAH 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

SEQUOYAH 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
7	08/21/85	F	744.0	F	4				DESIGN CONTROL, CONFIGURATION UPDATING, AND EMPLOYEE CONCERNS.

SUMMARY

SEQUOYAH 2 REMAINS IN AN EXTENDED ADMINISTRATIVE SHUTDOWN.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SEQUOYAH 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....TENNESSEE

COUNTY.....HAMILTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...9.5 MI NE OF
CHATTANOOGA, TN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...NOVEMBER 5, 1981

DATE ELEC ENER 1ST GENER...DECEMBER 23, 1981

DATE COMMERCIAL OPERATE....JUNE 1, 1982

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....CHICKAMAUGA LAKE

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....TENNESSEE VALLEY AUTHORITY

CORPORATE ADDRESS.....6 NORTH 38A LOOKOUT PLACE
CHATTANOOGA, TENNESSEE 37401

CONTRACTOR
ARCHITECT/ENGINEER.....TENNESSEE VALLEY AUTHORITY

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....TENNESSEE VALLEY AUTHORITY

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....E. FORD

LICENSING PROJ MANAGER.....C. STAHLE
DOCKET NUMBER.....50-328

LICENSE & DATE ISSUANCE....DPR-79, SEPTEMBER 15, 1981

PUBLIC DOCUMENT ROOM.....CHATTANOOGA - HAMILTON BICENTENNIAL LIBRARY
1001 BROAD STREET
CHATTANOOGA, TENNESSEE 37402

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JUNE 16-20 (86-36): THIS WAS A SPECIAL, ANNOUNCED INSPECTION IN THE AREAS OF PREVIOUS ENFORCEMENT MATTERS; HEALTH PHYSICS RESTART ISSUES; FOLLOW-UP ON ALLEGATIONS; ORGANIZATION AND MANAGEMENT CONTROLS; EXTERNAL EXPOSURE CONTROL; INTERNAL EXPOSURE CONTROL; CONTROL OF RADIOACTIVE MATERIAL; FACILITIES AND EQUIPMENT; LICENSEE'S PROGRAM FOR MAINTAINING OCCUPATIONAL RADIATION EXPOSURES AS LOW AS REASONABLY ACHIEVABLE (ALARA); SOLID WASTES; TRANSPORTATION; FOLLOW-UP ON PREVIOUS INSPECTOR IDENTIFIED ITEMS; AND IE INFORMATION NOTICES. ONE VIOLATION WAS IDENTIFIED FOR FAILURE TO PERFORM RADIATION SURVEYS ON THE TOP SURFACES OF EXCLUSIVE USE CLOSED TRANSPORT VEHICLES.

ENFORCEMENT SUMMARY

CONTRARY TO TS 6.8.1.C, A WRITTEN PROCEDURE WAS NOT ESTABLISHED FOR SURVEILLANCE REQUIREMENT OF TS 4.3.1.1.1, TABLE 4.3-1, ITEM 19, NOTE (4), WHICH REQUIRES A MANUAL ENGINEERED SAFETY FEATURE (ESF) FUNCTIONAL INPUT CHECK OF THE REACTOR TRIP SYSTEM EVERY 18 MONTHS. THIS INVOLVED ONE MISSED RELAY ON EACH OF TWO MULTICONTACT SWITCHES. AS SUCH, A FUNCTIONAL CHECK OF THESE RELAYS WAS NOT CONDUCTED.
(8602 4)

CONTRARY TO TECHNICAL SPECIFICATION 6.5.1.2, THE SEQUOYAH PROCEDURE IMPLEMENTING THE ABOVE REQUIREMENTS, SQA21, PROVIDED FOR A PORC COMPOSITION OF THE CHAIRMAN AND EIGHT MEMBERS. THE TWO "EXTRA MEMBERS", BEYOND THE TS REQUIRED COMPOSITION, COULD LEGALLY

ENFORCEMENT SUMMARY

FUNCTION AS ALTERNATES, BUT WERE RECOGNIZED BY THE LICENSEE AS MEMBERS. THE TWO "EXTRA MEMBERS" RESULTED IN THE QUORUM REQUIREMENTS BEING VIOLATED WITH RESPECT TO PORC MEMBERS PRESENT DURING MEETINGS 3351, 3364, 3367, 3378, 3381, 3389 IN MAY, 1985). THE PERSONS SERVING AS THE "EXTRA MEMBERS" DO MEET ANSI N18.1 QUALIFICATIONS.
(8602 5)

CONTRARY TO 10 CFR 71.5(A), 49 CFR 173.475(I) AND 49 CFR 173.441(B)(2), ON MAY 2 AND JUNE 5, 1986, THE LICENSEE FAILED TO COMPLY WITH DOT REQUIREMENTS FOR RADIOACTIVE MATERIAL SHIPMENTS SNP-428 AND SNP-430, TRANSFERRED TO THE CHEM-NUCLEAR SYSTEMS, INC. LOW LEVEL WASTE DISPOSAL SITE NEAR BARNWELL, S.C. IN THAT, PRIOR TO THE SHIPMENT, NO RADIATION LEVEL MEASUREMENTS WERE PERFORMED ON THE TOP SURFACES OF THE EXCLUSIVE USE CLOSED TRANSPORT VEHICLES.
(8603 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

ENVIRONMENTAL QUALIFICATION OF EQUIPMENT.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

MODE 5.

LAST IE SITE INSPECTION DATE: JUNE 16-20, 1986 +

INSPECTION REPORT NO: 50-328/86-36 +

REPORTS FROM LICENSEE

SEQUOYAH 2

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-001	04/26/86	06/24/86	FAILURE TO PERFORM A COMPLETE BORATION FLOW PATH VERIFICATION - MODE 5. ROOT CAUSE - AN INCORRECT STATEMENT IN THE SAFETY INJECTION (SI).
86-002	05/13/86	06/11/86	CONTAINMENT VENTILATION ISOLATION DUE TO SPIKE ON RADIATION MONITOR; CAUSED BY A COGNITIVE PERSONNEL ERROR.
86-003	05/30/86	06/27/86	CONTAINMENT VENTILATION ISOLATIONS CAUSED BY SPIKES ON RADIATION MONITORS.

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1. Docket: 50-335 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: N. W. GRANT (305) 552-3675
4. Licensed Thermal Power (MWt): 2700
5. Nameplate Rating (Gross MWe): 1000 X 0.89 = 890
6. Design Electrical Rating (Net MWe): 830
7. Maximum Dependable Capacity (Gross MWe): 867
8. Maximum Dependable Capacity (Net MWe): 827
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	84,239.0
13. Hours Reactor Critical	744.0	4,767.3	61,923.5
14. Rx Reserve Shtdwn Hrs	.0	.0	205.3
15. Hrs Generator On-Line	744.0	4,735.8	60,538.6
16. Unit Reserve Shtdwn Hrs	.0	.0	39.3
17. Gross Therm Ener (MWH)	2,008,970	12,704,116	153,594,351
18. Gross Elec Ener (MWH)	669,460	4,254,470	50,310,975
19. Net Elec Ener (MWH)	636,363	4,040,687	47,464,901
20. Unit Service Factor	100.0	93.1	71.9
21. Unit Avail Factor	100.0	93.1	71.9
22. Unit Cap Factor (MDC Net)	103.4	96.0	68.1
23. Unit Cap Factor (DER Net)	103.1	95.7	67.9
24. Unit Forced Outage Rate	.0	.4	4.1
25. Forced Outage Hours	.0	20.6	2,575.4

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

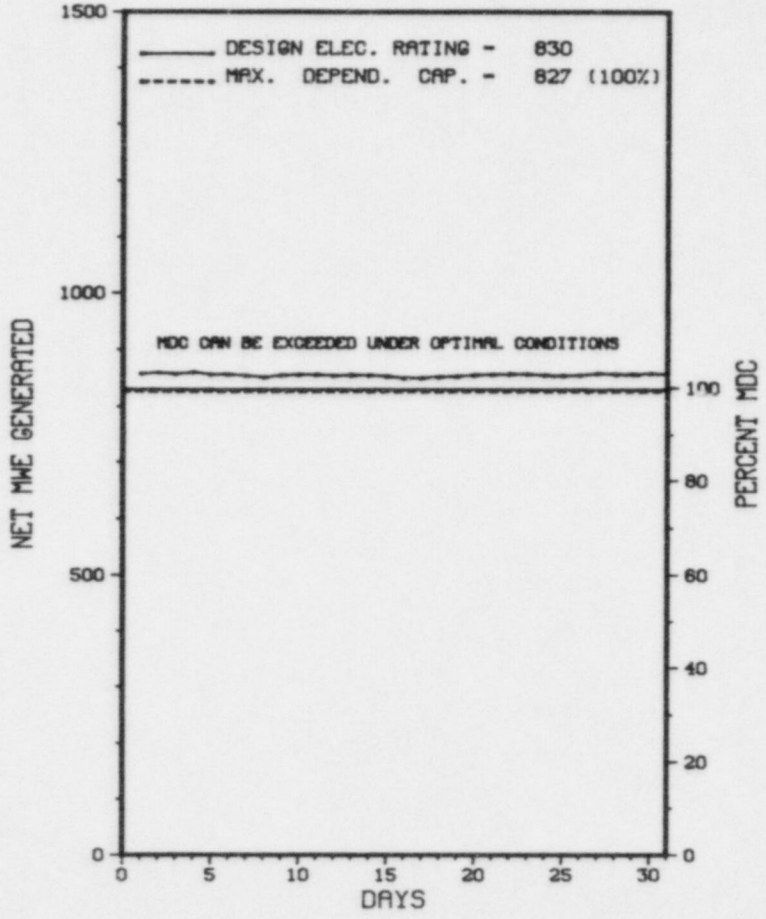
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* ST LUCIE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ST LUCIE 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* ST LUCIE 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

****
* SUMMARY *
****

ST. LUCIE 1 OPERATED ROUTINELY IN JULY WITH NO SHUTDOWNS OR POWER REDUCTIONS REPORTED.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* ST LUCIE 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....FLORIDA

COUNTY.....ST LUCIE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...12 MI SE OF
FT. PIERCE, FLA

TYPE OF REACTOR.....PWR

DATE INITIA CRITICALITY...APRIL 22, 1976

DATE ELEC ENER 1ST GENER...MAY 7, 1976

DATE COMMERCIAL OPERATE....DECEMBER 21, 1976

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....ATLANTIC OCEAN

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....FLORIDA POWER & LIGHT

CORPORATE ADDRESS.....9250 WEST FLAGLER STREET P.O. BOX 529100
MIAMI, FLORIDA 33152

CONTRACTOR
ARCHITECT/ENGINEER.....EBASCO

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....EBASCO

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....R. CRLENJAK

LICENSING PROJ MANAGER.....E. TOURIGNY
DOCKET NUMBER.....50-335

LICENSE & DATE ISSUANCE....DPR-67, MARCH 1, 1976

PUBLIC DOCUMENT ROOM.....INDIAN RIVER COMMUNITY COLLEGE LIBRARY
3209 VIRGINIA AVENUE
FT. PIERCE, FLORIDA 33450

INSPECTION STATUS

INSPECTION SUMMARY

+ THERE WERE NO INSPECTIONS CONDUCTED AT ST. LUCIE UNIT 1 FOR THE MONTH OF JULY.

ENFORCEMENT SUMMARY

CONTRARY TO 10 CFR 71.5 AND 49 CFR 173.441(A), ON DECEMBER 13, 1985, THE LICENSEE SHIPPED TWO BOXES OF RADIOACTIVE MATERIALS, BOX NO. 85-105 AND BOX NO. 85-112, TO A LAND DISPOSAL FACILITY AT BARNWELL, S.C. WITH DOSE RATES OF 250 AND 400 MILLIREM PER HOUR, RESPECTIVELY, ON THE EXTERNAL SURFACES OF THE PACKAGES. THE EXCEPTIONS OF 49 CFR 173.441(B) DID NOT APPLY BECAUSE THE SHIPMENT WAS NOT MADE IN AN ENCLOSED VEHICLE.
(8606 3)

CONTRARY TO 10 CFR 71.5 AND 49 CFR 173.425(B)(1), THE LICENSEE FAILED TO PACKAGE A SHIPMENT OF LSA RADIOACTIVE MATERIAL IN A DOT SPECIFICATION 7A TYPE A PACKAGE OR A STRONG, TIGHT PACKAGE IN THAT ON OCTOBER 22, 1985, SHIPMENT NO. 85-49 ARRIVED AT THE BURIAL FACILITY WITH A HOLE IN ONE B-25 METAL BOX.

CONTRARY TO 10 CFR 20.103(C)(2), ON APRIL 18, 1986, THE NRC INSPECTOR NOTED THE LICENSEE WAS MAKING ALLOWANCE IN ITS MAXIMUM PERMISSIBLE CONCENTRATION HOUR ASSIGNMENT RECORDS FOR USE OF RESPIRATORY PROTECTIVE EQUIPMENT IN ESTIMATING EXPOSURES OF INDIVIDUALS TO RADIOACTIVE MATERIAL IN AIR FOR PURPOSES OF DETERMINING COMPLIANCE WITH 10 CFR 20.103. HOWEVER, THE LICENSEE DID

****
* ST LUCIE 1 *
****

ENFORCEMENT SUMMARY

NOT HAVE WRITTEN PROCEDURES REGARDING MAINTENANCE OF RECORDS SHOWING THAT AN INDIVIDUAL HAD BEEN ISSUED A RESPIRATOR. CONTRARY TO THE REQUIREMENTS OF 10 CFR 73.71(C) CERTAIN SECURITY EVENTS WERE NOT REPORTED AS REQUIRED.

CONTRARY TO THE REQUIREMENTS OF THE SECURITY PLAN CERTAIN VITAL AREA BARRIERS DID NOT MEET THE CRITERIA OF 10 CFR 73.2(F). CONTRARY TO COMMITTMENTS IN THE APPROVED SECURITY PLAN PROPER ISSUANCE AND CONTROL OF CARD KEYS WAS NOT MAINTAINED IN ALL CASES. (8600 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATIONS.

LAST IE SITE INSPECTION DATE: MAY 13 - JUNE 16, 1986

INSPECTION REPORT NO: 50-335/86-16

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

1. Docket: 50-389 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: N. W. GRANT (305) 552-3675

4. Licensed Thermal Power (MWt): 2700

5. Nameplate Rating (Gross MWe): 0850

6. Design Electrical Rating (Net MWe): 830

7. Maximum Dependable Capacity (Gross MWe): 882

8. Maximum Dependable Capacity (Net MWe): 837

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	26,136.0
13. Hours Reactor Critical	744.0	3,661.3	21,710.2
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	744.0	3,604.1	21,176.3
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,008,268	9,584,356	54,230,890
18. Gross Elec Ener (MWH)	674,790	3,206,580	18,123,260
19. Net Elec Ener (MWH)	641,224	3,031,730	17,102,773
20. Unit Service Factor	100.0	70.8	81.0
21. Unit Avail Factor	100.0	70.8	81.0
22. Unit Cap Factor (MDC Net)	103.0	71.2	78.2
23. Unit Cap Factor (DER Net)	103.8	71.8	78.8
24. Unit Forced Outage Rate	.0	1.5	9.3
25. Forced Outage Hours	.0	53.2	?,179.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

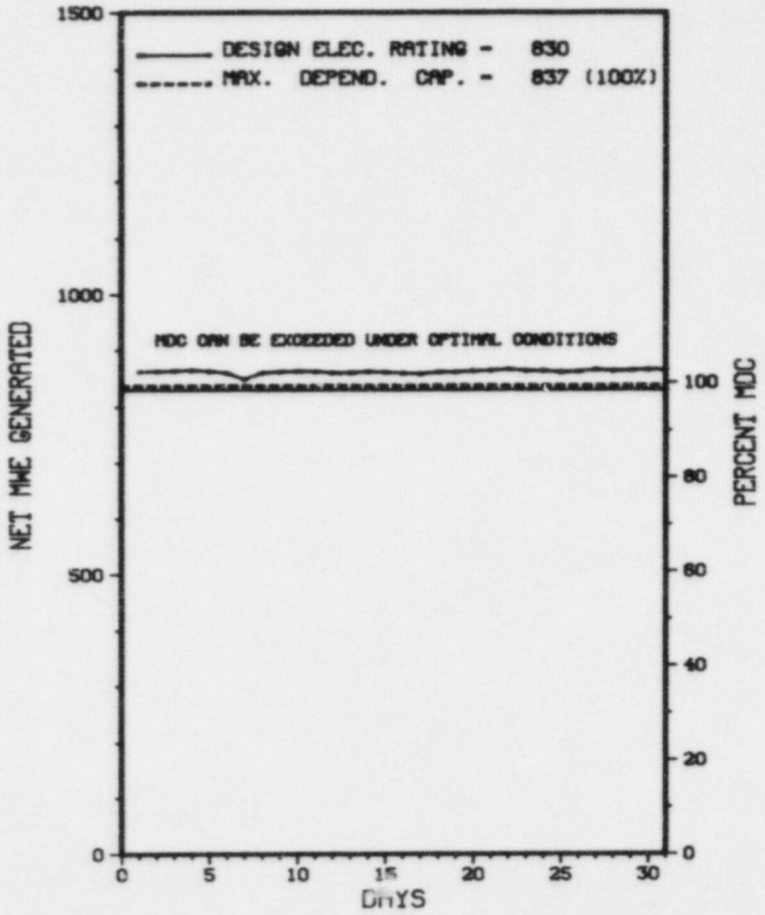
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* ST LUCIE 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ST LUCIE 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

ST LUCIE 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

ST. LUCIE 2 OPERATED ROUTINELY IN JULY WITH NO SHUTDOWNS OR POWER REDUCTIONS REPORTED.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* ST LUCIE 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....FLORIDA

COUNTY.....ST LUCIE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...12 MI SE OF
FT. PIERCE, FLA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JUNE 2, 1983

DATE ELEC ENER 1ST GENER...JUNE 13, 1983

DATE COMMERCIAL OPERATE....AUGUST 8, 1983

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....ATLANTIC OCEAN

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....FLORIDA POWER & LIGHT

CORPORATE ADDRESS.....9250 WEST FLAGLER ST., P.O. BOX 529100
MIAMI, FLORIDA 33152

CONTRACTOR
ARCHITECT/ENGINEER.....EBASCO

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....EBASCO

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....R. CRLENJAK

LICENSING PROJ MANAGER.....E. TOURIGNY
DOCKET NUMBER.....50-389

LICENSE & DATE ISSUANCE....NPF-16, JUNE 10, 1983

PUBLIC DOCUMENT ROOM.....INDIAN RIVER COMMUNITY COLLEGE LIBRARY
3209 VIRGINIA AVENUE
FT. PIERCE, FLORIDA 33450

INSPECTION STATUS

INSPECTION SUMMARY

+ THERE WERE NO INSPECTIONS CONDUCTED AT ST. LUCIE UNIT 2 FOR THE MONTH OF JULY.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

OTHER ITEMS

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATIONS.

LAST IE SITE INSPECTION DATE: MAY 13 - JUNE 16, 1986

INSPECTION REPORT NO: 50-389/86-15

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-007	04/17/86	05/19/86	PRESSURIZER CODE SAFETIES INOPERABLE DUE TO LIFT SETPOINT DRIFT.
86-009	05/22/86	06/21/86	LLRT ON CONTAINMENT PENETRATION 28A EXCEEDED TECHNICAL SPECIFICATION; CAUSE - A DEGRADED DISC.
86-010	06/04/86	07/07/86	REACTOR TRIP BY LOW STEAM GENERATOR WATER LEVEL DUE TO PERSONNEL ERROR.

1. Docket: 50-395 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: G. A. LOIGNON (803) 345-5209
4. Licensed Thermal Power (MWt): 2775
5. Nameplate Rating (Gross MWe): 0900
6. Design Electrical Rating (Net MWe): 900
7. Maximum Dependable Capacity (Gross MWe): 900
8. Maximum Dependable Capacity (Net MWe): 885
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	22,631.0
13. Hours Reactor Critical	730.9	4,858.8	16,852.1
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	710.8	4,797.9	16,438.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,898,812	13,018,234	42,855,800
18. Gross Elec Ener (MWH)	620,750	4,338,560	14,269,183
19. Net Elec Ener (MWH)	593,664	4,154,920	13,581,967
20. Unit Service Factor	95.5	94.3	72.6
21. Unit Avail Factor	95.5	94.3	72.6
22. Unit Cap Factor (MDC Net)	90.2	92.3	67.8
23. Unit Cap Factor (DER Net)	88.7	90.8	66.7
24. Unit Forced Outage Rate	4.5	5.7	7.6
25. Forced Outage Hours	33.2	289.1	1,349.8

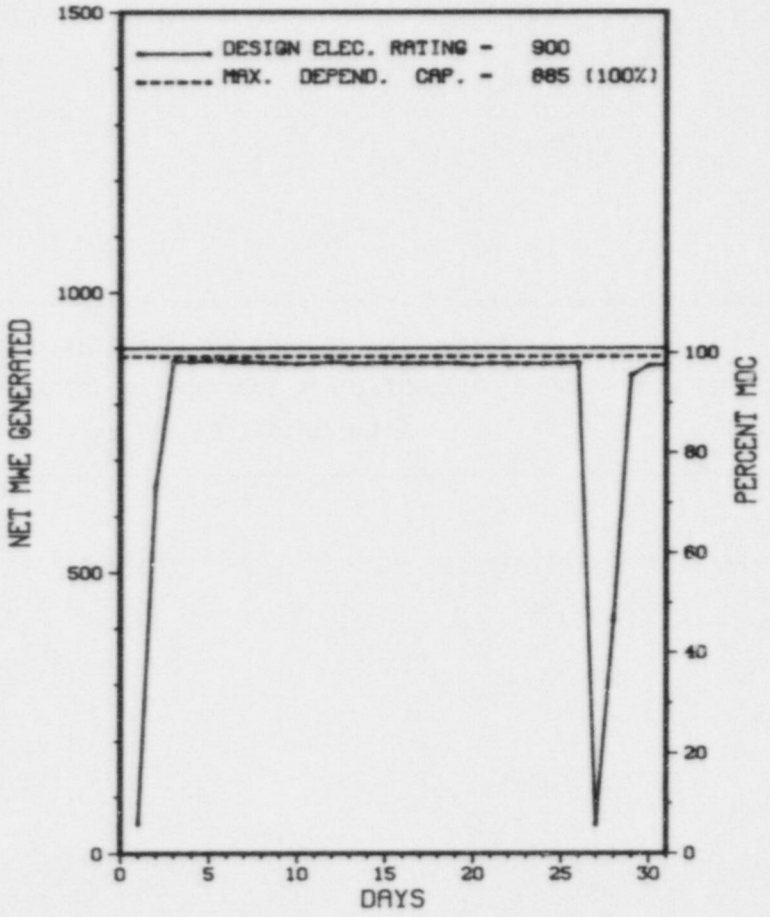
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* SUMMER 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SUMMER 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* SUMMER 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
4	06/29/86	F	14.3	A	4				CONTINUATION OF OUTAGE FROM PREVIOUS MONTH.
5	07/27/86	F	18.9	A	3				LOW EHC PRESSURE DURING CONTROL VALVE TESTING.

****
* SUMMARY *
****

SUMMER 1 HAD 2 OUTAGES IN JULY AS DISCUSSED ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SUMMER 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....SOUTH CAROLINA

COUNTY.....FAIRFIELD

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...26 MI NW OF
COLUMBIA, SC

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...OCTOBER 22, 1982

DATE ELEC ENER 1ST GENER...NOVEMBER 16, 1982

DATE COMMERCIAL OPERATE....JANUARY 1, 1984

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MONTICELLO RESERVOIR

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....SOUTH CAROLINA ELECTRIC & GAS CO.

CORPORATE ADDRESS.....P.O. BOX 764
COLUMBIA, SOUTH CAROLINA 29202

CONTRACTOR
ARCHITECT/ENGINEER.....GILBERT ASSOCIATES

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....DANIEL INTERNATIONAL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....R. PREVATTE

LICENSING PROJ MANAGER.....J. HOPKINS
DOCKET NUMBER.....50-395

LICENSE & DATE ISSUANCE....NPF-12, NOVEMBER 12, 1982

PUBLIC DOCUMENT ROOM.....FAIRFIELD COUNTY LIBRARY
GARDEN & WASHINGTON STREETS
WINNSBORO, SOUTH CAROLINA 29180

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION APRIL 1- MAY 31 (86-09): THIS ROUTINE, ANNOUNCED INSPECTION WAS CONDUCTED IN THE AREAS OF LICENSEE ACTION ON PREVIOUS ENFORCEMENT MATTERS, ON SITE FOLLOWUP OF EVENTS AND SUBSEQUENT WRITTEN REPORTS, MONTHLY SURVEILLANCE OBSERVATIONS, ENGINEERED SAFETY FEATURES SYSTEM WALKDOWN, MONTHLY MAINTENANCE OBSERVATION, AND OPERATIONAL SAFETY VERIFICATION. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

OTHER ITEMS

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: APRIL 1 - MAY 31, 1986 +

INSPECTION REPORT NO: 50-395/86-09 +

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-007	04/22/86	05/22/86	REACTOR BUILDING SPRAY PUMP 'B' INADVERTENTLY STARTED.
86-009	05/31/86	06/27/86	REACTOR TRIPPED DUE TO PERSONNEL ERROR DURING TESTING OF THE POWER RANGE NEUTRON DETECTOR CHANNELS.

1. Docket: 50-280 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: VIVIAN H. JONES (804) 357-3184
4. Licensed Thermal Power (MWt): 2441
5. Nameplate Rating (Gross MWe): 942 X 0.9 = 848
6. Design Electrical Rating (Net MWe): 788
7. Maximum Dependable Capacity (Gross MWe): 820
8. Maximum Dependable Capacity (Net MWe): 781
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	119,279.0
13. Hours Reactor Critical	441.0	3,171.2	75,499.3
14. Rx Reserve Shtdwn Hrs	.0	.0	3,774.5
15. Hrs Generator On-Line	327.8	3,023.9	73,862.0
16. Unit Reserve Shtdwn Hrs	.0	.0	3,736.2
17. Gross Therm Ener (MWH)	648,190	6,973,390	170,330,316
18. Gross Elec Ener (MWH)	205,250	2,308,945	55,080,778
19. Net Elec Ener (MWH)	192,484	2,191,310	52,221,439
20. Unit Service Factor	44.1	59.4	61.9
21. Unit Avail Factor	44.1	59.4	65.1
22. Unit Cap Factor (MDC Net)	33.1	55.2	56.1
23. Unit Cap Factor (DER Net)	32.8	54.7	55.6
24. Unit Forced Outage Rate	.0	1.8	18.5
25. Forced Outage Hours	.0	56.0	13,037.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

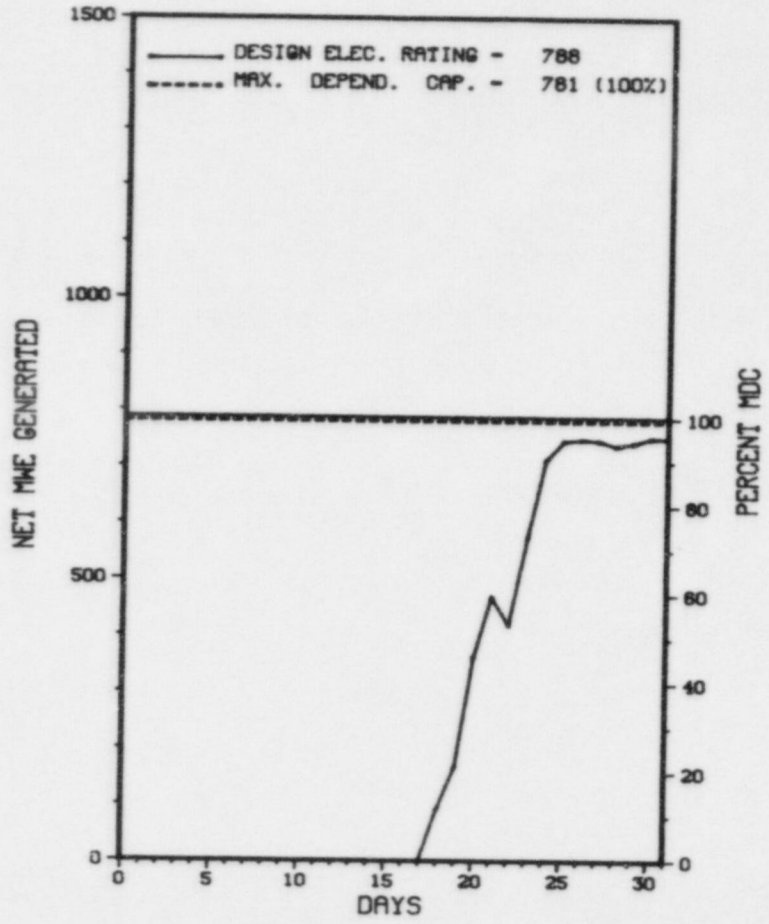
NONE

27. If Currently Shutdown Estimated Startup Date: 07/04/86

****
* SURRY 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SURRY 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

SURRY 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-05	05/10/86	S	416.2	C	1				PLANT SHUTDOWN FOR REFUELING.

SUMMARY

SURRY 1 COMPLETED A REFUELING OUTAGE ON JULY 18.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* SURRY 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....VIRGINIA

COUNTY.....SURRY

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...17 MI NW OF
NEWPORT NEWS, VA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JULY 1, 1972

DATE ELEC ENER 1ST GENER...JULY 4, 1972

DATE COMMERCIAL OPERATE....DECEMBER 22, 1972

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....JAMES RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....VIRGINIA POWER

CORPORATE ADDRESS.....P.O. BOX 26666
RICHMOND, VIRGINIA 23261

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....D. BURKE

LICENSING PROJ MANAGER.....C. PATEL
DOCKET NUMBER.....50-280

LICENSE & DATE ISSUANCE....DPR-32, MAY 25, 1972

PUBLIC DOCUMENT ROOM.....SWEM LIBRARY
COLLEGE OF WILLIAM AND MARY
WILLIAMSBURG, VIRGINIA 23185

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JUNE 2-6 (86-13): THIS WAS A ROUTINE, UNANNOUNCED INSPECTION IN THE AREAS OF INSERVICE INSPECTION, REPAIR ACTIVITIES AND REVIEW OF PREVIOUS OPEN ITEMS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REFUELING OUTAGE

LAST IE SITE INSPECTION DATE: JUNE 2-6, 1986 +

INSPECTION REPORT NO: 50-280/86-13 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-020	06/09/86		AN ENVIRONMENTAL QUALIFICATION INSPECTION MADE OF LIMITORQUE MOTOR VALVE OPERATOR WIRING AS A RESULT OF IEN 86-03 (LACK OF IDENTIFICATION WIRING NOT ENVIRONMENTALLY QUALIFIED).

1. Docket: 50-281 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: VIVIAN H. JONES (804) 357-3184
4. Licensed Thermal Power (MWt): 2441
5. Nameplate Rating (Gross MWe): 942 X 0.9 = 848
6. Design Electrical Rating (Net MWe): 788
7. Maximum Dependable Capacity (Gross MWe): 811
8. Maximum Dependable Capacity (Net MWe): 781
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	116,159.0
13. Hours Reactor Critical	592.1	4,411.1	76,353.5
14. Rx Reserve Shtdwn Hrs	.0	.0	23.8
15. Hrs Generator On-Line	584.1	4,374.8	75,140.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,331,725	10,495,120	175,802,147
18. Gross Elec Ener (MWH)	427,750	3,447,660	57,030,964
19. Net Elec Ener (MWH)	405,202	3,274,891	54,063,778
20. Unit Service Factor	78.5	86.0	64.7
21. Unit Avail Factor	78.5	86.0	64.7
22. Unit Cap Factor (MDC Net)	69.7	82.5	59.6
23. Unit Cap Factor (DER Net)	69.1	81.7	59.1
24. Unit Forced Outage Rate	21.5	10.4	12.9
25. Forced Outage Hours	159.9	506.1	8,437.9

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

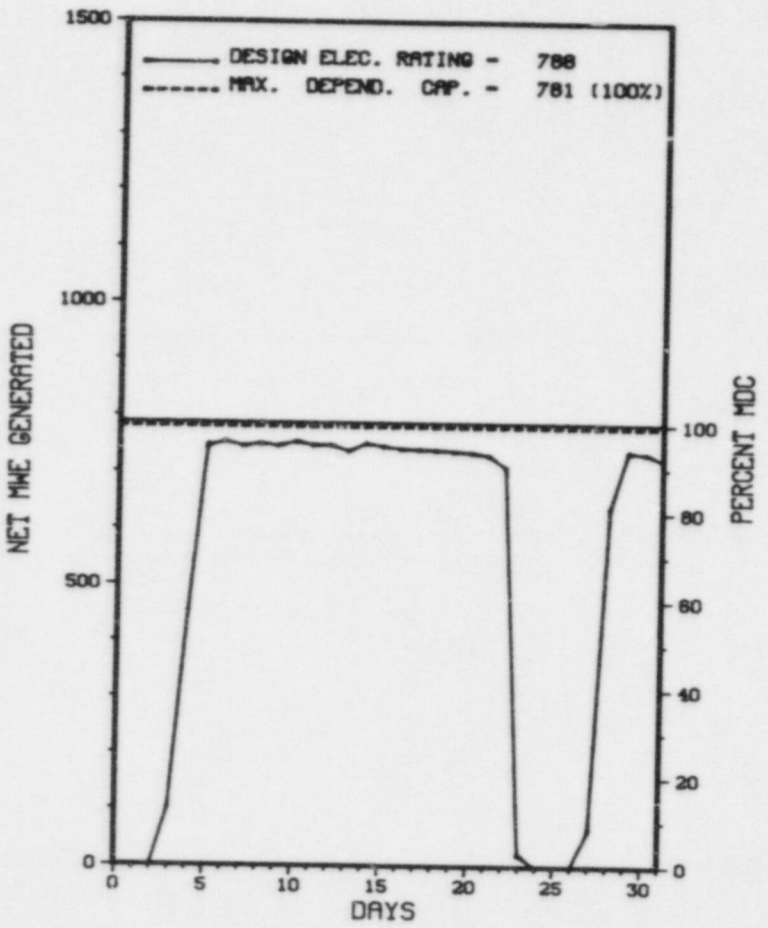
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* SURRY 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SURRY 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

SURRY 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-04	06/17/86	F	52.8	D	4	86-006			UNIT SHUTDOWN FOR REPAIR ON SW EXPANSION JOINT IN RS-E-1A AND PLUGGING OF S/G TUBE LEAK. CONTINUATION OF 6/17/86 OUTAGE.
86-05	07/23/86	F	107.1	D	1	86-007			UNIT WAS SHUTDOWN TO REPAIR TUBE LEAK IN RS-E-1B.

SUMMARY

SURRY 2 EXPERIENCED 2 SHUTDOWNS IN JULY AS DISCUSSED ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SURRY 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....VIRGINIA

COUNTY.....SURRY

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...17 MI NW OF
NEWPORT NEWS, VA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MARCH 7, 1973

DATE ELEC ENER 1ST GENER...MARCH 10, 1973

DATE COMMERCIAL OPERATE....MAY 1, 1973

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....JAMES RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....VIRGINIA POWER

CORPORATE ADDRESS.....P.O. BOX 26666
RICHMOND, VIRGINIA 23261

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....D. BURKE

LICENSING PROJ MANAGER.....C. PATEL
DOCKET NUMBER.....50-281

LICENSE & DATE ISSUANCE....DPR-37, JANUARY 29, 1973

PUBLIC DOCUMENT ROOM.....SWEM LIBRARY
COLLEGE OF WILLIAM AND MARY
WILLIAMSBURG, VIRGINIA 23185

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION JUNE 2-6 (86-13): THIS WAS A ROUTINE, UNANNOUNCED INSPECTION IN THE AREAS OF INSERVICE INSPECTION, REPAIR ACTIVITIES AND REVIEW OF PREVIOUS OPEN ITEMS. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

OTHER ITEMS

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 2-6, 1986 +

INSPECTION REPORT NO: 50-281/86-13 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-007	06/17/86	07/14/86	MANUAL REACTOR TRIP DUE TO HIGH STEAM GENERATOR LEVEL; CAUSED BY FAILURE OF VALVE FCV-2478 (ELLS FCV) TO CLOSE.

1. Docket: 50-387 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: J. A. HIRT (717) 542-3917

4. Licensed Thermal Power (MWt): 3293

5. Nameplate Rating (Gross MWe): 1280 X 0.9 = 1152

6. Design Electrical Rating (Net MWe): 1065

7. Maximum Dependable Capacity (Gross MWe): 1068

8. Maximum Dependable Capacity (Net MWe): 1032

9. If Changes Occur Above Since Last Report, Give Reasons:

---

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	27,600.0
13. Hours Reactor Critical	744.0	2,931.9	18,925.0
14. Rx Reserve Shtdwn Hrs	.0	102.1	773.2
15. Hrs Generator On-Line	744.0	2,791.8	18,412.4
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,407,031	8,674,169	56,254,442
18. Gross Elec Ener (MWH)	777,958	2,822,192	18,321,184
19. Net Elec Ener (MWH)	750,278	2,685,596	17,574,822
20. Unit Service Factor	100.0	54.9	66.7
21. Unit Avail Factor	100.0	54.9	66.7
22. Unit Cap Factor (MDC Net)	97.7	51.2	61.7
23. Unit Cap Factor (DER Net)	94.7	49.6	59.8
24. Unit Forced Outage Rate	.0	19.6	12.6
25. Forced Outage Hours	.0	681.5	2,661.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

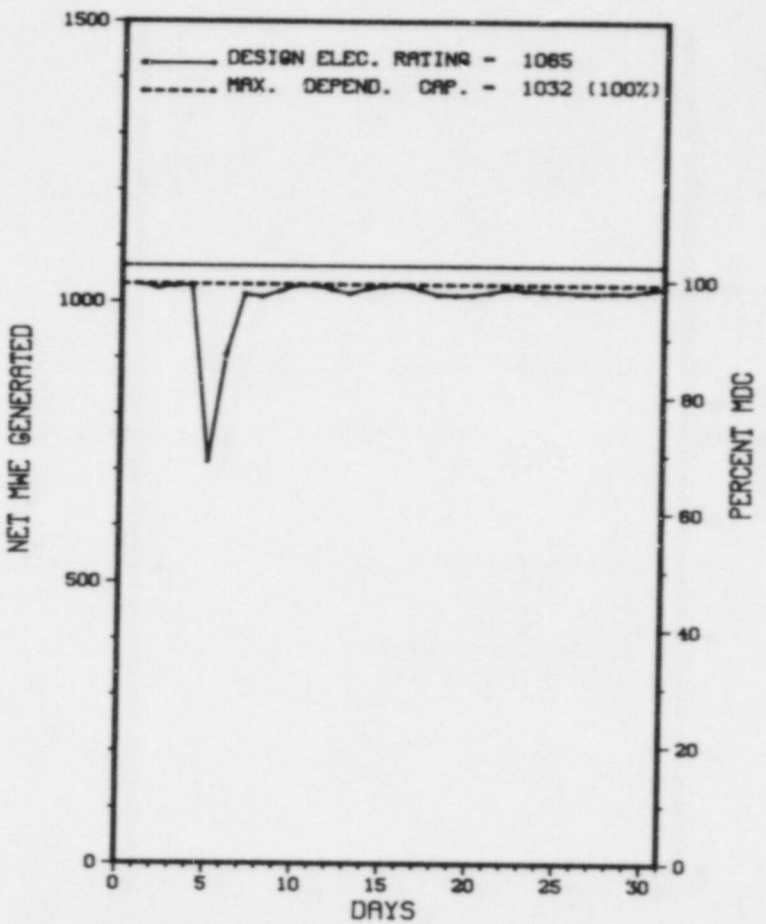
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* SUSQUEHANNA 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SUSQUEHANNA 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* SUSQUEHANNA 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
8	07/05/86	S	0.0	F	5		ZZ	ZZZZZZ	POWER REDUCTION FOR CONTROL ROD SEQUENCE EXCHANGE. MSIV STROKE TIME TESTING ALSO COMPLETED.

****
* SUMMARY *
****

SUSQUEHANNA 1 OPERATED ROUTINELY IN JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* SUSQUEHANNA 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....PENNSYLVANIA

COUNTY.....LUZERNE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...7 MI NE OF
BERWICK, PA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...SEPTEMBER 10, 1982

DATE ELEC ENER 1ST GENER.. NOVEMBER 16, 1982

DATE COMMERCIAL OPERATE....JUNE 8, 1983

CONDENSER COOLING METHOD...CC,HNDCT

CONDENSER COOLING WATER....SUSQUEHANNA RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PENNSYLVANIA POWER & LIGHT

CORPORATE ADDRESS.....2 NORTH NINTH STREET
ALLENTOWN, PENNSYLVANIA 18101

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....R. JACOBS

LICENSING PROJ MANAGER.....M. THADANI
DOCKET NUMBER.....50-387

LICENSE & DATE ISSUANCE....NPF-14, NOVEMBER 12, 1982

PUBLIC DOCUMENT ROOM.....OSTERHOUT FREE LIBRARY
71 SOUTH FRANKLIN STREET
WILKES-BARRE, PENNSYLVANIA 18701

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-388 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: J. A. HIRT (717) 542-3917
4. Licensed Thermal Power (MWt): 3293
5. Nameplate Rating (Gross MWe): 1152
6. Design Electrical Rating (Net MWe): 1065
7. Maximum Dependable Capacity (Gross MWe): 1068
8. Maximum Dependable Capacity (Net MWe): 1032
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

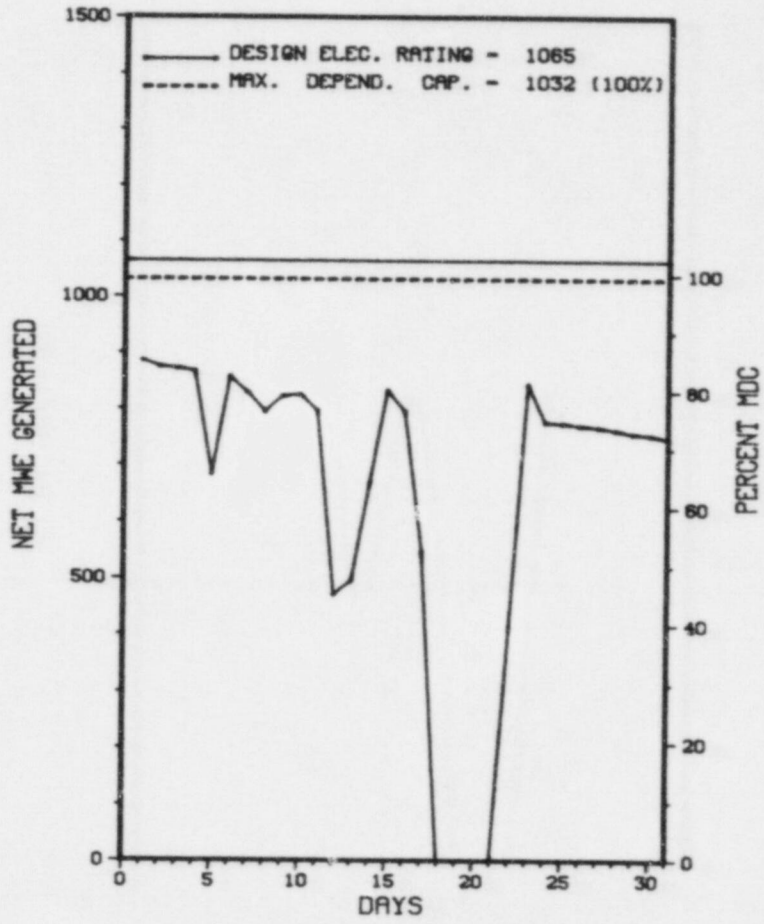
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	12,839.0
13. Hours Reactor Critical	659.1	4,273.8	11,395.0
14. Rx Reserve Shtdwn Hrs	.0	133.6	693.9
15. Hrs Generator On-Line	637.6	4,201.6	11,196.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,608,845	12,829,982	34,860,780
18. Gross Elec Ener (MWH)	509,494	4,205,628	11,418,428
19. Net Elec Ener (MWH)	485,229	4,040,611	10,994,921
20. Unit Service Factor	85.7	82.6	87.2
21. Unit Avail Factor	85.7	82.6	87.2
22. Unit Cap Factor (MDC Net)	63.2	77.0	83.0
23. Unit Cap Factor (DER Net)	61.2	74.6	80.4
24. Unit Forced Outage Rate	14.3	17.4	12.8
25. Forced Outage Hours	106.4	885.4	1,642.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
REFUELING OUTAGE: AUGUST 9, 1986; 84 DAYS.
27. If Currently Shutdown Estimated Startup Date: N/A

SUSQUEHANNA 2

AVERAGE DAILY POWER LEVEL (MWe) PLOT

SUSQUEHANNA 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

SUSQUEHANNA 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
6	07/12/86	F	0.0	A	5		CE	VALVOP	POWER REDUCTION TO FACILITATE MAINTENANCE ON RCIC INJECTION VALVE DUE TO ALARA CONCERNS.
7	07/17/86	F	106.4	A	2	86-010	CF	VALVEX	CONTROLLED SHUTDOWN DUE TO REACTOR COOLANT UNIDENTIFIED LEAKAGE EXCEEDING 5 GPM. THE PACKING OF RHR LOOP "B" TESTABLE CHECK VALVE HAD FAILED AND WAS REPLACED.

SUMMARY

SUSQUEHANNA INCURRED 1 SHUTDOWN AND 1 POWER REDUCTION IN JULY AS DISCUSSED ABOVE.

Type
F-Forced
S-Sched

Reason
A-Equip Failure
B-Maint or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Admin
G-Oper Error
H-Other

Method
1-Manual
2-Manual Scram
3-Auto Scram
4-Continued
5-Reduced Load
9-Other

System & Component
Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* SUSQUEHANNA 2 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....PENNSYLVANIA

COUNTY.....LUZERNE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...7 MI NE OF
BERWICK, PA

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...MAY 8, 1984

DATE ELEC ENER 1ST GENER...JULY 3, 1984

DATE COMMERCIAL OPERATE....FEBRUARY 12, 1985

CONDENSER COOLING METHOD...CC,HNDCT

CONDENSER COOLING WATER....SUSQUEHANNA RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PENNSYLVANIA POWER & LIGHT

CORPORATE ADDRESS.....2 NORTH NINTH STREET
ALLENTOWN, PENNSYLVANIA 18101

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....L. PLISCO

LICENSING PROJ MANAGER.....M. THADANI
DOCKET NUMBER.....50-388

LICENSE & DATE ISSUANCE....NPF-22, JUNE 27, 1984

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71 SOUTH FRANKLIN STREET
WILKES-BARRE, PENNSYLVANIA 18701

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-289 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: C. W. SMYTH (717) 948-8551
4. Licensed Thermal Power (MWt): 2535
5. Nameplate Rating (Gross MWe): 968 X 0.9 = 871
6. Design Electrical Rating (Net MWe): 819
7. Maximum Dependable Capacity (Gross MWe): 840
8. Maximum Dependable Capacity (Net MWe): 776
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	104,424.0
13. Hours Reactor Critical	744.0	4,061.6	37,878.2
14. Rx Reserve Shtdwn Hrs	.0	990.8	1,875.0
15. Hrs Generator On-Line	744.0	4,005.6	37,039.5
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,879,956	10,012,439	89,362,835
18. Gross Elec Ener (MWH)	608,815	3,302,856	29,695,009
19. Net Elec Ener (MWH)	574,180	3,098,185	27,749,898
20. Unit Service Factor	100.0	78.7	35.5
21. Unit Avail Factor	100.0	78.7	35.5
22. Unit Cap Factor (MDC Net)	99.5	78.5	34.0*
23. Unit Cap Factor (DER Net)	94.2	74.4	32.4
24. Unit Forced Outage Rate	.0	4.1	61.4
25. Forced Outage Hours	.0	171.3	58,939.8

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

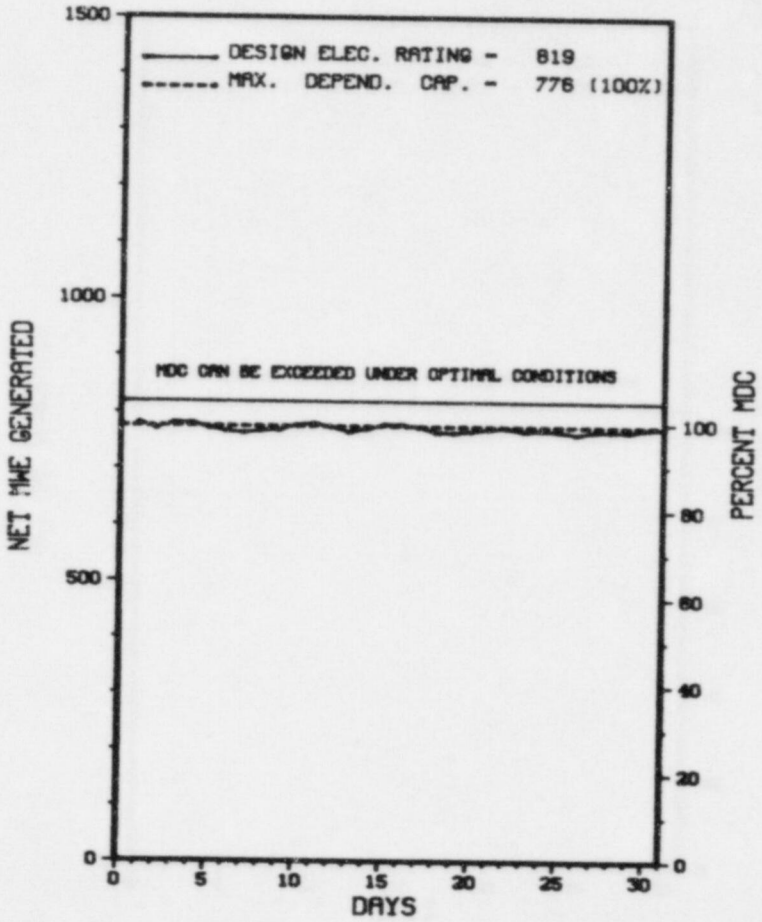
REFUELING OUTAGE: NOVEMBER 1, 1986 - 4 TO 5 MONTHS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* THREE MILE ISLAND 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

THREE MILE ISLAND 1



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

THREE MILE ISLAND 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

THREE MILE ISLAND 1 OPERATED ROUTINELY IN JULY WITH NO SHUTDOWNS A POWER REDUCTIONS REPORTED.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* THREE MILE ISLAND 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....PENNSYLVANIA

COUNTY.....DAUPHIN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...10 MI SE OF
HARRISBURG, PA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JUNE 5, 1974

DATE ELEC ENER 1ST GENER...JUNE 19, 1974

DATE COMMERCIAL OPERATE....SEPTEMBER 2, 1974

CONDENSER COOLING METHOD... COOLING TOWERS

CONDENSER COOLING WATER....SUSQUEHANNA RIVER

ELECTRIC RELIABILITY
COUNCIL.....MID-ATLANTIC
AREA COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....GPU NUCLEAR CORP.

CORPORATE ADDRESS.....P.O. BOX 480
MIDDLETOWN, PENNSYLVANIA 17057

CONTRACTOR
ARCHITECT/ENGINEER.....GILBERT ASSOCIATES

NUC STEAM SYS SUPPLIER...BABCOCK & WILCOX

CONSTRUCTOR.....UNITED ENG. & CONSTRUCTORS

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....R. CONTE

LICENSING PROJ MANAGER.....J. THOMA
DOCKET NUMBER.....50-289

LICENSE & DATE ISSUANCE....DPR-50, APRIL 19, 1974

PUBLIC DOCUMENT ROOM.....GOVERNMENT PUBLICATIONS SECTION
STATE LIBRARY OF PENNSYLVANIA
FORUM BUILDING
COMMONWEALTH AND WALNUT STREET
HARRISBURG, PENNSYLVANIA 17105

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

****
* THREE MILE ISLAND 1 *
****

OTHER ITEMS

NO INPUT PROVIDED.

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-344 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: F. J. UHMER (503) 556-3713 X495
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1280 X 0.95 = 1216
6. Design Electrical Rating (Net MWe): 1130
7. Maximum Dependable Capacity (Gross MWe): 1122
8. Maximum Dependable Capacity (Net MWe): 1050
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

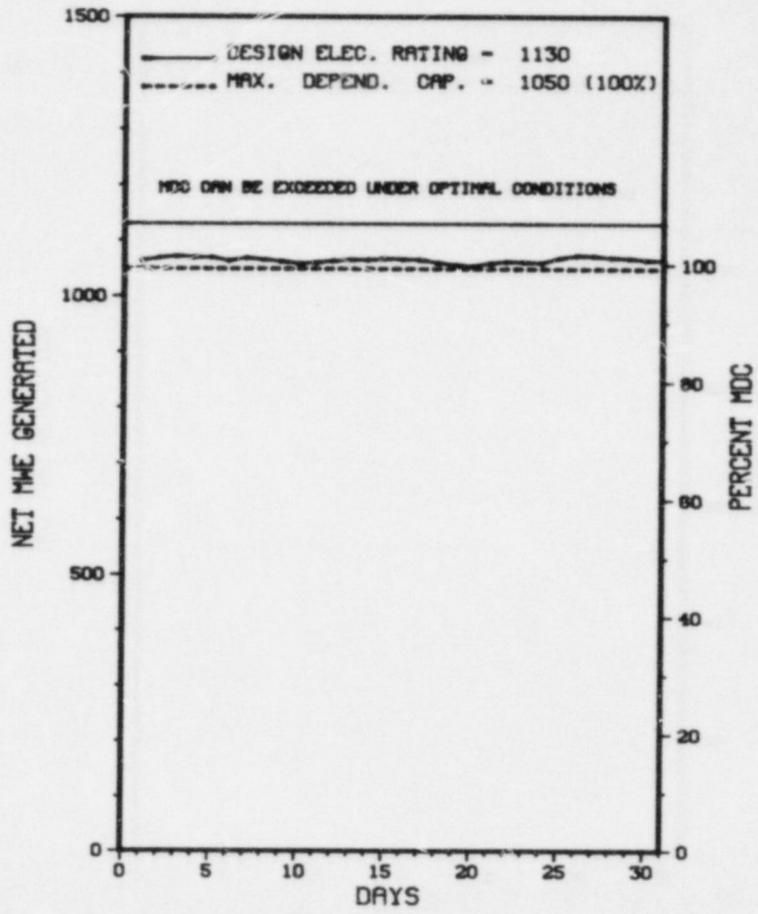
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	86,903.0
13. Hours Reactor Critical	744.0	3,554.1	54,104.5
14. Rx Reserve Shtdwn Hrs	.0	.0	3,875.4
15. Hrs Generator On-Line	744.0	3,494.7	52,549.0
16. Unit Reserve Shtdwn Hrs	.0	.0	3,237.0
17. Gross Therm Ener (MWH)	2,536,944	11,288,621	167,819,204
18. Gross Elec Ener (MWH)	831,253	3,629,381	54,447,697
19. Net Elec Ener (MWH)	792,740	3,439,164	51,500,438
20. Unit Service Factor	100.0	68.7	60.5
21. Unit Avail Factor	100.0	68.7	64.2
22. Unit Cap Factor (MDC Net)	101.5	64.4	56.4
23. Unit Cap Factor (DER Net)	94.3	59.8	52.4
24. Unit Forced Outage Rate	.0	1.6	14.7
25. Forced Outage Hours	.0	55.6	9,066.0

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: N/A

****
* TROJAN *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

TROJAN



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

TROJAN

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

TROJAN OPERATED ROUTINELY IN JULY WITH NO SHUTDOWNS OR POWER REDUCTIONS REPORTED.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure F-Admin B-Maint or Test G-Oper Error C-Refueling H-Other D-Regulatory Restriction E-Operator Training & License Examination	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

****
* TROJAN *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....OREGON

COUNTY.....COLUMBIA

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...32 MI N OF
PORTLAND, ORE

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...DECEMBER 15, 1975

DATE ELEC ENER 1ST GENER...DECEMBER 23, 1975

DATE COMMERCIAL OPERATE....MAY 20, 1976

CONDENSER COOLING METHOD...COOLING TOWERS

CONDENSER COOLING WATER....COOLING TOWER

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....PORTLAND GENERAL ELECTRIC

CORPORATE ADDRESS.....121 S.W. SALMON STREET
PORTLAND, OREGON 97204

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....S. RICHARDS

LICENSING PROJ MANAGER.....T. CHAN
DOCKET NUMBER.....50-344

LICENSE & DATE ISSUANCE....NPF-1, NOVEMBER 21, 1975

PUBLIC DOCUMENT ROOM.....LIBRARY ASSOCIATION OF PORTLAND
SOCIAL SCIENCES & SCIENCE DEPARTMENT
801 SW 10TH AVENUE
PORTLAND, OREGON 97205

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON AUGUST 11-22, 1986 (REPORT NO. 50-344/86-23) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JUNE 16 - AUGUST 4, 1986 (REPORT NO. 50-344/86-25) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JULY 14-18, 1986 (REPORT NO. 50-344/86-26) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF THE PROCUREMENT PROGRAM, AND ONSITE FOLLOWUP OF WRITTEN REPORTS OF NONROUTINE EVENTS BY ONE NRC INSPECTOR. DURING THIS INSPECTION, TWO INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ ENFORCEMENT CONFERENCE ON JULY 9, 1986 (REPORT NO. 50-344/86-27) SUMMARY: AN ENFORCEMENT CONFERENCE WAS HELD ON JULY 9, 1986, CONCERNING AN APPARENT VIOLATION IDENTIFIED DURING A SPECIAL INSPECTION OF QUALITY CONTROL ACTIVITIES (INSPECTION REPORT 50-344/86-24).

+ INSPECTION ON JULY 22-31, 1986 (REPORT NO. 50-344/86-28) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JULY 28 - AUGUST 1, 1986 (REPORT NO. 50-344/86-29) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

TECHNICAL SPECIFICATION 6.11 REQUIRES THAT PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10 CFR PART 20 AND SHALL BE APPROVED, MAINTAINED, AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE. THE LICENSEE'S RADIATION PROTECTION MANUAL DEVELOPED PURSUANT TO THIS SPECIFICATION REQUIRES IN SECTION II.B.4.A THAT A DETERMINATION OF THE MEDICAL STATUS OF EACH PERSON WHO MAY BE REQUIRED TO WEAR RESPIRATORY PROTECTIVE EQUIPMENT WILL BE MADE PRIOR TO HIS ASSIGNMENT TO USE IT, AND REPEATED ANNUALLY THEREAFTER. SECTION II.H.2.C STATES THAT, "RESPIRATOR WILL ONLY BE ISSUED TO PERSONNEL QUALIFIED TO USE THE RESPIRATOR. QUALIFICATIONS CONSIST OF ... 2) MEDICAL EVALUATION ..." CONTRARY TO THE REQUIREMENT, ON APRIL 16 AND 18, 1986, A WORKER WAS ISSUED A RESPIRATOR FOR USE WHOSE MEDICAL REEVALUATION HAD EXPIRED ON FEBRUARY 28, 1986, AND ANOTHER WORKER WAS ISSUED A RESPIRATOR FOR USE ON APRIL 30, 1986, WHOSE MEDICAL REEVALUATION HAD EXPIRED ON FEBRUARY 19, 1986. IN EACH CASE, RESPIRATORS WERE ISSUED FOR AND USED PURSUANT TO RADIATION WORK PERMIT REQUIREMENTS.
(8601 4)

10 CFR 20.401(B) REQUIRES, IN PART, THAT EACH LICENSEE SHALL MAINTAIN RECORDS SHOWING THE RESULTS OF SURVEYS REQUIRED BY 10 CFR 20.201(B) "SURVEYS." 10 CFR 20.201(B) REQUIRES THAT EACH LICENSEE MAKE OR CAUSE TO BE MADE SURVEYS AS: (1) MAY BE NECESSARY FOR THE LICENSEE TO COMPLY WITH THE REGULATIONS IN 10 CFR 20, AND (2) ARE REASONABLE UNDER THE CIRCUMSTANCES TO EVALUATE THE EXTENT OF RADIATION HAZARDS THAT MAY BE PRESENT. CONTRARY TO THE REQUIREMENT, THE LICENSEE DID NOT MAINTAIN RECORDS OF SURVEYS PERFORMED ON MAY 12 AND APRIL 19 AND 25, 1986, TO EVALUATE THE POSSIBLE INTAKE OF RADIOACTIVE MATERIALS OF WORKERS WHO RECEIVED OPEN SKIN OR FLESH WOUNDS WHILE WORKING IN CONTAMINATED AREAS.
(8601 5)

10 CFR 50, APPENDIX B, CRITERION V, AND THE TROJAN NUCLEAR PLANT NUCLEAR QUALITY ASSURANCE PROGRAM, SECTION 5.0, REQUIRE IN PART THAT ACTIVITIES AFFECTING QUALITY BE PRESCRIBED BY AND ACCOMPLISHED IN ACCORDANCE WITH DOCUMENTED PROCEDURES. MAINTENANCE PROCEDURE MP-5-1, "PRESSURIZER SAFETY VALVE INSERVICE TEST," ATTACHMENT I.B.5, STEPS 6.H AND 6.K, AND MAINTENANCE REQUEST 86-2120, WORK INSTRUCTIONS STEP 7, REQUIRED A QUALITY CONTROL INSPECTOR TO WITNESS THE BOLT TORQUING OF THE INLET FLANGE OF PRESSURIZER SAFETY VALVE #PSV-8010A, IN ACCORDANCE WITH QUALITY CONTROL PROCEDURE (QCP) 3, "VERIFICATION/WITNESSING INSPECTIONS." QCP 3, SECTION II, DEFINES WITNESSING AS, "AN INDEPENDENT CONFIRMATION, BY MEANS OF ACTUAL PHYSICAL OBSERVATION, THAT A CONDITION OR PROCESS COMPLIES WITH SPECIFIED REQUIREMENTS." CONTRARY TO THE REQUIREMENT, ON MAY 13, 1986, BOLT TORQUING OF THE INLET FLANGE OF VALVE PSV-8010A WAS NOT WITNESSED BY A QUALITY CONTROL INSPECTOR. THE QUALITY CONTROL INSPECTOR SIGNED DOCUMENTATION THAT THE BOLT TORQUING WAS WITNESSED.
(8602 3)

10 CFR 50, APPENDIX B, CRITERION XVI AND THE TROJAN NUCLEAR PLANT NUCLEAR QUALITY ASSURANCE PROGRAM, SECTION 15.2.3, REQUIRE IN PART THAT NONCONFORMANCES BE IDENTIFIED AND CORRECTED. NUCLEAR DIVISION PROCEDURE 600-1, "CONTROL OF NONCONFORMING MATERIALS, PARTS, AND COMPONENTS," SECTION 5.0, REQUIRES IN PART, THAT CORRECTIVE ACTION BE ASSIGNED AND IMPLEMENTED FOR NONCONFORMANCES. CONTRARY TO THE REQUIREMENT, CORRECTIVE ACTION WAS NOT ASSIGNED AND IMPLEMENTED FOR NONCONFORMANCE REPORT (NCR) NUMBER 86-024, WHICH DOCUMENTED NONCONFORMANCES ASSOCIATED WITH ELECTRICAL CONDUIT SUPPORT FOR THE 'A' TRAIN OF THE HYDROGEN RECOMBINER SYSTEM.
(8602 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

OTHER ITEMS

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

100% POWER OPERATIONS.

LAST IE SITE INSPECTION DATE: 08/04-15/86

INSPECTION REPORT NO: 50-344/86-23

R E P O R T S F R O M L I C E N S E E

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-06-L0	06-10-86	07-10-86	NAOH ADDITIVE FLOWRATE TEST FAILED DUE TO MISAPPLICATION OF KEROTEST GLOBE VALVES
86-07-L0	06-19-86	07-16-86	REACTOR TRIP DUE TO PERSONNEL ERROR RESULTING IN EXCEEDING LOW POWER RANGE TRIP SETPOINT

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1. Docket: 50-250 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: N. W. GRANT (305) 552-3675
4. Licensed Thermal Power (MWt): 2200
5. Nameplate Rating (Gross MWe): 894 X 0.85 = 760
6. Design Electrical Rating (Net MWe): 693
7. Maximum Dependable Capacity (Gross MWe): 700
8. Maximum Dependable Capacity (Net MWe): 666
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

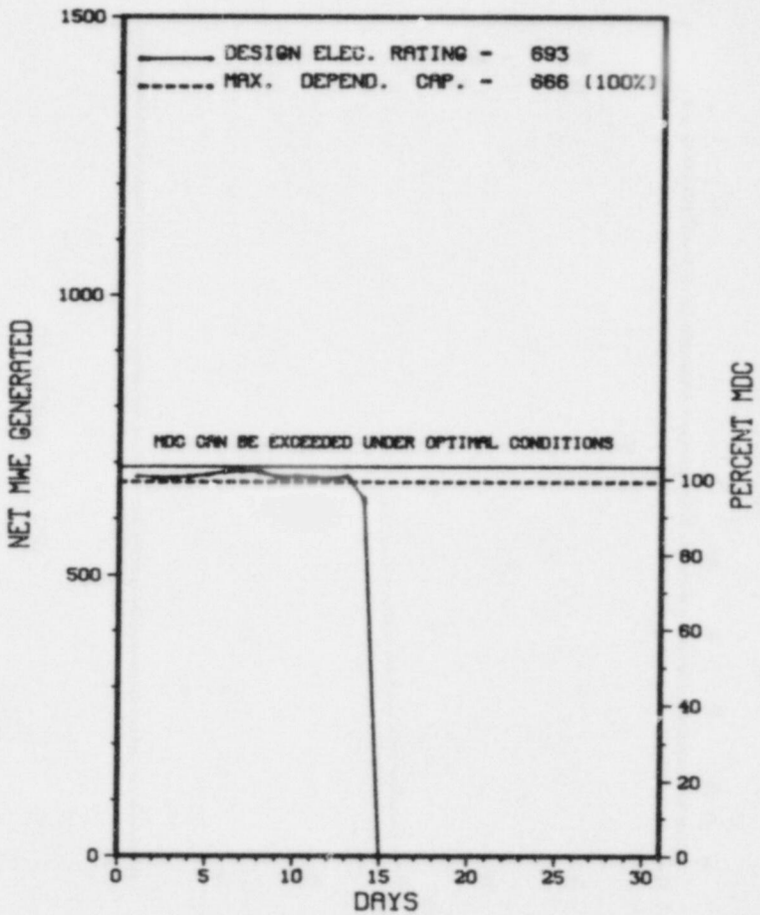
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	119,696.6
13. Hours Reactor Critical	337.9	3,539.4	84,336.3
14. Rx Reserve Shtdwn Hrs	.0	.0	844.3
15. Hrs Generator On-Line	337.6	3,489.1	81,896.1
16. Unit Reserve Shtdwn Hrs	.0	.0	121.8
17. Gross Therm Ener (MWH)	737,454	7,541,205	169,746,726
18. Gross Elec Ener (MWH)	237,900	2,469,640	54,341,185
19. Net Elec Ener (MWH)	222,954	2,335,244	51,444,82
20. Unit Service Factor	45.4	68.6	68.4
21. Unit Avail Factor	45.4	68.6	68.5
22. Unit Cap Factor (MDC Net)	45.0	68.9	66.1*
23. Unit Cap Factor (DER Net)	43.2	66.2	62.9
24. Unit Forced Outage Rate	.0	25.5	7.6
25. Forced Outage Hours	.0	1,191.5	6,226.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: 08/08/86

****
* TURKEY POINT 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

TURKEY POINT 3



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986 UNIT SHUTDOWNS / REDUCTIONS

****
* TURKEY POINT 3 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
08	07/15/86	S	406.4	B	1		EB	ENGINE	THE UNIT WAS SHUTDOWN TO PERFORM ELECTRICAL SYSTEM MODIFICATIONS REQUIRED FOR EMERGENCY DIESEL GENERATOR LOAD CONSIDERATIONS.

****
* SUMMARY *
****

TURKEY POINT 3 HAD 1 OUTAGE IN JULY AS DISCUSSED ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* TURKEY POINT 3 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....FLORIDA

COUNTY.....DADE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI S OF
MIAMI, FLA

TYPE OF REACTOR..... PWR

DATE INITIAL CRITICALITY...OCTOBER 20, 1972

DATE ELEC ENER 1ST GENER...NOVEMBER 2, 1972

DATE COMMERCIAL OPERATE....DECEMBER 14, 1972

CONDENSER COOLING METHOD...CLOSED CANAL

CONDENSER COOLING WATER....CLOSED CYCLE CANAL

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....FLORIDA POWER & LIGHT

CORPORATE ADDRESS.....9250 WEST FLAGLER STREET P.O. BOX 013100
MIAMI, FLORIDA 33174

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....T. PEEBLES

LICENSING PROJ MANAGER.....D. MCDONALD
DOCKET NUMBER.....50-250

LICENSE & DATE ISSUANCE....DPR-31, JULY 19, 1972

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FLORIDA INTERNATIONAL UNIVERSITY
MIAMI, FLORIDA 33199

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MARCH 23-28 AND APRIL 7-11 (86-18): THIS ROUTINE, ANNOUNCED INSPECTION WAS CONDUCTED ON SITE IN THE AREAS OF THE LICENSEE'S PHASE 1 AND PHASE 2 SELECTED SAFETY SYSTEM REVIEW, INSTRUMENT CALIBRATION, AND SELECTED SURVEILLANCES. ONE VIOLATION WAS IDENTIFIED FOR FOUR EXAMPLES OF FAILURE TO EITHER ESTABLISH OR IMPLEMENT PROCEDURES IN AREAS OF LIMITORQUE MAINTENANCE NORMAL VALVE LINEUPS FOR INTAKE COOLING WATER (ICW) AND COMPONENT COOLING WATER (CCW) SYSTEMS, AND MAINTENANCE OF THE EQUIPMENT OUT OF SERVICE (EOOS) LOG. ANOTHER VIOLATION WAS IDENTIFIED FOR FAILURE TO PROMPTLY IDENTIFY AND CORRECT DEFICIENCIES WITH RESPECT TO CONTROL OF COMPONENT COOLING WATER (CCW) FLOW TO THE SAFETY INJECTION (SI) PUMPS. THIS ITEM IS BEING CONSIDERED FOR ESCALATED ENFORCEMENT ACTION, THEREFORE, NO NOTICE OF VIOLATION FOR THIS ITEM IS BEING ISSUED AT THIS TIME.

INSPECTION APRIL 3-6, 16-18 (86-24): THIS SPECIAL INSPECTION INVOLVED THE AREAS OF ACCELERATED REQUALIFICATION TRAINING, EMERGENCY OPERATING PROCEDURES, AND THE EMERGENCY DIESEL GENERATOR LOADING SAFETY EVALUATION AND ASSOCIATED CONFIRMATION OF ACTION LETTER. TWO VIOLATIONS AND ONE DEVIATION WERE IDENTIFIED: A. DEVIATION (250,251/86-24-04) - FAILURE TO REVISE PROCEDURES TO ASSURE THAT DIESEL GENERATOR LOADING REMAINED NO MORE THAN 2845 KW AS COMMITTED BY CONFIRMATION OF ACTION LETTER; (B) VIOLATION (250,251/86-24-06) - FAILURE TO PROVIDE ADEQUATE CORRECTIVE ACTIONS TO PRECLUDE REPETITION OF TRAINING DEFICIENCIES ON THE GAMMA METRICS NEUTRON FLUX MONITOR; AND (C) VIOLATION (250,251/86-24-08) - FAILURE TO PERFORM 10CFR 50.59 EVALUATION OF EDG LOADS BEYOND THOSE ANALYZED IN JPE-L-86-59, REVISION 1. NO NOTICE OF VIOLATION FOR THIS ITEM WILL BE INCLUDED IN THE REPORT AS THIS MATTER IS BEING CONSIDERED FOR ENFORCEMENT ACTION AS PART OF A SEPARATE REPORT.

INSPECTION APRIL 28 - MAY 2 (86-29): THIS SPECIAL, ANNOUNCED INSPECTION WAS PERFORMED AT THE LICENSEE'S ENGINEERING OFFICE AND THE PLANT SITE TO REVIEW THE CONTROL AND DISTRIBUTION OF EMERGENCY ELECTRICAL LOADS WHICH ARE CONNECTED TO THE EMERGENCY DIESEL

INSPECTION SUMMARY

GENERATORS (EDG) IN THE EVENT OF A DESIGN BASE ACCIDENT (LOCA). THIS REPORT INCLUDES THE EVALUATION MADE AS THE RESULT OF A MEETING HELD IN REGION II OFFICES ON MAY 20, 1986 BETWEEN FP&L PERSONNEL AND NRC. ONE UNRESOLVED ITEM WAS IDENTIFIED - EMERGENCY DIESEL GENERATOR LOAD CONTROL.

INSPECTION MAY 12 - JUNE 9 (86-30): THIS ROUTINE, UNANNOUNCED INSPECTION ENTAILED DIRECT INSPECTION AT THE SITE, INCLUDING BACKSHIFT INSPECTION, IN THE AREAS OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, ANNUAL AND MONTHLY SURVEILLANCE OBSERVATIONS, MAINTENANCE OBSERVATIONS AND REVIEWS, OPERATIONAL SAFETY VERIFICATION, ENGINEERED SAFETY FEATURES WALKDOWN, INDEPENDENT INSPECTION, AND FOLLOW-UP OF PLANT EVENTS. VIOLATION - FAILURE TO MEET THE REQUIREMENTS OF TECHNICAL SPECIFICATION (TS) 6.8.1.

INSPECTION JUNE 2-6 (86-31): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED THE AREAS OF LICENSEE ACTIONS ON PREVIOUS ENFORCEMENT MATTERS (92701B AND 92702B), HOUSEKEEPING (54834B), MATERIAL IDENTIFICATION AND CONTROL (42902B), MATERIAL CONTROL (42940B), GULFALLOY SUPPLIED MATERIALS (92705B), INSERVICE TESTING OF PUMPS AND VALVES (61700 AND 61726), IE BULLETINS (IEBS) (92703B), FOLLOWUP ON IE NOTICES (IEN) (92717) AND INSPECTOR FOLLOWUP ITEMS. ONE VIOLATION WAS IDENTIFIED - 50-250/86-31-04: "FAILURE TO LOCK AND VERIFY THE LOCKED CONDITION OF VALVE 3-891A DURING IST". ONE DEVIATION WAS IDENTIFIED 50-250, 251/86-31-01: "FAILURE TO SUBMIT ISI RELIEF REQUEST AS COMMITTED".

ENFORCEMENT SUMMARY

CONTRARY TO TS 6.8.1 AND APPENDIX A OF USNRC REGULATORY GUIDE 1.33, THE LICENSEE APPROVED A NEW PROCEDURE FOR MOV MAINTENANCE, 0-CME-102.1, BUT IT HAD NOT BEEN ISSUED FOR USE YET. THIS PROCEDURE ALSO INCLUDED THE SAME TORQUE SWITCH SETTINGS AS THE TOP. THE LICENSEE WAS INFORMED THAT TOP 166 AND 0-CME-102.1 WERE INADEQUATE, IN THAT THEY SPECIFY THE INCORRECT TORQUE SWITCH SETTINGS RECOMMENDED BY LIMITORQUE. THE INSPECTOR CONSIDERS THAT THE LICENSEE SET MOVS-4-750/751 TORQUE SWITCHES TO TECHNICALLY ACCEPTABLE VALUE BUT DID NOT REVISE THE PROCEDURES PRIOR TO ACCOMPLISHING THESE ACTIONS AND CONSEQUENTLY PERFORMED WORK CONTRARY TO THE PROCEDURE.

(8601 4)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVI, AFTER THE NRC RESIDENT INSPECTION STAFF IDENTIFIED THAT PRIOR TO AUGUST 14, 1985, NO OPERATOR TRAINING HAD BEEN PROVIDED ON THE USE OF THE GAMMA METRICS NEUTRON FLUX MONITOR PURSUANT TO LICENSEE COMMITMENTS, THE LICENSEE'S CORRECTIVE ACTIONS FAILED TO PRECLUDE REPETITION, IN THAT, REPLACEMENT HOT LICENSE CLASS 10 OPERATORS RECEIVING LICENSES AFTER FEBRUARY 1986, WERE NOT PROVIDED TRAINING IN THE USE OF THE GAMMA METRICS NEUTRON FLUX MONITOR. CONTRARY TO TS 6.8.1, REGULATORY GUIDE 1.33 AND AP 0190.19: (1) ON APRIL 9, 1986, PWO 6230 WAS ISSUED AS A PRIORITY CLASS "AA" WORK ORDER BUT THE MAINTENANCE TECHNICIAN DID NOT THOROUGHLY DOCUMENT HIS MAINTENANCE ACTIONS, IN THAT HE FAILED TO ORIGINATE A CALIBRATION RECORD SHEET FOR A FLOW METER HE INSTALLED. ADDITIONALLY, HE FAILED TO INDICATE ON THE PWO THAT HE HAD PERFORMED THE REQUIRED METER CALIBRATION. (2) ON MAY 3, 1986, THE PSN AUTHORIZED WORK TO START ON PRIORITY CLASS "AA" PWO 6379 WHEN THE PLANT WAS NOT IN A LOAD THREATENING CONDITION NOR IN AN ACTION STATEMENT OF TS 3.0.1. ADDITIONALLY, THE PWO WAS NOT MADE AVAILABLE FOR REVIEW BY THE QC DEPARTMENT WITHIN ONE DAY. TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES BE IMPLEMENTED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTIONS 5.1 AND 5.3 OF ANSI N18.7-1972 AND APPENDIX A OF USNRC REGULATORY GUIDE 1.33. (1) CONTRARY TO THE ABOVE, ON MAY 2, 1986, AN OPERATOR FAILED TO PROPERLY IMPLEMENT OP 1004.2, IN THAT WHILE PERFORMING STEP 8.61 HE REMAINED AT PROTECTION INSTRUMENT RACK 36 AND, WHILE PERFORMING STEP 8.63, TRIPPED THE "A" REACTOR TRIP BREAKER THEREBY INADVERTENTLY TRIPPING THE UNIT 3 REACTOR. (2) CONTRARY TO THE ABOVE, ON MAY 2, 1986, AN OPERATOR FAILED TO PROPERLY IMPLEMENT OP 4304.1 ON TWO CONSECUTIVE OCCASIONS, IN THAT DURING AN INITIAL START OF THE "B" EDG, SECTION 8.7 WAS NOT IMPLEMENTED, CAUSING THE EDG TO FAIL TO START. WHILE REALIGNING THE "B" EDG FOR A SUBSEQUENT START ATTEMPT, THE OPERATOR IMPROPERLY IMPLEMENTED SECTION 8.3 IN THAT HE CLOSED THE STARTING AIR SUPPLY VALVE FOR THE "A" EDG, RENDERING THE 'A' EDG TEMPORARILY INOPERABLE. (3) CONTRARY TO THE ABOVE, ON MAY 9, 1986, AN OPERATOR FAILED TO PROPERLY IMPLEMENT OP 4304.1 IN THAT, DURING A LINEUP TO START THE "B" EDG, SECTION 8.11 WAS NOT PROPERLY IMPLEMENTED BECAUSE THE OPERATOR PRESSED THE LOCAL START PUSHBUTTON INSTEAD OF THE FUEL OIL PRIME PUSHBUTTON. THIS ACTION CAUSED AN UNEXPECTED LOCAL START OF THE "B" EDG. CONTRARY TO 10 CFR, APPENDIX B, CRITERION VI, AS IMPLEMENTED BY THE FPLTQAR, TQR 6.0, QP 6.6, AND AP 0103.10, AS OF MAY 13, 1986, DRAWING

ENFORCEMENT SUMMARY

5610-T-E-4536, REVISION 0, SHEETS 1 AND 2, ENTITLED "DIESEL GENERATOR B", RESPECTIVELY, WERE NOT ACCURATE, IN THAT: (1) THE DRAWING SHEETS FAILED TO SHOW THE EXISTENCE OF A FUEL OIL SKID TANK DRAIN VALVE FOR EACH EDG; (2) THE DRAWING SHEETS SHOWED THAT VALVE 292A AND 292B, DRAINS FOR THE EDG RADIATOR COOLING SYSTEM, WERE NORMALLY CLOSED VALVES WHEN ACTUALLY THEY WERE NORMALLY OPEN VALVES; (3) NUMEROUS VALVE NUMBERS SPECIFIED ON THE DRAWING SHEETS CONFLICTED WITH THE VALVE NUMBERS UTILIZED BY APPROVED OPERATING PROCEDURE 0-OP-023; AND (4) DRAWING SHEET 1 SHOWED THE PRESENCE OF STARTING AIR FLASK DRAIN VALVE 269A, WHICH DOES NOT EXIST, AND DRAWING SHEET 2 SHOWED A STARTING AIR FLASK DRAIN PIPING CONFIGURATION WHICH WAS NOT ACCURATE. CONTRARY TO THE REQUIREMENTS OF TS 6.8.1, AS OF MAY 12, 1986, PROCEDURE 0-OP-023, EMERGENCY DIESEL GENERATOR, DID NOT ADEQUATELY ESTABLISH PROCEDURES FOR THE STARTUP AND OPERATION OF THE "A" AND "B" EDGS, IN THAT: (1) THE PROCEDURE DID NOT ADDRESS THE CONTROL OF VALVES 292 AND 293 FOR EITHER THE "A" OR "B" EDG RADIATOR COOLING WATER SYSTEM DRAINS, AND THE DRAIN VALVES FOR THE "A" OR "B" EDG FUEL OIL SKID TANK; (2) THE PROCEDURE ADDRESSED THE POSITION OF SKID TANK SOLENOID VALVE SV-3-3522 BYPASS LINE ISOLATION VALVE 70-048A, WHICH HAS NOT BEEN INSTALLED FOR THE "A" EDG; AND (3) THE PROCEDURE DID NOT ADDRESS THE CONTROL OF VALVE 269B FOR THE STARTING AIR FLASK DRAINS FOR THE "B" EDG.
(8602 4)

CONTRARY TO TS 6.8.1 AND APPENDIX A OF USNRC REGULATORY GUIDE 1.33, OPERATING PROCEDURE 14004.1, STEAM GENERATOR PROTECTION CHANNELS - PERIODIC TEST, DATED OCTOBER 25, 1985 WAS NOT PROPERLY IMPLEMENTED ON JUNE 5, 1986. WHILE PERFORMING STEPS 8.3.79 THROUGH 8.3.83, THE CHANNEL 3 STEAM HEADER PRESSURE AND STEAM BREAK PROTECTION PORTION OF THE PROCEDURE, A POWER SUPPLY WAS SUBSTITUTED FOR THE PROCEDURALLY REQUIRED TRANSMITTER SIMULATOR, AND TEST SIGNAL INPUT AND MONITORING LOCATIONS OTHER THAN THOSE SPECIFIED BY THE PROCEDURE WERE USED.
(8603 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

SELECT SAFETY SYSTEM OPERABILITY REVIEW IN PROGRESS.

FACILITY ITEMS (PLANS AND PROCEDURES):

PROCEDURE UPGRADE PROGRAM (PUP) IN PROGRESS.

MANAGERIAL ITEMS:

PEP IN PROGRESS.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 2-6, 1986 +

INSPECTION REPORT NO: 50-250/86-31 +

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-018	04/16/86	05/15/86	UNIT SHUTDOWN DUE TO EXCEEDING ADMINISTRATIVE GUIDELINES FOR INTAKE COOLING WATER SYSTEM OPERATION; CAUSE - ICW INLET TEMPERATURE INCREASED SUCH THAT 3 CCW HEAT EXCHANGERS WERE REQUIRED.
86-024	06/11/86	07/11/86	'3B' INTAKE COOLING WATER PUMP OUT-OF-SERVICE AND NOT RETURNED TO OPERABLE STATUS WITHIN TECHNICAL SPECIFICATION REQUIREMENTS; CAUSE - ADDITIONAL VERIFICATION PERFORMED.
86-025	06/12/86	07/14/86	TECHNICAL SPECIFICATION 3.6.D.1 EXCEEDED - 3A AND 3C CHARGING PUMPS OUT-OF-SERVICE, UNIT IN A 24 HOUR LCO; CAUSE - A CRACKED WELD ON THE DISCHARGE LINE DISCOVERED.
86-026	06/12/86	07/14/86	LEFT TURBINE STOP VALVE FAILED TO CLOSE COMPLETELY; CAUSE - MECHANICAL FAILURE OF THE STOP VALVE'S PILOT VALVE.
86-028	06/25/86	06/30/86	10CFR PART 21 DEFICIENCY EXISTED CONCERNING THE POTENTIAL FOR LOSS OF THE MINIMUM FLOW FOR THE SAFETY INJECTION PUMPS.

1. Docket: 50-251 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: N. W. GRANT (305) 552-3675
4. Licensed Thermal Power (MWt): 2200
5. Nameplate Rating (Gross MWe): 894 X 0.85 = 760
6. Design Electrical Rating (Net MWe): 693
7. Maximum Dependable Capacity (Gross MWe): 700
8. Maximum Dependable Capacity (Net MWe): 666
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	113,424.0
13. Hours Reactor Critical	.0	225.4	77,860.8
14. Rx Reserve Shtdwn Hrs	.0	.0	166.6
15. Hrs Generator On-Line	.0	225.1	75,327.5
16. Unit Reserve Shtdwn Hrs	.0	.0	31.2
17. Gross Therm Ener (MWH)	0	440,167	159,486,896
18. Gross Elec Ener (MWH)	0	139,235	50,781,867
19. Net Elec Ener (MWH)	-3,807	121,220	48,085,477
20. Unit Service Factor	.0	4.4	66.4
21. Unit Avail Factor	.0	4.4	66.4
22. Unit Cap Factor (MDC Net)	.0	3.6	65.1*
23. Unit Cap Factor (DER Net)	.0	3.4	61.2
24. Unit Forced Outage Rate	.0	.0	6.5
25. Forced Outage Hours	.0	.0	4,824.7

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

NONE

27. If Currently Shutdown Estimated Startup Date: 08/19/86

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* TURKEY POINT 4 *
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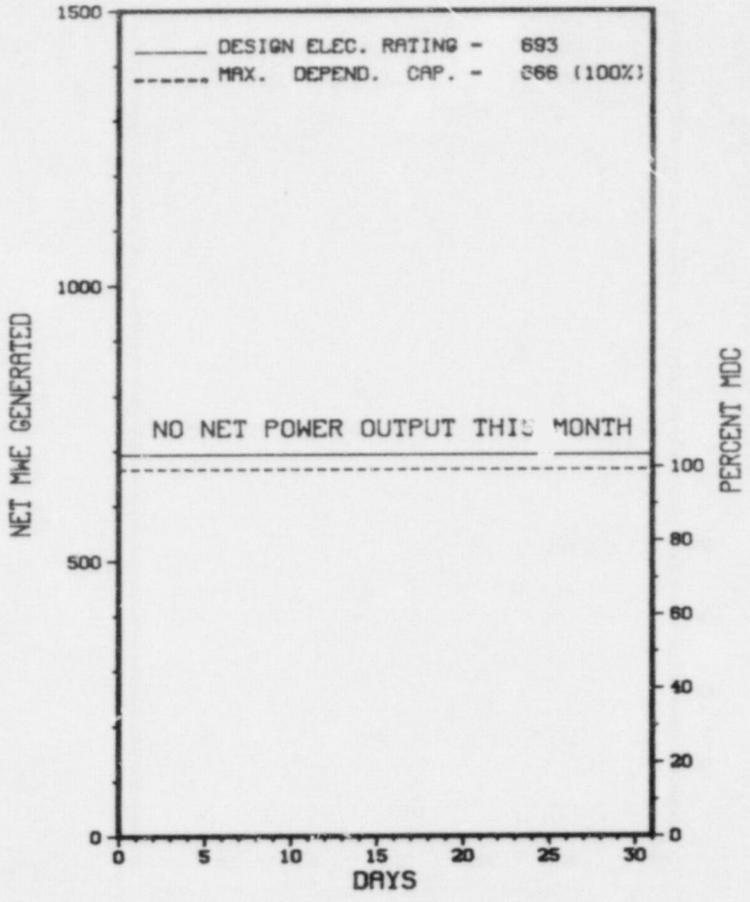
AVERAGE DAILY POWER LEVEL (MWe) PLOT

TURKEY POINT 4



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

TURKEY POINT 4

No	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
02	01/10/86	S	744.0	C	4		RC	FUELXX	UNIT #4 REMAINED SHUTDOWN FOR REFUELING AND SCHEDULED MAINTENANCE.

SUMMARY

TURKEY POINT 4 REMAINS SHUTDOWN FOR REFUELING AND MAINTENANCE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* TURKEY POINT 4 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....FLORIDA

COUNTY.....DADE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI S OF
MIAMI, FLA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JUNE 11, 1973

DATE ELEC ENER 1ST GENER...JUNE 21, 1973

DATE COMMERCIAL OPERATE....SEPTEMBER 7, 1973

CONDENSER COOLING METHOD...CLOSED CANAL

CONDENSER COOLING WATER....CLOSED CYCLE CANAL

ELECTRIC RELIABILITY
COUNCIL.....SOUTHEASTERN ELECTRIC
RELIABILITY COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....FLORIDA POWER & LIGHT

CORPORATE ADDRESS.....9250 WEST FLAGLER STREET P.O. BOX 013100
MIAMI, FLORIDA 33174

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....II

IE RESIDENT INSPECTOR.....T. PEEBLES

LICENSING PROJ MANAGER.....D. MCDONALD
DOCKET NUMBER.....50-251

LICENSE & DATE ISSUANCE....DPR-41, APRIL 10, 1973

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FLORIDA INTERNATIONAL UNIVERSITY
MIAMI, FLORIDA 33199

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION MARCH 23-28 AND APRIL 7-11 (86-18): THIS ROUTINE, ANNOUNCED INSPECTION WAS CONDUCTED ON SITE IN THE AREAS OF THE LICENSEE'S PHASE 1 AND PHASE 2 SELECTED SAFETY SYSTEM REVIEW, INSTRUMENT CALIBRATION, AND SELECTED SURVEILLANCES. ONE VIOLATION WAS IDENTIFIED FOR FOUR EXAMPLES OF FAILURE TO EITHER ESTABLISH OR IMPLEMENT PROCEDURES IN AREAS OF LIMITORQUE MAINTENANCE NORMAL VALVE LINEUPS FOR INTAKE COOLING WATER (ICW) AND COMPONENT COOLING WATER (CCW) SYSTEMS, AND MAINTENANCE OF THE EQUIPMENT OUT OF SERVICE (EOOS) LOG. ANOTHER VIOLATION WAS IDENTIFIED FOR FAILURE TO PROMPTLY IDENTIFY AND CORRECT DEFICIENCIES WITH RESPECT TO CONTROL OF COMPONENT COOLING WATER (CCW) FLOW TO THE SAFETY INJECTION (SI) PUMPS. THIS ITEM IS BEING CONSIDERED FOR ESCALATED ENFORCEMENT ACTION, THEREFORE, NO NOTICE OF VIOLATION FOR THIS ITEM IS BEING ISSUED AT THIS TIME.

INSPECTION APRIL 3-6, 16-18 (86-24): THIS SPECIAL INSPECTION INVOLVED THE AREAS OF ACCELERATED REQUALIFICATION TRAINING, EMERGENCY OPERATING PROCEDURES, AND THE EMERGENCY DIESEL GENERATOR LOADING SAFETY EVALUATION AND ASSOCIATED CONFIRMATION OF ACTION LETTER. TWO VIOLATIONS AND ONE DEVIATION WERE IDENTIFIED: A. DEVIATION (250,251/86-24-04) - FAILURE TO REVISE PROCEDURES TO ASSURE THAT DIESEL GENERATOR LOADING REMAINED NO MORE THAN 2845 KW AS COMMITTED BY CONFIRMATION OF ACTION LETTER; (B) VIOLATION (250,251/86-24-06) - FAILURE TO PROVIDE ADEQUATE CORRECTIVE ACTIONS TO PRECLUDE REPETITION OF TRAINING DEFICIENCIES ON THE GAMMA METRICS NEUTRON FLUX MONITOR; AND (C) VIOLATION (250,251/86-24-08) - FAILURE TO PERFORM 10CFR 50.59 EVALUATION OF EDG LOADS BEYOND THOSE ANALYZED IN JPE-L-86-59, REVISION 1. NO NOTICE OF VIOLATION FOR THIS ITEM WILL BE INCLUDED IN THE REPORT AS THIS MATTER IS BEING CONSIDERED FOR ENFORCEMENT ACTION AS PART OF A SEPARATE REPORT.

INSPECTION APRIL 28 - MAY 2 (86-29): THIS SPECIAL, ANNOUNCED INSPECTION WAS PERFORMED AT THE LICENSEE'S ENGINEERING OFFICE AND THE PLANT SITE TO REVIEW THE CONTROL AND DISTRIBUTION OF EMERGENCY ELECTRICAL LOADS WHICH ARE CONNECTED TO THE EMERGENCY DIESEL

INSPECTION SUMMARY

GENERATORS (EDG) IN THE EVENT OF A DESIGN BASE ACCIDENT (LOCA). THIS REPORT INCLUDES THE EVALUATION MADE AS THE RESULT OF A MEETING HELD IN REGION II OFFICES ON MAY 20, 1986 BETWEEN FP&L PERSONNEL AND NRC. ONE UNRESOLVED ITEM WAS IDENTIFIED - EMERGENCY DIESEL GENERATOR LOAD CONTROL.

INSPECTION MAY 12 - JUNE 9 (86-30): THIS ROUTINE, UNANNOUNCED INSPECTION ENTAILED DIRECT INSPECTION AT THE SITE, INCLUDING BACKSHIFT INSPECTION, IN THE AREAS OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, ANNUAL AND MONTHLY SURVEILLANCE OBSERVATIONS, MAINTENANCE OBSERVATIONS AND REVIEWS, OPERATIONAL SAFETY VERIFICATION, ENGINEERED SAFETY FEATURES WALKDOWN, INDEPENDENT INSPECTION, AND FOLLOW-UP OF PLANT EVENTS. VIOLATION - FAILURE TO MEET THE REQUIREMENTS OF TECHNICAL SPECIFICATION (TS) 6.8.1.

INSPECTION JUNE 2-6 (86-31): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED THE AREAS OF LICENSEE ACTIONS ON PREVIOUS ENFORCEMENT MATTERS (92701B AND 92702B), HOUSEKEEPING (54834B), MATERIAL IDENTIFICATION AND CONTROL (42902B), MATERIAL CONTROL (42940B), GULFALLOY SUPPLIED MATERIALS (92705B), INSERVICE TESTING OF PUMPS AND VALVES (61700 AND 61726), IE BULLETINS (IEBS) (92703B), FOLLOWUP ON IE NOTICES (IEN) (92717) AND INSPECTOR FOLLOWUP ITEMS. ONE VIOLATION WAS IDENTIFIED - 50-250/86-31-04: "FAILURE TO LOCK AND VERIFY THE LOCKED CONDITION OF VALVE 3-891A DURING IST". ONE DEVIATION WAS IDENTIFIED 50-250, 251/86-31-01: "FAILURE TO SUBMIT ISI RELIEF REQUEST AS COMMITTED".

ENFORCEMENT SUMMARY

CONTRARY TO TS 6.8.1 AND APPENDIX A OF USNRC REGULATORY GUIDE 1.33, THE LICENSEE APPROVED A NEW PROCEDURE FOR MOV MAINTENANCE, O-CME-102.1, BUT IT HAD NOT BEEN ISSUED FOR USE YET. THIS PROCEDURE ALSO INCLUDED THE SAME TORQUE SWITCH SETTINGS AS THE TOP. THE LICENSEE WAS INFORMED THAT TOP 166 AND O-CME-102.1 WERE INADEQUATE, IN THAT THEY SPECIFY THE INCORRECT TORQUE SWITCH SETTINGS RECOMMENDED BY LIMITORQUE. THE INSPECTOR CONSIDERS THAT THE LICENSEE SET MOVS-4-750/751 TORQUE SWITCHES TO TECHNICALLY ACCEPTABLE VALUE BUT DID NOT REVISE THE PROCEDURES PRIOR TO ACCOMPLISHING THESE ACTIONS AND CONSEQUENTLY PERFORMED WORK CONTRARY TO THE PROCEDURE.

(8601 4)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVI, AFTER THE NRC RESIDENT INSPECTION STAFF IDENTIFIED THAT PRIOR TO AUGUST 14, 1985, NO OPERATOR TRAINING HAD BEEN PROVIDED ON THE USE OF THE GAMMA METRICS NEUTRON FLUX MONITOR PURSUANT TO LICENSEE COMMITMENTS, THE LICENSEE'S CORRECTIVE ACTIONS FAILED TO PRECLUDE REPETITION, IN THAT, REPLACEMENT HOT LICENSE CLASS 10 OPERATORS RECEIVING LICENSES AFTER FEBRUARY 1986, WERE NOT PROVIDED TRAINING IN THE USE OF THE GAMMA METRICS NEUTRON FLUX MONITOR. TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES BE IMPLEMENTED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTIONS 5.1 AND 5.3 OF ANSI N18.7-1972 AND APPENDIX A OF USNRC REGULATORY GUIDE 1.33. (1) CONTRARY TO THE ABOVE, ON MAY 2, 1986, AN OPERATOR FAILED TO PROPERLY IMPLEMENT OP 1004.2, IN THAT WHILE PERFORMING STEP 8.61 HE REMAINED AT PROTECTION INSTRUMENT RACK 36 AND, WHILE PERFORMING STEP 8.63, TRIPPED THE "A" REACTOR TRIP BREAKER THEREBY INADVERTENTLY TRIPPING THE UNIT 3 REACTOR. (2) CONTRARY TO THE ABOVE, ON MAY 2, 1986, AN OPERATOR FAILED TO PROPERLY IMPLEMENT OP 4304.1 ON TWO CONSECUTIVE OCCASIONS, IN THAT DURING AN INITIAL START OF THE "B" EDG, SECTION 8.7 WAS NOT IMPLEMENTED, CAUSING THE EDG TO FAIL TO START. WHILE REALIGNING THE "B" EDG FOR A SUBSEQUENT START ATTEMPT, THE OPERATOR IMPROPERLY IMPLEMENTED SECTION 8.3 IN THAT HE CLOSED THE STARTING AIR SUPPLY VALVE FOR THE "A" EDG, RENDERING THE 'A' EDG TEMPORARILY INOPERABLE. (3) CONTRARY TO THE ABOVE, ON MAY 9, 1986, AN OPERATOR FAILED TO PROPERLY IMPLEMENT OP 4304.1 IN THAT, DURING A LINEUP TO START THE "B" EDG, SECTION 8.11 WAS NOT PROPERLY IMPLEMENTED BECAUSE THE OPERATOR PRESSED THE LOCAL START PUSHBUTTON INSTEAD OF THE FUEL OIL PRIME PUSHBUTTON. THIS ACTION CAUSED AN UNEXPECTED LOCAL START OF THE "B" EDG. CONTRARY TO 10 CFR, APPENDIX B, CRITERION VI, AS IMPLEMENTED BY THE FPLTQAR, TQR 6.0, QP 6.6, AND AP 0103.10, AS OF MAY 13, 1986, DRAWING 5610-T-E-4536, REVISION 0, SHEETS 1 AND 2, ENTITLED "DIESEL GENERATOR B", RESPECTIVELY, WERE NOT ACCURATE, IN THAT: (1) THE DRAWING SHEETS FAILED TO SHOW THE EXISTENCE OF A FUEL OIL SKID TANK DRAIN VALVE FOR EACH EDG; (2) THE DRAWING SHEETS SHOWED THAT VALVE 292A AND 292B, DRAINS FOR THE EDG RADIATOR COOLING SYSTEM, WERE NORMALLY CLOSED VALVES WHEN ACTUALLY THEY WERE NORMALLY OPEN VALVES; (3) NUMEROUS VALVE NUMBERS SPECIFIED ON THE DRAWING SHEETS CONFLICTED WITH THE VALVE NUMBERS UTILIZED BY APPROVED OPERATING PROCEDURE O-OP-023; AND (4) DRAWING SHEET 1 SHOWED THE PRESENCE OF STARTING AIR FLASK DRAIN VALVE 269A, WHICH DOES NOT EXIST, AND DRAWING SHEET 2 SHOWED A

ENFORCEMENT SUMMARY

STARTING AIR FLASK DRAIN PIPING CONFIGURATION WHICH WAS NOT ACCURATE. CONTRARY TO THE REQUIREMENTS OF TS 6.8.1, AS OF MAY 12, 1986, PROCEDURE 0-OP-023, EMERGENCY DIESEL GENERATOR, DID NOT ADEQUATELY ESTABLISH PROCEDURES FOR THE STARTUP AND OPERATION OF THE "A" AND "B" EDGS, IN THAT: (1) THE PROCEDURE DID NOT ADDRESS THE CONTROL OF VALVES 292 AND 293 FOR EITHER THE "A" OR "B" EDG RADIATOR COOLING WATER SYSTEM DRAINS, AND THE DRAIN VALVES FOR THE "A" OR "B" EDG FUEL OIL SKID TANK; (2) THE PROCEDURE ADDRESSED THE POSITION OF SKID TANK SOLENOID VALVE SV-3-3522 BYPASS LINE ISOLATION VALVE 70-048A, WHICH HAS NOT BEEN INSTALLED FOR THE "A" EDG; AND (3) THE PROCEDURE DID NOT ADDRESS THE CONTROL OF VALVE 269B FOR THE STARTING AIR FLASK DRAINS FOR THE "B" EDG. (8602 4)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

SELECT SAFETY SYSTEM OPERABILITY REVIEW IN PROGRESS.

FACILITY ITEMS (PLANS AND PROCEDURES):

PROCEDURE UPGRADE PROGRAM (PUP) IN PROGRESS.

MANAGERIAL ITEMS:

PEP IN PROGRESS.

PLANT STATUS:

REFUELING OUTAGE. PLANT IN COLD SHUTDOWN.

LAST IE SITE INSPECTION DATE: JUNE 2-6, 1986 +

INSPECTION REPORT NO: 50-251/86-31 +

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-011	06/13/86	07/14/86	PROCESS RADIATION MONITOR R-11 SPIKED HIGH, ACTUATING THE CONTAINMENT AND CONTROL ROOM VENTILATION ISOLATION LOGIC; CAUSE - KEYING OF A FM RADIO IN THE CONTROL ROOM.

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1. Docket: 50-271 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: G. A. WALLIN (802) 257-7711 X2272

4. Licensed Thermal Power (MWt): 1593

5. Nameplate Rating (Gross MWe): 626 X 0.9 = 563

6. Design Electrical Rating (Net MWe): 514

7. Maximum Dependable Capacity (Gross MWe): 535

8. Maximum Dependable Capacity (Net MWe): 504

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	121,489.8
13. Hours Reactor Critical	714.1	714.5	93,825.4
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	648.6	648.6	91,366.8
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	915,088	915,088	133,024,706
18. Gross Elec Ener (MWH)	290,965	290,965	44,246,867
19. Net Elec Ener (MWH)	271,725	271,725	41,971,975
20. Unit Service Factor	87.2	12.8	75.2
21. Unit Avail Factor	87.2	12.8	75.2
22. Unit Cap Factor (MDC Net)	72.5	10.6	68.5
23. Unit Cap Factor (DER Net)	71.1	10.4	67.2
24. Unit Forced Outage Rate	.0	.0	6.9
25. Forced Outage Hours	.0	.0	5,466.6

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

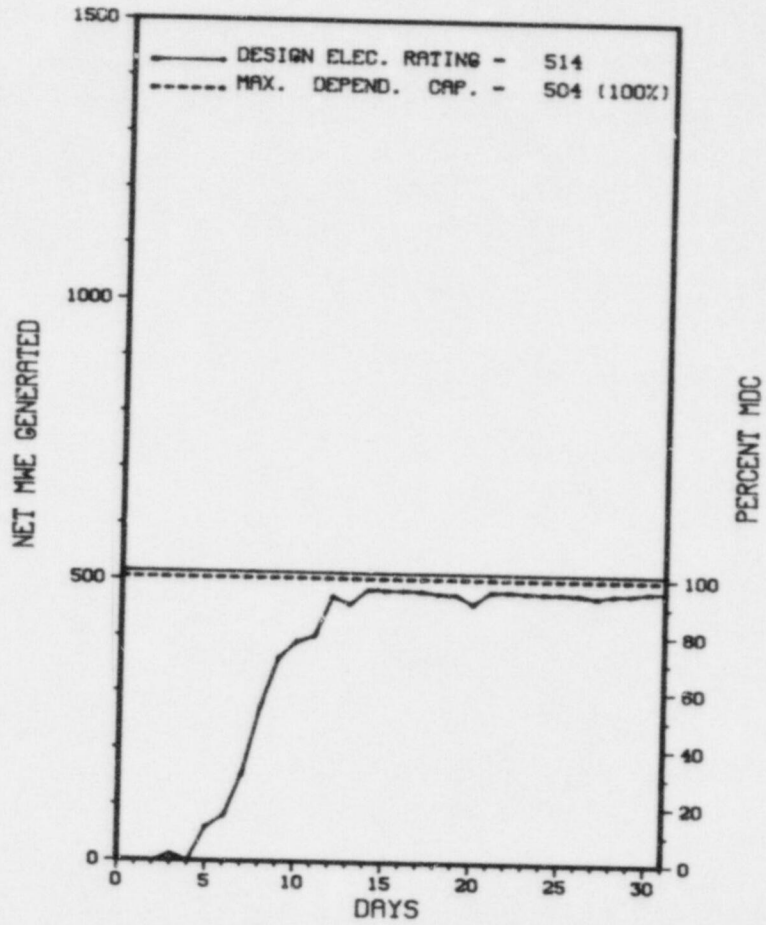
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* VERMONT YANKEE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

VERMONT YANKEE 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

VERMONT YANKEE 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
85-14	07/01/86	S	63.0	C	1				1985/1986 REFUELING/MAINTENANCE/RECIRC PIPE REPLACEMENT OUTAGE.
86-01	07/03/86	S	32.4	B	1				TURBINE OVERSPEED TESTING.
86-02	07/10/86	S	0.0	B	5		CB	PUMPXX	POWER REDUCTION FOR RECIRC PIPING STABILITY TESTING.
86-03	07/13/86	S	0.0	B	5		CB	PUMPXX	POWER REDUCTION FOR RECIRC PIPING STABILITY TESTING.
86-03-	07/13/86	S	0.0	B	5		RB	CONROD	POWER REDUCTION FOR TURBINE AND CONTROL ROD SURVEILLANCE.

SUMMARY

VERMONT YANKEE EXPERIENCED 2 OUTAGES AND 3 POWER REDUCTIONS IN JULY AS DISCUSSED ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* VERMONT YANKEE 1 *
****

FACILITY DATA

FACILITY DESCRIPTION

LOCATION
STATE.....VERMONT

COUNTY.....WINDHAM

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...5 MI S OF
BRATTLEBORO, VT

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...MARCH 24, 1972

DATE ELEC ENER 1ST GENER...SEPTEMBER 20, 1972

DATE COMMERCIAL OPERATE....NOVEMBER 30, 1972

CONDENSER COOLING METHOD...COOLING TOWER

CONDENSER COOLING WATER....CONNECTICUT RIVER

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....VERMONT YANKEE NUCLEAR POWER

CORPORATE ADDRESS.....RD #5, BOX 169, FERRY ROAD
BRATTLEBORO, VERMONT 05301

CONTRACTOR
ARCHITECT/ENGINEER.....EBASCO

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....EBASCO

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....W. RAYMOND

LICENSING PROJ MANAGER.....V. ROONEY
DOCKET NUMBER.....50-271

LICENSE & DATE ISSUANCE....DPR-28, FEBRUARY 28, 1973

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224 MAIN STREET
BRATTLEBORO, VERMONT 05301

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-397 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: LEONARD HUTCHISON (509) 377-2501 X2486
4. Licensed Thermal Power (MWt): 3323
5. Nameplate Rating (Gross MWe): 1201
6. Design Electrical Rating (Net MWe): 1100
7. Maximum Dependable Capacity (Gross MWe): 1140
8. Maximum Dependable Capacity (Net MWe): 1095
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	14,287.2
13. Hours Reactor Critical	717.4	2,875.0	10,191.2
14. Rx Reserve Shtdwn Hrs	.0	340.4	340.4
15. Hrs Generator On-Line	7[illegible].2	2,656.9	9,683.4
16. Unit Reserve Shtdwn Hrs	.0	381.7	381.7
17. Gross Therm Ener (MWH)	2,078,400	6,318,330	23,882,610
18. Gross Elec Ener (MWH)	696,850	2,086,620	7,909,870
19. Net Elec Ener (MWH)	667,000	1,998,390	7,585,163
20. Unit Service Factor	95.3	52.2	67.8
21. Unit Avail Factor	95.3	59.7	70.4
22. Unit Cap Factor (MDC Net)	81.9	35.9	48.5
23. Unit Cap Factor (DER Net)	81.5	35.7	48.3
24. Unit Forced Outage Rate	4.7	3.6	8.4
25. Forced Outage Hours	34.8	100.0	885.0

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* WASHINGTON NUCLEAR 2 *
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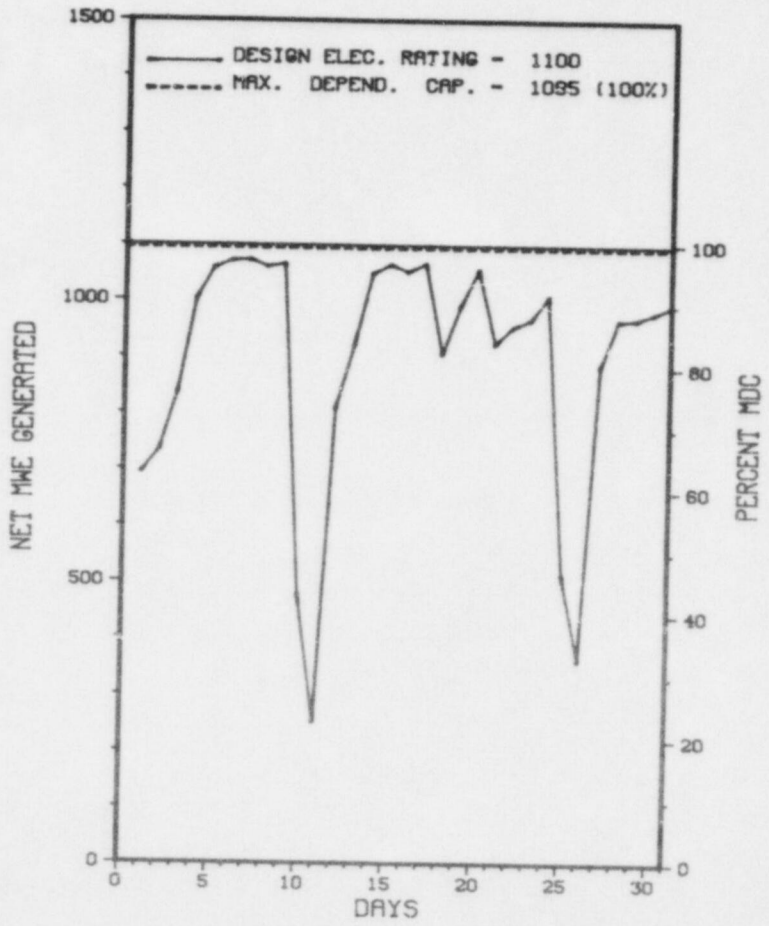
AVERAGE DAILY POWER LEVEL (MWe) PLOT

WASHINGTON NUCLEAR 2



JULY 1986

Report Period JUL 1986 U N I T S H U T D O W N S / R E D U C T I O N S

****
* WASHINGTON NUCLEAR 2 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-08	07/10/86	F	17.1	G	3	86-023	EC	INSTRU	REACTOR SCRAMED AT 100% POWER ON HIGH APRM NEUTRON FLUX, WHEN POWER TO MICROWAVE TELEMETRY EQUIPMENT WAS MOMENTARILY INTERRUPED WHILE TROUBLE SHOOTING A GROUND ON 125VDC DISTRIBUTION SYSTEM. AN ENGINEERING EVALUATION WILL BE MADE TO PROPERLY INDENTIFY CRITICAL POWER SUPPLIES.
86-09	07/25/86	F	17.7	A	3	86-025	HB	VALVEX	REACTOR SCRAMMED AT 72% POWER ON RPV HIGH PRESSURE, WHILE PERFORMING WEEKLY TURBINE VALVE TESTS. A BROKEN PIN TO LINKAGE ON A GOVERNOR VALVE CAUSED FALSE VALVE POSITION SIGNAL AND RESULTED IN FAILURE OF DEH SYSTEM TO CONTROL PRESSURE. THE PIN WAS REPLACED AND PLANT RETURNED TO SERVICE.

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* SUMMARY *
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WNP-2 EXPERIENCED 2 SHUTDOWNS IN JULY AS DISCUSSED ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* WASHINGTON NUCLEAR 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....WASHINGTON

COUNTY.....BENTON

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...12 MI. NW OF
RICHLAND, WASH.

TYPE OF REACTOR.....BWR

DATE INITIAL CRITICALITY...JANUARY 19, 1984

DATE ELEC ENER 1ST GENER...MAY 27, 1984

DATE COMMERCIAL OPERATE....DECEMBER 13, 1984

CONDENSER COOLING METHOD...COOLING TOWERS

CONDENSER COOLING WATER....MECHANICAL TOWERS

ELECTRIC RELIABILITY
COUNCIL.....WESTERN SYSTEMS
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....WASHINGTON PUBLIC POWER SUPPLY SYSTEM

CORPORATE ADDRESS.....P.O. BOX 968
RICHLAND, WASHINGTON 99352

CONTRACTOR
ARCHITECT/ENGINEER.....BURNS & ROE

NUC STEAM SYS SUPPLIER...GENERAL ELECTRIC

CONSTRUCTOR.....BECHTEL

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....V

IE RESIDENT INSPECTOR.....R. DODDS

LICENSING PROJ MANAGER.....J. BRADFUTE
DOCKET NUMBER.....50-397

LICENSE & DATE ISSUANCE....NPF-21, APRIL 13, 1984

PUBLIC DOCUMENT ROOM.....RICHLAND PUBLIC LIBRARY
SWIFT AND NORTHGATE STREETS
RICHLAND, WA 99352

INSPECTION STATUS

INSPECTION SUMMARY

+ INSPECTION ON JUNE 16-27, 1986 (REPORT NO. 50-397/86-11) AREAS INSPECTED: ANNUAL ANNOUNCED TEAM INSPECTION OF THE WNP-2 PLANT, FOCUSED ON MANAGEMENT CONTROLS (PROCEDURES, POLICIES, ADMINISTRATIVE ORDERS, ETC.) AND THE INVOLVEMENT OF MANAGEMENT IN THE IMPLEMENTATION OF THESE CONTROLS AS THEY APPLIED TO THE OPERATION AND MAINTENANCE OF WNP-2. THE FOLLOWING ACTIVITIES OF THE LICENSEE WERE EXAMINED: 1) OPERATOR RECOVERY ACTIONS; 2) QUALITY ASSURANCE PROGRAM; 3) MEASURING AND TEST EQUIPMENT; 4) TECHNICAL SPECIFICATION SURVEILLANCES AND TESTS; 5) CORRECTIVE AND PREVENTIVE MAINTENANCE PROGRAMS; 6) PLANT PROCEDURES; 7) PLANT MODIFICATIONS; 8) QUALITY ASSURANCE SURVEILLANCE; 9) SYSTEM WALKDOWN. TO THE MAXIMUM EXTENT FEASIBLE THE EFFECTIVENESS OF THESE ACTIVITIES WERE ASSESSED AS THEY APPLY TO THE FOLLOWING PLANT PHYSICAL SYSTEMS: 1) RESIDUAL HEAT REMOVAL SYSTEM (RHR); 2) STANDBY SERVICE WATER (SSW); 3) EMERGENCY POWER.

RESULTS: IN THE AREAS INSPECTED, SEVERAL VIOLATIONS OF NRC REQUIREMENTS WERE IDENTIFIED, SOME WITH MULTIPLE EXAMPLES. ENFORCEMENT ACTION AS A RESULT OF THIS INSPECTION WILL BE THE SUBJECT OF SEPARATE CORRESPONDENCE.

+ INSPECTION ON APRIL 28 - MAY 2, 1986 (REPORT NO. 50-397/86-12) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JUNE 9 - AUGUST 7, 1986 (REPORT NO. 50-397/86-19) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JUNE 1 - JULY 12, 1986 (REPORT NO. 50-397/86-21) AREAS INSPECTED: ROUTINE INSPECTION BY THE RESIDENT INSPECTORS OF AND A REGIONALLY BASED INSPECTOR OF CONTROL ROOM OPERATIONS, ENGINEERED SAFETY FEATURE (ESF) STATUS, SURVEILLANCE PROGRAM, MAINTENANCE PROGRAM, LICENSEE EVENT REPORTS, SPECIAL INSPECTION TOPICS, AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS.

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* WASHINGTON NUCLEAR 2 *
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INSPECTION SUMMARY

DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: TWO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED PERTAINING TO RADIATION CONTROLS AND TOOL AND EQUIPMENT ACCOUNTABILITY DURING THE REPAIR OF AN RHR SYSTEM VALVE.

+ INSPECTION ON JUNE 10 - JULY 10, 1986 (REPORT NO. 50-397/86-22) AREAS INSPECTED: UNANNOUNCED INSPECTION BY A REGIONALLY BASED INSPECTOR OF THE IMPLEMENTATION OF THE WNP-2 NUCLEAR POWER PLANT FIRE PROTECTION PROGRAM, QUALITY ASSURANCE PROGRAM, AND OTHER ITEMS IDENTIFIED DURING THE OUTAGE. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: OF THE AREAS EXAMINED, VIOLATIONS WERE IDENTIFIED IN THE FIRE PROTECTION AREA.

+ INSPECTION ON JUNE 25-27, 1986 (REPORT NO. 50-397/86-23) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION BY A REGIONALLY BASED INSPECTOR OF THE LICENSEE'S INTERNAL EXPOSURE CONTROL PROGRAM; RADIATION PROTECTION, CHEMISTRY, RADWASTE, AND ENVIRONMENTAL MONITORING ORGANIZATION, STAFFING, QUALIFICATIONS, AND TRAINING; FACILITY TOUR, AND A REVIEW OF THE RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM ANNUAL REPORT. DURING THIS INSPECTION, VARIOUS INSPECTION PROCEDURES WERE UTILIZED.

RESULTS: OF THE ELEVEN AREAS EXAMINED, ONE VIOLATION WAS IDENTIFIED IN ONE AREA: TECHNICAL SPECIFICATIONS, SECTION 6.11.1, ADHERENCE TO HEALTH PHYSICS PROCEDURES.

+ INSPECTION ON AUGUST 4-8, 1986 (REPORT NO. 50-397/86-24) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON JULY 14 - AUGUST 15, 1986 (REPORT NO. 50-397/86-25) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE PLANT HAS RESUMED OPERATION AT FULL POWER.

LAST IE SITE INSPECTION DATE: 07/14-08/15/86+

INSPECTION REPORT NO: 50-397/86-25+

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-11-L0	05-09-86	06-05-86	MOMENTARY LOSS OF RPS 'B' POWER RESULTS IN A HALF SCRAM AND NUCLEAR STEAM SUPPLY SHUTOFF SYSTEM ACTUATION
86-12-L0	05-12-86	06-11-86	RPS ACTUATION DURING RECIRCULATION PUMP START
86-13-L0	05-14-86	06-05-86	SPURIOUS NOISE IN NEUTRON MONITOR CHANNEL DUE TO WELDING RESULTS IN A SCRAM
86-14-L0	05-23-86	06-20-86	TYPE 'C' CONT LRT NOT PERFORMED ON TWO VALVES

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1. Docket: 50-382 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: GEORGE MILLER (504) 467-8211
4. Licensed Thermal Power (MWt): 3390
5. Nameplate Rating (Gross MWe): 1153
6. Design Electrical Rating (Net MWe): 1104
7. Maximum Dependable Capacity (Gross MWe): 1120
8. Maximum Dependable Capacity (Net MWe): 1104
9. If Changes Occur Above Since Last Report, Give Reasons:
MAXIMUM DEPENDABLE CAPACITIES HAVE BEEN REVISED.
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

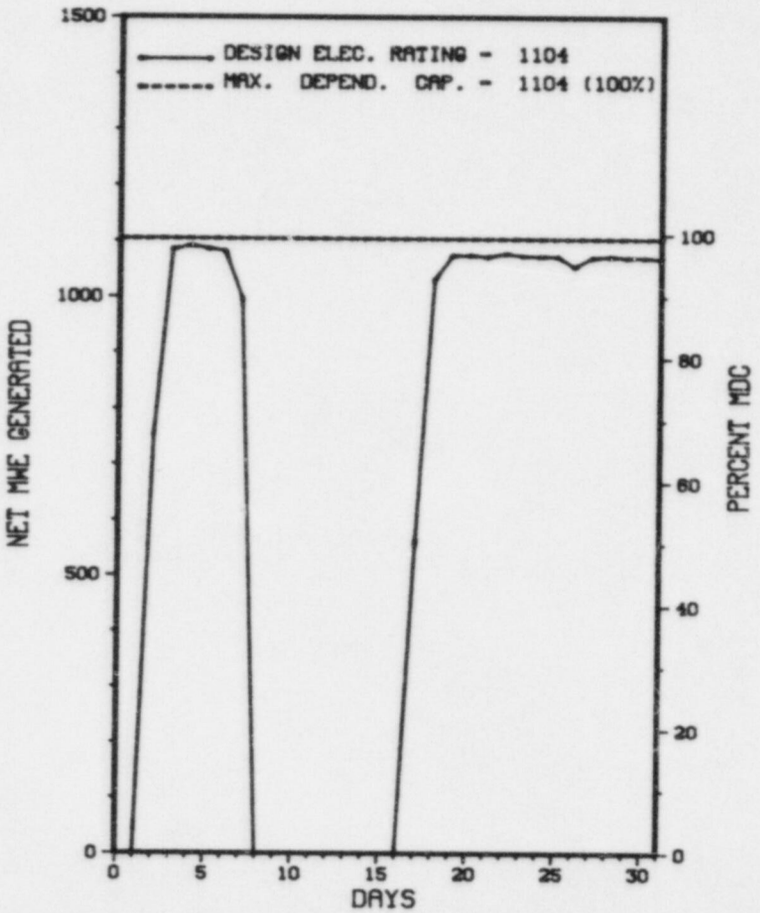
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	7,464.0
13. Hours Reactor Critical	527.4	4,249.3	6,118.0
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	511.9	4,196.8	5,998.6
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,646,157	13,809,476	19,451,915
18. Gross Elec Ener (MWH)	544,650	4,661,540	6,559,390
19. Net Elec Ener (MWH)	513,837	4,441,617	6,246,770
20. Unit Service Factor	68.8	82.5	80.4
21. Unit Avail Factor	68.8	82.5	80.4
22. Unit Cap Factor (MDC Net)	62.6	79.4	75.8
23. Unit Cap Factor (DER Net)	62.6	79.1	75.8
24. Unit Forced Outage Rate	31.2	7.4	13.0
25. Forced Outage Hours	232.1	333.7	898.5

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
REFUELING/MAINT.: NOVEMBER - DECEMBER 1986, 8 WEEKS.
27. If Currently Shutdown Estimated Startup Date: N/A

****
* WATERFORD 3 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

WATERFORD 3



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

WATERFORD 3

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-10	06/30/86	F	16.8	A	4	86-013	JC	CON	AT 100% POWER, A REACTOR TRIP OCCURRED ON LOW DNBR DUE TO A POSITION INDICATION FAILURE FOR CONTROL ELEMENT ASSEMBLY #35.
86-11	07/08/86	F	215.3	A	1		AB	SEAL	UNIT WAS MANUALLY SHUT DOWN TO REPLACE REACTOR COOLANT PUMP 2B SEAL.

SUMMARY

WATERFORD 3 HAD 2 OUTAGES IN JULY AS DISCUSSED ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* WATERFORD 3 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....LOUISIANA

COUNTY.....ST CHARLES

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...20 MI W OF
NEW ORLEANS, LA

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MARCH 4, 1985

DATE ELEC ENER 1ST GENER...MARCH 18, 1985

DATE COMMERCIAL OPERATE....SEPTEMBER 24, 1985

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....MISSISSIPPI RIVER

ELECTRIC RELIABILITY
COUNCIL.....SOUTHWEST POWER POOL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....LOUISIANA POWER & LIGHT

CORPORATE ADDRESS.....142 DELARONDE STREET
NEW ORLEANS, LOUISIANA 70174

CONTRACTOR
ARCHITECT/ENGINEER.....EBASCO

NUC STEAM SYS SUPPLIER...COMBUSTION ENGINEERING

CONSTRUCTOR.....EBASCO

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....J. LUEHMAN

LICENSING PROJ MANAGER.....J. WILSON
DOCKET NUMBER.....50-382

LICENSE & DATE ISSUANCE....NPF-38, MARCH 16, 1985

PUBLIC DOCUMENT ROOM.....HEAD LIBRARIAN
LOUISIANA COLLECTION
EARL K. LONG LIBRARY
UNIVERSITY OF NEW ORLEANS
LAKEFRONT DRIVE
NEW ORLEANS, LOUISIANA 70148

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION CONDUCTED APRIL 21-25, 1986 (86-10) ROUTINE, UNANNOUNCED INSPECTION OF SECURITY ORGANIZATION MANAGEMENT, SECURITY PROGRAM AUDIT, RECORDS AND REPORTS, MANAGEMENT EFFECTIVENESS-SECURITY, AND SECURITY PLAN CHANGES. WITHIN THE AREAS INSPECTED ONE APPARENT VIOLATION WAS IDENTIFIED.

INSPECTION CONDUCTED MAY 1-30, 1986 (86-11) ROUTINE, UNANNOUNCED INSPECTION OF PLANT STATUS; LICENSEE EVENT REPORT (LER) FOLLOWUP; MONTHLY SURVEILLANCE; ROUTINE INSPECTION; LICENSE CONDITIONS; LICENSEE'S RESPONSE TO SELECTED SAFETY ISSUES; LICENSEE FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS; REPORT OF FACILITY CHANGES, TESTS AND CALIBRATION CONTROL PROGRAM; AND ESF SYSTEM WALKDOWN. WITHIN THE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED. ONE RESOLVED ITEM WAS IDENTIFIED.

INSPECTION CONDUCTED JUNE 1-30, 1986 (86-13) ROUTINE, UNANNOUNCED INSPECTION OF: PLANT STATUS, LICENSEE EVENT REPORT (LER) FOLLOWUP, MONTHLY SURVELLIANCE, ROUTINE INSPECTION, LICENSEE FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS, MONTHLY MAINTENANCE, POTENTIAL GENERIC PROBLEMS, LICENSE CONDITIONS, AND IE BULLENTINS. WITHIN THE AREAS INSPECTED, TWO VIOLATIONS AND ONE DEVIATION WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

LAST IE SITE INSPECTION DATE: JUNE 1-30, 1986

INSPECTION REPORT NO: 50-382/86-13

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-008 (LER)	5/04/86	5/29/86	OPERATOR ERROR DURING SURVEILLANCE TESTING RESULTED IN INADVERTENT ACTUATION OF EMERGENCY FEEDWATER SYSTEM
86-009 (LER)	5/09/86	6/09/86	REED SWITCH FAILURE RESULTS IN REACTOR TRIP ON LOW DEPARTURE FROM NUCLEATE BOILING RATIO
86-010 (LER)	6/02/86	7/01/86	IMPROPERLY FIELD SCHEDULING CARDS RESULTS IN EXCEEDING THE SURVEILLANCE FREQUENCY FOR SECONDARY ACTIVITY SAMPLE
86-011	6/05/86	7/07/86	WALKDOWN AS A RESULT OF PART 21 REPORT IDENTIFIED TWO MISSING INTERNAL PENETRATION SEALS IN ADDITION TO SEVERAL DEFICIENT SEAL ARRANGEMENTS
86-012	6/25/86	7/22/86	LACK OF INTER-DEPARTMENTAL COMMUNICATIONS RESULTED IN DEFICIENT FIRE DOOR SURVEILLANCE

1. Docket: 50-482 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: M. WILLIAMS (316) 364-8831
4. Licensed Thermal Power (MWt): 3411
5. Nameplate Rating (Gross MWe): 1250
6. Design Electrical Rating (Net MWe): 1170
7. Maximum Dependable Capacity (Gross MWe): 1170
8. Maximum Dependable Capacity (Net MWe): 1128
9. If Changes Occur Above Since Last Report, Give Reasons:

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	7,966.7
13. Hours Reactor Critical	683.7	4,389.6	7,179.9
14. Rx Reserve Shtdwn Hrs	51.1	92.5	171.2
15. Hrs Generator On-Line	664.8	4,352.4	7,124.0
16. Unit Reserve Shtdwn Hrs	.0	.0	19.0
17. Gross Therm Ener (MWH)	2,210,358	14,154,316	23,029,249
18. Gross Elec Ener (MWH)	762,270	4,946,805	8,017,895
19. Net Elec Ener (MWH)	728,383	4,727,486	7,669,586
20. Unit Service Factor	89.4	85.6	89.4
21. Unit Avail Factor	89.4	85.6	89.7
22. Unit Cap Factor (MDC Net)	86.8	82.4	55.0*
23. Unit Cap Factor (DER Net)	83.7	79.4	82.3
24. Unit Forced Outage Rate	1.9	5.8	5.0
25. Forced Outage Hours	12.7	269.1	377.2

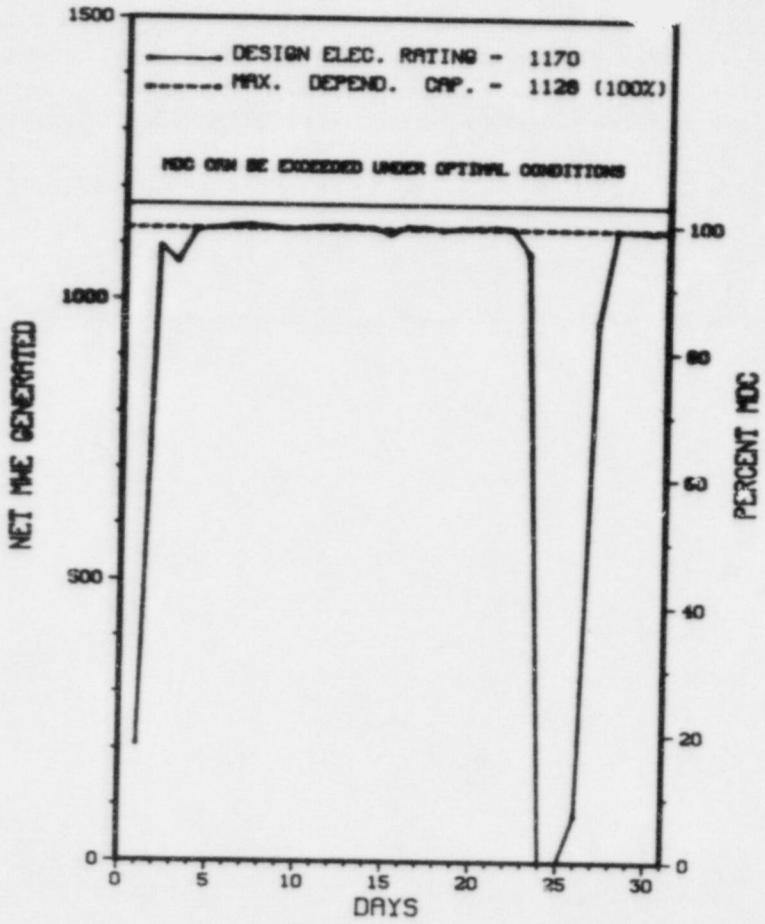
26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
REFUELING: 10/09/86 - 55 DAYS.

27. If Currently Shutdown Estimated Startup Date: N/A

****
* WOLF CREEK 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

WOLF CREEK 1



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

U N I T S H U T D O W N S / R E D U C T I O N S

****
* WOLF CREEK 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
7	07/01/86	F	6.0	B	3				EXTENSION ON JUNE UNIT TRIP CAUSED BY LOW-LOW CONDENSER HOTWELL LEVEL SIGNAL DURING TESTING.
8	07/01/86	F	6.7	G	3				UNIT TRIP, REACTOR TRIP DUE TO LOW-LOW STEAM GENERATOR LEVEL IN MANUAL CONTROL.
9	07/24/86	S	66.5	B	3				UNIT TRIP, REACTOR TRIP DUE TO LOW-LOW STEAM GENERATOR LEVEL IN MANUAL CONTROL WHILE TAKING UNIT OFF-LINE TO REPAIR A STATOR COOLING WATER LEAK.

****
* SUMMARY *
****

WOLF CREEK INCURRED 3 SHUTDOWNS IN JULY AS DISCUSSED ABOVE.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* WOLF CREEK 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....KANSAS

COUNTY.....COFFEY

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...3.5 MI NE OF
BURLINGTON, KAN

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...MAY 22, 1985

DATE ELEC ENER 1ST GENER...JUNE 12, 1985

DATE COMMERCIAL OPERATE....SEPTEMBER 3, 1985

CONDENSER COOLING METHOD...COOLING LAKE

CONDENSER COOLING WATER....COOLING LAKE

ELECTRIC RELIABILITY
COUNCIL.....SOUTHWEST POWER POOL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....KANSAS GAS & ELECTRIC

CORPORATE ADDRESS.....P.O. BOX 208
WICHITA, KANSAS 67201

CONTRACTOR
ARCHITECT/ENGINEER.....BECHTEL

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....DANIEL INTERNATIONAL

TURBINE SUPPLIER.....GENERAL ELECTRIC

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....IV

IE RESIDENT INSPECTOR.....J. CUMMINS

LICENSING PROJ MANAGER.....P. OCONNOR
DOCKET NUMBER.....50-482

LICENSE & DATE ISSUANCE....NPF-42, JUNE 4, 1985

PUBLIC DOCUMENT ROOM.....WILLIAM ALLAN WHITE LIBRARY
GOVERNMENT DOCUMENTS DIVISION
EMPORIA STATE UNIVERSITY
1200 COMMERCIAL STREET
EMPORIA, KANSAS 66801

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION CONDUCTED APRIL 1-30, 1986 (86-08) ROUTINE, UNANNOUNCED INSPECTION INCLUDING FOLLOWUP ON A PREVIOUSLY IDENTIFIED NRC ITEM; OPERATIONAL SAFETY VERIFICATION; ENGINEERED SAFETY FEATURES SYSTEM WALKDOWN; ONSITE FOLLOWUP OF EVENTS; MONTHLY SURVEILLANCE OBSERVATION; MONTHLY MAINTENANCE OBSERVATION; AND PLANT TOURS. WITHIN THE SEVEN AREAS INSPECTED, THREE VIOLATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED MAY 5-9, 1986 (86-12) ROUTINE, UNANNOUNCED INSPECTION OF THE PHYSICAL SECURITY PLAN (PSP) AND IMPLEMENTING PROCEDURES, MANAGEMENT EFFECTIVENESS, SECURITY ORGANIZATION MANAGEMENT, SECURITY PROGRAM AUDIT, AND RECORDS AND REPORTS. WITHIN THE AREAS INSPECTED, ONE APPARENT VIOLATION WAS IDENTIFIED.

INSPECTION CONDUCTED MAY 1-31, 1986 (86-13) ROUTINE UNANNOUNCED INSPECTION INCLUDING PLANT STATUS; FOLLOWUP ON PREVIOUSLY IDENTIFIED NRC ITEMS; OPERTIONAL SAFETY VERIFICATION MONTHLY SURVEILLANCE OBSERVATION, MONTHLY MAINTENANCE OBSERVATION; ENVIRONMENTAL QUALIFICATION OF ELECTRIC EQUIPMENT; AND SURVEY OF BIOFOULING DETECTION INSTRUMENTATION ON COOLING WATER HEAT EXCHANGERS. WITHIN THE SEVEN AREAS INSPECTED, THREE VIOLATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED MAY 12-16, 1986 (86-14) SPECIAL, UNNOUNCED INSPECTION TO DETERMINE WHAT WIRING IS INSTALLED IN LIMITORQUE OPERATORS AND TO REVIEW LICENSEE'S ENVIRONMENTAL DOCUMENTATION TO ENSURE THAT QUALIFICATION OF WIRING IS ADEQUATELY ESTABLISHED. THE INSPECTION ALSO INCLUDED A REVIEW OF THE LICENSEE'S ACTIONS RELATIVE TO INFORMATION NOTICES.

INSPECTION SUMMARY

INSPECTION CONDUCTED JUNE 1 - JULY 5, 1986 (86-16) ROUTINE, UNANNOUNCED INSPECTION INCLUDING PLANT STATUS; FOLLOWUP ON PREVIOUSLY IDENTIFIED NRC ITEMS; OPERATIONAL SAFETY VERIFICATION; ENGINEERED SAFETY FEATURES SYSTEM WALKDOWN; ONSITE FOLLOWUP OF EVENTS; MONTHLY SURVEILLANCE OBSERVATION; MONTHLY MAINTENANCE OBSERVATION; PLANT SAFETY REVIEW COMMITTEE; AND NUCLEAR SAFETY REVIEW COMMITTEE. WITHIN THE NINE AREAS INSPECTED, ONE VIOLATION WAS IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

PLANT STATUS:

NONE

LAST IE SITE INSPECTION DATE: JULY 5, 1986

INSPECTION REPORT NO: 50-482/86-16

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

1. Docket: 50-029 O P E R A T I N G S T A T U S
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: S. WHIPPLE (617) 872-8100
4. Licensed Thermal Power (MWt): 600
5. Nameplate Rating (Gross MWe): 185 X 1.0 = 185
6. Design Electrical Rating (Net MWe): 175
7. Maximum Dependable Capacity (Gross MWe): 180
8. Maximum Dependable Capacity (Net MWe): 167
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

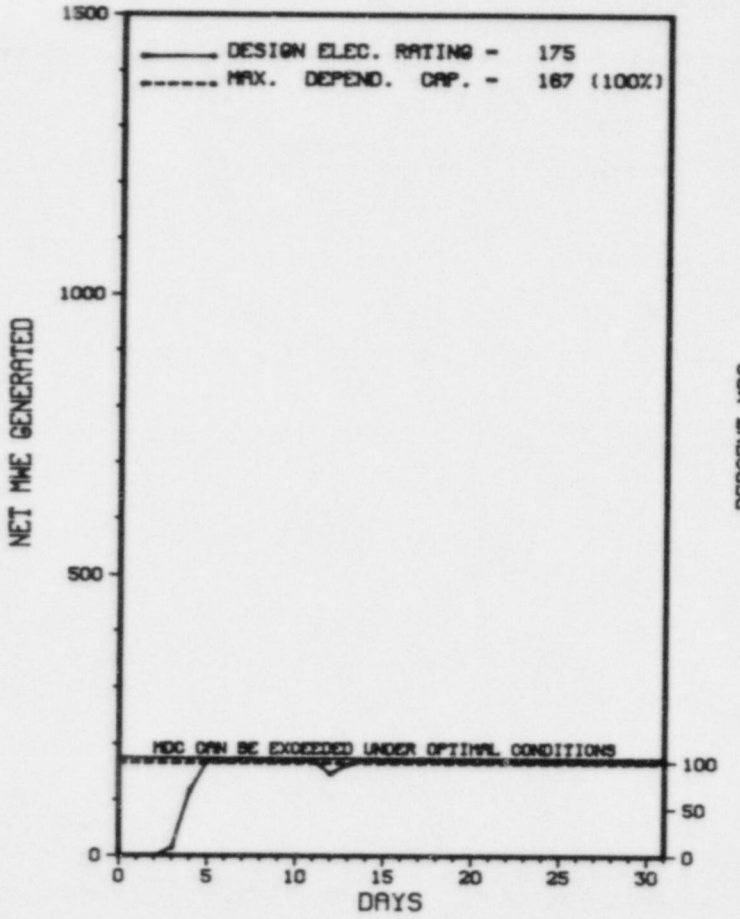
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	225,332.0
13. Hours Reactor Critical	685.4	4,701.0	180,222.2
14. Rx Reserve Shtdwn Hrs	.0	.0	.0
15. Hrs Generator On-Line	679.6	4,690.8	175,349.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	396,445	2,759,025	95,439,642
18. Gross Elec Ener (MWH)	120,514	840,493	28,923,166
19. Net Elec Ener (MWH)	112,886	787,388	27,063,605
20. Unit Service Factor	91.3	92.2	77.8
21. Unit Avail Factor	91.3	92.2	77.8
22. Unit Cap Factor (MDC Net)	90.9	92.7	73.8*
23. Unit Cap Factor (DER Net)	86.7	88.4	70.3*
24. Unit Forced Outage Rate	8.7	7.8	5.3
25. Forced Outage Hours	64.4	396.2	8,722.3

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
NONE
27. If Currently Shutdown Estimated Startup Date: N/A

****
* YANKEE-ROWE 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

YANKEE-ROWE 1



JULY 1986

* Item calculated with a Weighted Average

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

****
* YANKEE-ROWE 1 *
****

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
86-3	06/18/86	F	64.4	A	4				CONTINUATION OF SHUTDOWN TO REPAIR S/G BLOWDOWN LINE.

****
* SUMMARY *
****

YANKEE ROWE RETURNED TO POWER ON JULY 3 AFTER STEAM GENERATOR BLOWDOWN LINE REPAIR.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* YANKEE-ROWE 1 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....MASSACHUSETTS

COUNTY.....FRANKLIN

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...25 MI NE OF
PITTSFIELD, MASS

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...AUGUST 19, 1960

DATE ELEC ENER 1ST GENER...NOVEMBER 10, 1960

DATE COMMERCIAL OPERATE....JULY 1, 1961

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....DEERFIELD RIVER

ELECTRIC RELIABILITY
COUNCIL.....NORTHEAST POWER
COORDINATING COUNCIL

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....YANKEE ATOMIC ELECTRIC

CORPORATE ADDRESS.....1671 WORCESTER RD.
FRAMINGHAM, MASSACHUSETTS 01701

CONTRACTOR
ARCHITECT/ENGINEER.....STONE & WEBSTER

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....STONE & WEBSTER

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....I

IE RESIDENT INSPECTOR.....H. EICHENHOLZ

LICENSING PROJ MANAGER.....E. MCKENNA
DOCKET NUMBER.....50-029

LICENSE & DATE ISSUANCE....DPR-3, DECEMBER 24, 1963

PUBLIC DOCUMENT ROOM.....GREENFIELD COMMUNITY COLLEGE
1 COLLEGE DRIVE
GREENFIELD, MASSACHUSETTS 01301

INSPECTION STATUS

INSPECTION SUMMARY

NO INSPECTION INPUT PROVIDED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENTS:

NO INPUT PROVIDED.

FACILITY ITEMS (PLANS AND PROCEDURES):

NO INPUT PROVIDED.

OTHER ITEMS

MANAGERIAL ITEMS:

NO INPUT PROVIDED.

PLANT STATUS:

NO INPUT PROVIDED.

LAST IE SITE INSPECTION DATE: NO INPUT PROVIDED.

INSPECTION REPORT NO: NO INPUT PROVIDED.

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NO INPUT PROVIDED.			

1. Docket: 50-295 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: GERRI AUSTIN (312) 746-2084
4. Licensed Thermal Power (MWt): 3250
5. Nameplate Rating (Gross MWe): 1220 X .9 = 1098
6. Design Electrical Rating (Net MWe): 1040
7. Maximum Dependable Capacity (Gross MWe): 1085
8. Maximum Dependable Capacity (Net MWe): 1040
9. If Changes Occur Above Since Last Report, Give Reasons:
NONE
10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:
NONE

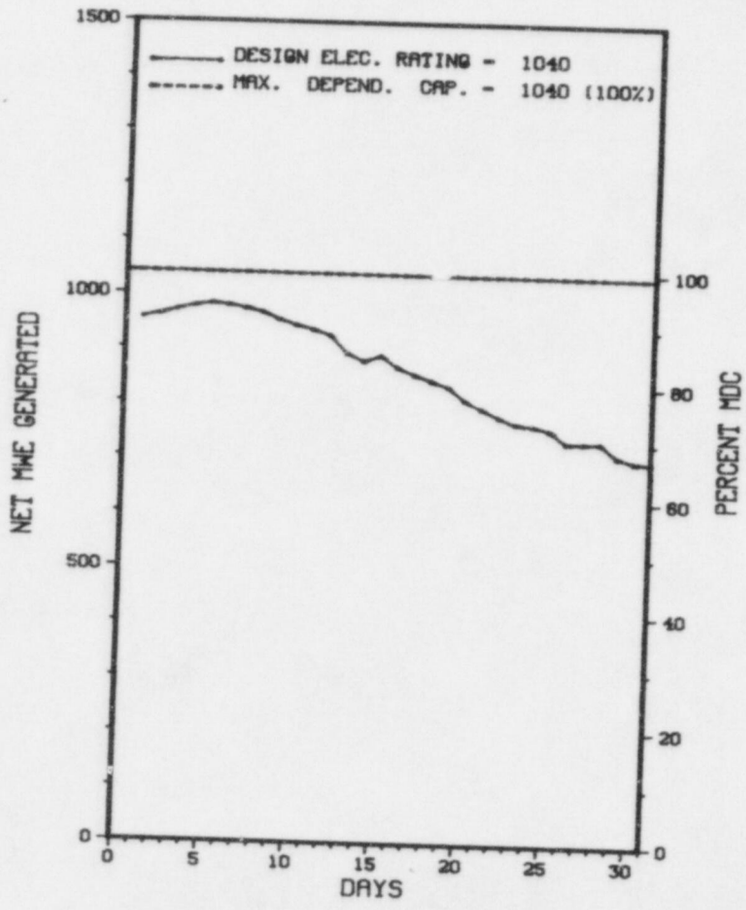
	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	110,303.0
13. Hours Reactor Critical	744.0	4,674.1	78,391.2
14. Rx Reserve Shtdwn Hrs	.0	.0	2,621.8
15. Hrs Generator On-Line	744.0	4,635.1	76,241.2
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	2,083,198	14,240,662	220,874,965
18. Gross Elec Ener (MWH)	670,651	4,666,925	70,385,723
19. Net Elec Ener (MWH)	639,016	4,465,935	66,875,279
20. Unit Service Factor	100.0	91.1	69.1
21. Unit Avail Factor	100.0	91.1	69.1
22. Unit Cap Factor (MDC Net)	82.6	84.4	58.3
23. Unit Cap Factor (DER Net)	82.6	84.4	58.3
24. Unit Forced Outage Rate	.0	8.9	13.9
25. Forced Outage Hours	.0	451.9	11,668.2

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):
REFUELING OUTAGE: 09/03/86
27. If Currently Shutdown Estimated Startup Date: N/A

****
* ZION 1 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ZION 1



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

ZION 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

ZION 1 OPERATED ROUTINELY DURING JULY.

Type	Reason	Method	System & Component
F-Forced S-Sched	A-Equip Failure B-Maint or Test C-Refueling D-Regulatory Restriction E-Operator Training & License Examination F-Admin G-Oper Error H-Other	1-Manual 2-Manual Scram 3-Auto Scram 4-Continued 5-Reduced Load 9-Other	Exhibit F & H Instructions for Preparation of Data Entry Sheet Licensee Event Report (LER) File (NUREG-0161)

INSPECTION SUMMARY

IN THREE AREAS; ONE VIOLATION WAS IDENTIFIED IN THE TWO REMAINING AREAS (INADEQUATE TEST CONTROL)

INSPECTION ON JULY 2 (86015; 86014): ROUTINE UNANNOUNCED INSPECTION OF THE RESOLUTION OF AN IE BULLETIN. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY

LAST IE SITE INSPECTION DATE: 01/01/86

INSPECTION REPORT NO: 86020

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-25	06/16/86	07/16/86	CONTAINMENT SPRAY ADDITIVE TANK BELOW MINIMUM LEVEL
86-27	07/08/86	08/07/86	INOPERABLE FIRE BARRIER

1. Docket: 50-304 OPERATING STATUS
2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0
3. Utility Contact: GERRI AUSTIN (312) 746-2084
4. Licensed Thermal Power (MWt): 3250
5. Nameplate Rating (Gross MWe): 1220 X 0.9 = 1098
6. Design Electrical Rating (Net MWe): 1040
7. Maximum Dependable Capacity (Gross MWe): 1085
8. Maximum Dependable Capacity (Net MWe): 1040
9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):
11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	104,016.0
13. Hours Reactor Critical	501.7	4,110.5	75,529.9
14. Rx Reserve Shtdwn Hrs	.0	.0	226.1
15. Hrs Generator On-Line	411.1	3,764.9	73,372.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,292,922	11,411,157	217,080,662
18. Gross Elec Ener (MWH)	429,513	3,818,651	68,138,246
19. Net Elec Ener (MWH)	406,513	3,640,446	64,817,888
20. Unit Service Factor	55.3	74.0	70.5
21. Unit Avail Factor	55.3	74.0	70.5
22. Unit Cap Factor (MDC Net)	52.5	68.8	59.9
23. Unit Cap Factor (DER Net)	52.5	68.8	59.9
24. Unit Forced Outage Rate	44.7	10.8	15.6
25. Forced Outage Hours	332.9	458.0	13,596.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

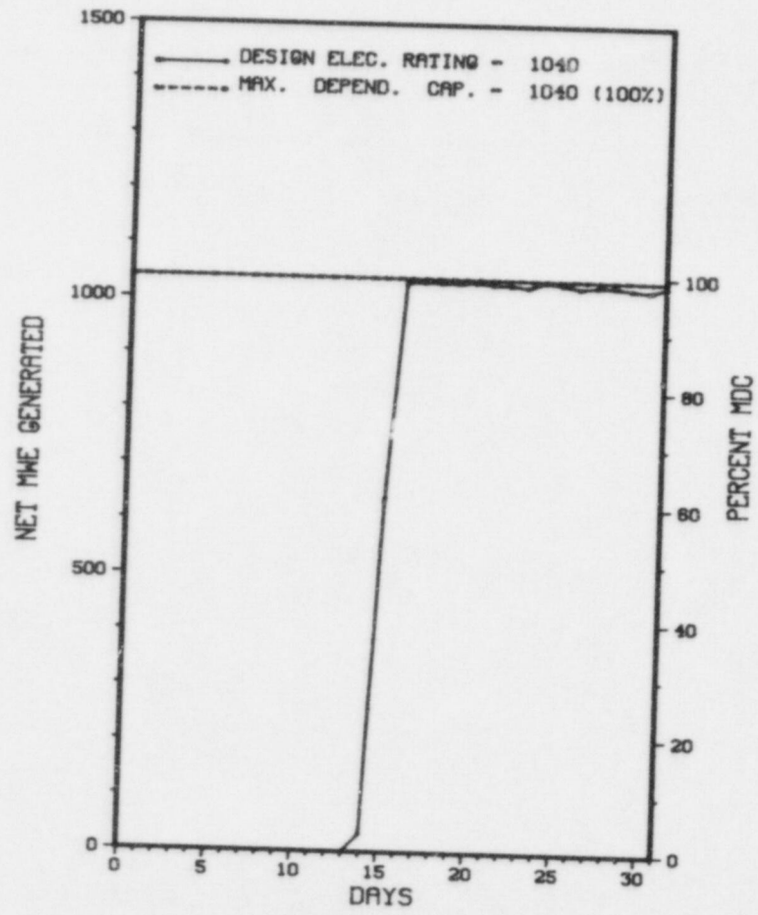
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* ZION 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ZION 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

ZION 1

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence

NONE

SUMMARY

ZION 1 OPERATED ROUTINELY DURING JULY.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	Licensee Event Report
			9-Other	(LER) File (NUREG-0161)

ZION 1

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....LAKE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...40 MI N OF
CHICAGO, ILL

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...JUNE 19, 1973

DATE ELEC ENER 1ST GENER...JUNE 28, 1973

DATE COMMERCIAL OPERATE....DECEMBER 31, 1973

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....COMMONWEALTH EDISON

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....M. HOLZMER

LICENSING PROJ MANAGER.....T. NORRIS
DOCKET NUMBER.....50-295

LICENSE & DATE ISSUANCE....DPR-39, OCTOBER 19, 1973

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ZION, ILLINOIS 60099

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON MARCH 12-13 (86008): ROUTINE, ANNOUNCED INSPECTION BY ONE REGIONAL INSPECTOR OF PLANT TRIPS - SAFETY SYSTEM CHALLENGES. IT WAS CONDUCTED IN ACCORDANCE WITH NRC INSPECTION PROCEDURE NO. 93702. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON APRIL 14 THROUGH JUNE 13 (86011; 86010): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; UNIT 1 TRIP ON LOSS OF EHC; LOSS OF ENGINEERED SAFETY FEATURES LOGIC DUE TO PULLED FUSES; FAILURE OF THE 1B MAIN STEAM CHECK VALVE; UNIT 1 UNUSUAL EVENT DUE TO EXCESSIVE REACTOR COOLANT SYSTEM LEAKAGE; UNIT 2 TRIP ON LOSS OF THE 2B MAIN FEEDWATER PUMP; MISSED SURVEILLANCE OF CONTROL ROOM MAKEUP CHARCOAL FILTER; OPERATIONAL SAFETY AND ENGINEERED SAFETY FEATURE (ESF) SYSTEM WALKDOWN; SURVEILLANCE; MAINTENANCE; LICENSEE EVENT REPORTS (LERS); TRAINING; IE BULLETIN FOLLOWUP; AND REGION III REQUESTS. IN ADDITION, THIS REPORT DOCUMENTS THE APRIL 29, 1986, MEETING BETWEEN THE LICENSEE SITE AND CORPORATE STAFFS AND THE NRC REGION III STAFF REGARDING OPERATION WITH INCORRECT NEGATIVE FLUX RATE REACTOR TRIP SETPOINTS AND REGARDING THE DECEMBER 14, 1985, LOSS OF RESIDUAL HEAT REMOVAL EVENT. OF THE 15 AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN 13 AREAS, AND TWO VIOLATIONS WERE IDENTIFIED IN THE REMAINING TWO AREAS (FAILURE TO FOLLOW OPERATING PROCEDURES AND FAILURE TO CONDUCT A TECHNICAL SPECIFICATION REQUIRED SURVEILLANCE ON THE CONTROL ROOM VENTILATION CHARCOAL FILTER).

INSPECTION ON MAY 5 THROUGH JUNE 5 AND JUNE 11 (86012; 86011): ROUTINE, ANNOUNCED INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS (92701), ERROR IN ZION UNITS 1 AND 2 TECHNICAL SPECIFICATION F Q NORMALIZED OPERATIONS ENVELOPE (92701), ISOTHERMAL/MODERATOR TEMPERATURE COEFFICIENT MEASUREMENTS (61708), CONTROL ROD WORTH MEASUREMENTS (61710), AND SHUTDOWN MARGIN/ESTIMATED CRITICAL CONDITION CALCULATION (61707). OF THE FIVE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED

INSPECTION SUMMARY

IN THREE AREAS; ONE VIOLATION WAS IDENTIFIED IN THE TWO REMAINING AREAS (INADEQUATE TEST CONTROL)

INSPECTION ON JULY 2 (86015; 86014): ROUTINE UNANNOUNCED INSPECTION OF THE RESOLUTION OF AN IE BULLETIN. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

ENFORCEMENT SUMMARY

NONE

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY

LAST IE SITE INSPECTION DATE: 01/01/86

INSPECTION REPORT NO: 86020

REPORTS FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-25	06/16/86	07/16/86	CONTAINMENT SPRAY ADDITIVE TANK BELOW MINIMUM LEVEL
86-27	07/08/86	08/07/86	INOPERABLE FIRE BARRIER

1. Docket: 50-304 OPERATING STATUS

2. Reporting Period: 07/01/86 Outage + On-line Hrs: 744.0

3. Utility Contact: GERRI AUSTIN (312) 746-2084

4. Licensed Thermal Power (MWt): 3250

5. Nameplate Rating (Gross MWe): 1220 X 0.9 = 1098

6. Design Electrical Rating (Net MWe): 1040

7. Maximum Dependable Capacity (Gross MWe): 1085

8. Maximum Dependable Capacity (Net MWe): 1040

9. If Changes Occur Above Since Last Report, Give Reasons:

NONE

10. Power Level To Which Restricted, If Any (Net MWe):

11. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
12. Report Period Hrs	744.0	5,087.0	104,016.0
13. Hours Reactor Critical	501.7	4,110.5	75,529.9
14. Rx Reserve Shtdwn Hrs	.0	.0	226.1
15. Hrs Generator On-Line	411.1	3,764.9	73,372.7
16. Unit Reserve Shtdwn Hrs	.0	.0	.0
17. Gross Therm Ener (MWH)	1,292,922	11,411,157	217,080,662
18. Gross Elec Ener (MWH)	429,513	3,818,651	68,138,246
19. Net Elec Ener (MWH)	406,513	3,640,446	64,817,888
20. Unit Service Factor	55.3	74.0	70.5
21. Unit Avail Factor	55.3	74.0	70.5
22. Unit Cap Factor (MDC Net)	52.5	68.8	59.9
23. Unit Cap Factor (DER Net)	52.5	68.8	59.9
24. Unit Forced Outage Rate	44.7	10.8	15.6
25. Forced Outage Hours	332.9	458.0	13,596.1

26. Shutdowns Sched Over Next 6 Months (Type,Date,Duration):

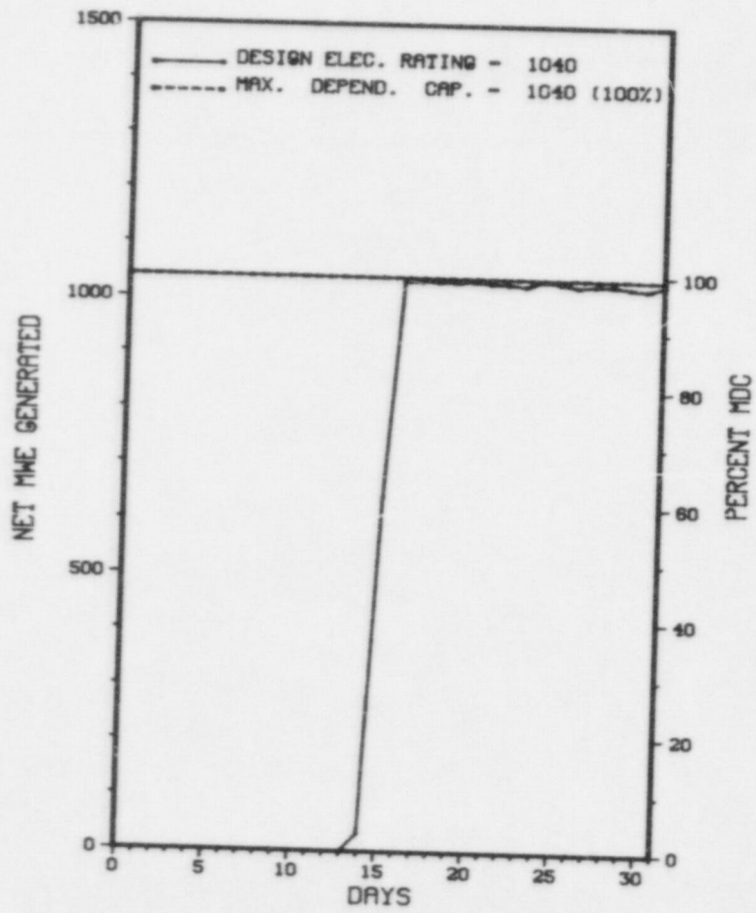
NONE

27. If Currently Shutdown Estimated Startup Date: N/A

****
* ZION 2 *
****

AVERAGE DAILY POWER LEVEL (MWe) PLOT

ZION 2



JULY 1986

Report Period JUL 1986

UNIT SHUTDOWNS / REDUCTIONS

ZION 2

No.	Date	Type	Hours	Reason	Method	LER Number	System	Component	Cause & Corrective Action to Prevent Recurrence
4	06/27/86	F	332.9	A	4				CONTINUATION REACTOR TRIP DUE TO HIGH OVER-TEMPERATURE DELTA "T" ON LOOPS B&D.

SUMMARY

ZION 2 RETURNED ONLINE FROM REPAIRS ON JULY 14TH AND OPERATED ROUTINELY THE REMAINDER OF THE MONTH.

Type	Reason		Method	System & Component
F-Forced	A-Equip Failure	F-Admin	1-Manual	Exhibit F & H
S-Sched	B-Maint or Test	G-Oper Error	2-Manual Scram	Instructions for
	C-Refueling	H-Other	3-Auto Scram	Preparation of
	D-Regulatory Restriction		4-Continued	Data Entry Sheet
	E-Operator Training & License Examination		5-Reduced Load	licensee Event Report
			9-Other	(LER) File (NUREG-0161)

****
* ZION 2 *
****

FACILITY DATA

Report Period JUL 1986

FACILITY DESCRIPTION

LOCATION
STATE.....ILLINOIS

COUNTY.....LAKE

DIST AND DIRECTION FROM
NEAREST POPULATION CTR...40 MI N OF
CHICAGO, ILL

TYPE OF REACTOR.....PWR

DATE INITIAL CRITICALITY...DECEMBER 24, 1973

DATE ELEC ENER 1ST GENER...DECEMBER 26, 1973

DATE COMMERCIAL OPERATE....SEPTEMBER 17, 1974

CONDENSER COOLING METHOD...ONCE THRU

CONDENSER COOLING WATER....LAKE MICHIGAN

ELECTRIC RELIABILITY
COUNCIL.....MID-AMERICA
INTERPOOL NETWORK

UTILITY & CONTRACTOR INFORMATION

UTILITY
LICENSEE.....COMMONWEALTH EDISON

CORPORATE ADDRESS.....P.O. BOX 767
CHICAGO, ILLINOIS 60690

CONTRACTOR
ARCHITECT/ENGINEER.....SARGENT & LUNDY

NUC STEAM SYS SUPPLIER...WESTINGHOUSE

CONSTRUCTOR.....COMMONWEALTH EDISON

TURBINE SUPPLIER.....WESTINGHOUSE

REGULATORY INFORMATION

IE REGION RESPONSIBLE.....III

IE RESIDENT INSPECTOR.....M. HOLZMER

LICENSING PROJ MANAGER.....J. NORRIS
DOCKET NUMBER.....50-304

LICENSE & DATE ISSUANCE....DPR-48, NOVEMBER 14, 1973

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ZION, ILLINOIS 60099

INSPECTION STATUS

INSPECTION SUMMARY

INSPECTION ON APRIL 14 THROUGH JUNE 13 (86011; 86010): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; UNIT 1 TRIP ON LOSS OF EHC; LOSS OF ENGINEERED SAFETY FEATURES LOGIC DUE TO PULLED FUSES; FAILURE OF THE 1B MAIN STEAM CHECK VALVE; UNIT 1 UNUSUAL EVENT DUE TO EXCESSIVE REACTOR COOLANT SYSTEM LEAKAGE; UNIT 2 TRIP ON LOSS OF THE 2B MAIN FEEDWATER PUMP; MISSED SURVEILLANCE OF CONTROL ROOM MAKEUP CHARCOAL FILTER; OPERATIONAL SAFETY AND ENGINEERED SAFETY FEATURE (ESF) SYSTEM WALKDOWN; SURVEILLANCE; MAINTENANCE; LICENSEE EVENT REPORTS (LERS); TRAINING; IE BULLETIN FOLLOWUP; AND REGION III REQUESTS. IN ADDITION, THIS REPORT DOCUMENTS THE APRIL 29, 1986, MEETING BETWEEN THE LICENSEE SITE AND CORPORATE STAFFS AND THE NRC REGION III STAFF REGARDING OPERATION WITH INCORRECT NEGATIVE FLUX RATE REACTOR TRIP SETPOINTS AND REGARDING THE DECEMBER 14, 1985, LOSS OF RESIDUAL HEAT REMOVAL EVENT. OF THE 15 AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN 13 AREAS, AND TWO VIOLATIONS WERE IDENTIFIED IN THE REMAINING TWO AREAS (FAILURE TO FOLLOW OPERATING PROCEDURES AND FAILURE TO CONDUCT A TECHNICAL SPECIFICATION REQUIRED SURVEILLANCE ON THE CONTROL ROOM VENTILATION CHARCOAL FILTER).

INSPECTION ON MAY 5 THROUGH JUNE 5 AND JUNE 11 (86012; 86011): ROUTINE, ANNOUNCED INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS (92701), ERROR IN ZION UNITS 1 AND 2 TECHNICAL SPECIFICATION F Q NORMALIZED OPERATIONS ENVELOPE (92701), ISOTHERMAL/MODERATOR TEMPERATURE COEFFICIENT MEASUREMENTS (61708), CONTROL ROD WORTH MEASUREMENTS (61710), AND SHUTDOWN MARGIN/ESTIMATED CRITICAL CONDITION CALCULATION (61707). OF THE FIVE AREAS INSPECTED, NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE VIOLATION WAS IDENTIFIED IN THE TWO REMAINING AREAS (INADEQUATE TEST CONTROL)

INSPECTION ON JULY 2 (86015; 86014): ROUTINE UNANNOUNCED INSPECTION OF THE RESOLUTION OF AN IE BULLETIN. NO VIOLATIONS OR DEVIATIONS WERE IDENTIFIED.

****
* ZION 2 *
****

INSPECTION SUMMARY

ENFORCEMENT SUMMARY

10 CFR 50, APPENDIX B, CRITERION XI AS IMPLEMENTED BY THE COMMONWEALTH EDISON CORPORATE QUALITY ASSURANCE MANUAL, REQUIRES THAT TESTING BE PERFORMED IN ACCORDANCE WITH WRITTEN TEST PROCEDURES AND THAT TEST RESULTS BE DOCUMENTED AND EVALUATED TO ASSURE THAT TEST REQUIREMENTS HAVE BEEN SATISFIED. CONTRARY TO THE ABOVE: (A) THE LICENSEE PERFORMED AN INADEQUATE RESULTS EVALUATION FOR TECHNICAL STAFF SURVEILLANCE PROCEDURE TSS 15.6.51, "ZERO POWER PHYSICS MEASUREMENTS FOLLOWING REFUELING," PERFORMED FOR UNIT 2 CYCLE 9 ON JANUARY 27-29, 1986, IN THAT THE INDEPENDENT REVIEW OF THE CALCULATION OF THE CRITICAL OPERATIONS RESTRICTIONS TO ASSURE A NEGATIVE MODERATOR COEFFICIENT FAILED TO DETECT THAT THERE WERE TWO ERRORS PRESENT IN THE CALCULATION. THE FAILURE TO IDENTIFY AND CORRECT THE ERRORS HAD MINOR SAFETY SIGNIFICANCE; (B) TESTING WAS NOT PERFORMED IN ACCORDANCE WITH WRITTEN TEST PROCEDURES IN THAT: (1) PROCEDURE STEPS WERE SKIPPED (NOT SIGNED NOR PERFORMED BEFORE PROCEEDING TO THE NEXT STEP) DURING THE PERFORMANCE OF THE FOLLOWING TEST PROCEDURE: TSS 15.6.51, "ZERO POWER PHYSICS MEASUREMENTS FOLLOWING REFUELING," PERFORMED FOR UNIT 2 CYCLE 9 ON JANUARY 27-29, 1986: STEPS 7.7.B AND 7.9.B. (2) PROCEDURE STEPS AT THE END OF SURVEILLANCE PROCEDURES WERE NOT SIGNED OFF PRIOR TO PROCEEDING TO AN OPERATIONAL PHASE FOR WHICH THE SURVEILLANCES WERE REQUIRED TO BE PERFORMED: TSS 15.6.54, "ISOTHERMAL MODERATOR TEMPERATURE COEFFICIENT MEASUREMENTS," PERFORMED FOR UNIT 2 CYCLE 9 ON JANUARY 27-29, 1986: STEPS G.1, G.2 AND G.3.; TSS 15.6.55, "ROD AND BORON WORTH MEASUREMENTS," PERFORMED FOR UNIT 2 CYCLE 9 ON JANUARY 27, 1986: STEPS A.5.6, A.5.6.1 AND B.5.11. (3) THE STATION NUCLEAR ENGINEER SIGNED A STEP IN TSS 15.6.51 THAT STATED, "BASED ON ROD WORTH MEASUREMENTS ADEQUATE SHUTDOWN MARGIN EXISTS, "PRIOR TO THE TIME THAT THE PROCEDURE STEPS RELATED TO EVALUATING ROD WORTH MEASUREMENTS HAD BEEN SIGNED IN PROCEDURES TSS 15.6.53, "BORON ENDPOINT MEASUREMENT," TSS 15.6.55, "ROD AND BORON WORTH MEASUREMENTS," AND TSS 15.6.51, "ZERO POWER PHYSICS TESTING FOLLOWING REFUELING."
(8601 5)

OTHER ITEMS

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

THE UNIT IS OPERATING NORMALLY

LAST IE SITE INSPECTION DATE: 09/08/86

INSPECTION REPORT NO: 86018

REPORTS FROM LICENSEE

****
* ZION 2 *
****

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
86-16	06/27/86	07/28/86	REACTOR TRIP DUE TO LIGHTNING STRIKE

SECTION 3

APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY

PRESSURIZED WATER REACTORS

FACILITY	(a) CORE SIZE (NO. OF ASSEMBLIES)	PRESENT AUTH. STORAGE POOL CAP. (FUEL ASSEMBLIES)	NO. OF ASSEMBLIES STORED	REMAINING CAPACITY (NO. OF ASSEMBLIES)	REMAINING CAPACITY IF PENDING REQUEST APPROVED (NO. OF ASSEMBLIES)	NEXT REFUEL SCHED. DATE	(b) WILL FILL PRESENT AUTH. CAPACITY
ARKANSAS 1	177	988	384	604		09-86	1998
ARKANSAS 2	177	988	289	699		N/S	2003
BEAVER VALLEY 1	157	833	284	549		N/S	1995
BYRON 1	193	1050	0	1050		10-86	1995
CALLAWAY 1	193	1340	64	1336		N/S	1993
CALVERT CLIFFS 1	217	1830(c)	1000(c)	830(c)(m)	1098	10-86	1991
CALVERT CLIFFS 2	217					03-87	1991
CATAWBA 1	193	1418	0	1418		08-86	2008
COOK 1	193	2050(c)	866(c)	1184(c)		N/S	1994
COOK 2	193					N/S	1994
CRYSTAL RIVER 3	177	1163	328	829		N/S	1997
DAVIS-BESSE 1	177	735	204	531		03-87	1993
DIABLO CANYON 1	193	1400	0	1400		09-86	1993
DIABLO CANYON 2							
FARLEY 1	157	1407	273	1134		10-86	1991
FARLEY 2	157	1407	240	1167		N/S	1994
FORT CALHOUN 1	133	729	348	381		05-87	1996
GINNA	121	1016	420	596		N/S	1993
HADDAM NECK	157	1168	597	571		07-87	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	980	460	520		N/S	1993
INDIAN POINT 3	193	840	292	548		N/S	1993
KEWAUNEE	121	990	376	614(m)		N/S	1993
MAINE YANKEE	217	1476	721	755		N/S	1987
MCGUIRE 1	193	1463	220	1243(n)		N/S	2010
MCGUIRE 2	193	1463	141	1266		05-86	2010
MILLSTONE 2	217	1112	663	449		09-86	1994
MILLSTONE 3	0	0	0	0			
NORTH ANNA 1	157	1737(c)	520(c)	1217		N/S	1993
NORTH ANNA 2	157					N/S	1993
OCONEE 1	177	1312(l)	956	356(l)(n)		07-87	1991
OCONEE 2	177					08-86	1991
OCONEE 3	177	875	469	406		02-87	1991
PALISADES	204	798	477	321		N/S	2002
PALO VERDE 1	241	1329	0	1329		03-87	1993
PALO VERDE 2	241	1329	0	1329		01-88	2003
POINT BEACH 1	121	1502(c)	875(c)	626(c)		N/S	1995
POINT BEACH 2	121					09-86	1995
PRAIRIE ISLAND 1	121	1586(c)	781(c)	805(c)(m)		N/S	1993
PRAIRIE ISLAND 2	121					N/S	1993
RANCHO SECO 1	177	1080	316	764		03-87	2000
ROBINSON 2	157	541	274	266(e)	379	N/S	1988(g)
SALEM 1	193	1170	380	790		09-87	2001
SALEM 2	193	1170	140	1030		10-86	2004
SAN ONOFRE 1	157	216	146	70		03-88	1988
SAN ONOFRE 2	217	800	160	640		01-88	1997
SAN ONOFRE 3	217	800	72	728		11-86	

Report Period JUL 1986

STATUS OF SPENT FUEL STORAGE CAPABILITY

PRESSURIZED WATER REACTORS

FACILITY	(a) CORE SIZE (NO. OF ASSEMBLIES)	PRESENT AUTH. STORAGE POOL CAP. (FUEL ASSEMBLIES)	NO. OF ASSEMBLIES STORED	REMAINING CAPACITY (NO. OF ASSEMBLIES)	REMAINING CAPACITY IF PENDING REQUEST APPROVED (NO. OF ASSEMBLIES)	NEXT REFUEL SCHED. DATE	(b) WILL FILL PRESENT AUTH. CAPACITY
SEQUOYAH 1	193	1386	348	1033		N/S	1994
SEQUOYAH 2(d)	193					N/S	1994
ST LUCIE 1	217	728	372	356		N/S	1993
ST LUCIE 2	217	1076	152	924		N/S	1993
SUMMER 1	157	1276	96	1180		N/S	2008
SURRY 1	157	1044(c)	901(c)	143(c)		N/S	1987
SURRY 2	157					N/S	1987
THREE MILE ISLAND 1	177	752	208	544		11-86	1991
THREE MILE ISLAND 2	177	442	0	442		N/S	
TROJAN	193	1408	425	983		N/S	1993
TURKEY POINT 3	157	1404	445	959(m)		N/S	1993
TURKEY POINT 4	157	1404	482	922		N/S	1993
WATERFORD 3	217	1088	0	1088		N/S	1993
WOLF CREEK 1	193	1340	0	1340		N/S	
YANKEE-ROWE 1	76	721	325	396		N/S	1993
ZION 1	193	2112(c)	1008(c)	1104(c)		08-86	1995
ZION 2	193					02-87	1995

INDEPENDENT SPENT FUEL STORAGE INSTALLATIONS(h)

MORRIS OPERATIONS	750 MTU(j)	315	385 MTU(j)	1490 MTU(j)	
NFS(i)	250 MTU	170 MTU	80 MTU		

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(d) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2772 BWR and 1232 PWRassemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium; 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) Reserved.
(l) This is the station total.
(m) Installed capacity is less than that authorized.
(n) McGuire 1 authorized to accept Oconee fuel assemblies.

N/S = Not Scheduled

STATUS OF SPENT FUEL STORAGE CAPABILITY

BOILING WATER REACTORS

FACILITY	(a) CORE SIZE (NO. OF ASSEMBLIES)	PRESENT AUTH. STORAGE POOL CAP. (FUEL ASSEMBLIES)	NO. OF ASSEMBLIES STORED	REMAINING CAPACITY (NO. OF ASSEMBLIES)	REMAINING CAPACITY IF PENDING REQUEST APPROVED (NO. OF ASSEMBLIES)	NEXT REFUEL SCHED. DATE	(b) WILL FILL PRESENT AUTH. CAPACITY
BIG ROCK POINT 1	84	441	192	249		N/S	1993
BROWNS FERRY 1	764	3471	1288	2183		N/S	1993
BROWNS FERRY 2	764	3471	1161	2310(m)	1819	N/S	1993
BROWNS FERRY 3	764	3471	1004	2467(m)		N/S	1993
BRUNSWICK 1	560	(f)	160PWR+656BWR	123		N/S	1992
BRUNSWICK 2	560		144PWR+564BWR	107		N/S	1993
COOPER STATION	548	2366	790	1576		10-86	1996
DRESDEN 1	464	672	221	451		N/S	1990
DRESDEN 2	724	3537(c)	1413 (c)	2124(c)	(c)	12-86	1993
DRESDEN 3	724	3537	1271	2266		N/S	1993
DUANE ARNOLD	368	2050	696	1354		02-87	1998
FITZPATRICK	560	2244	1012	768		09-86	1992
GRAND GULF 1	800	1440	0	1440		09-86	1993
HATCH 1	560	6026	1580	4446		N/S	1999
HATCH 2	560			1325		09-86	1999
HUMBOLDT BAY	172	487	251	236		N/S	
LA CROSSE	72	440	261	179		N/S	1992
LASALLE 1	764	2162	191	1971		N/S	1988
LASALLE 2	764					N/S	1988
LIMERICK 1	764	2040	0	2040		N/S	1993
MILLSTONE 1	580	2184	1546	638		05-87	1991
MONTICELLO	484	2237	822	1415		N/S	1999
NINE MILE POINT 1	532	2776	1377	1399	1788	N/S	1996
OYSTER CREEK 1	560	2600	1902	698		N/S	1990

STATUS OF SPENT FUEL STORAGE CAPABILITY

BOILING WATER REACTORS

FACILITY	(a) CORE SIZE (NO. OF ASSEMBLIES)	PRESENT AUTH. STORAGE POOL CAP. (FUEL ASSEMBLIES)	NO. OF ASSEMBLIES STORED	REMAINING CAPACITY (NO. OF ASSEMBLIES)	REMAINING CAPACITY IF PENDING REQUEST APPROVED (NO. OF ASSEMBLIES)	NEXT REFUEL SCHED. DATE	(b) WILL FILL PRESENT AUTH. CAPACITY
PEACH BOTTOM 2	764	2816	1462	1354	2357	02-87	1995
PEACH BOTTOM 3	764	2816	1496	1320	2323	09-87	1996
PILGRIM 1	580	2320	1128	1192(m)		07-86	1990
QUAD CITIES 1	724	3657	1896	1761		09-87	2003
QUAD CITIES 2	724	3897	836	3061		10-86	2003
RIVER BEND 1							
SUSQUEHANNA 1	764	2840	382	2458		N/S	1997
SUSQUEHANNA 2	764	2840	0	2840		08-86	1997
VERMONT YANKEE 1	368	2000	1296	704		N/S	1992
WASHINGTON NUCLEAR*	764	2658	191	2467		N/S	1993

INDEPENDENT SPENT FUEL STORAGE INSTALLATIONS(h)

MORRIS OPERATIONS	750 MTU(j)	315	585 MTU(j)	1490 MTU(j)
NFS(i)	250 MTU	170 MTU	80 MTU	

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
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(l) This is the station total.
(m) Installed capacity is less than that authorized.
(n) McGuire 1 authorized to accept Oconee fuel assemblies.

N/S = Not Scheduled

(INCLUDES BOTH LICENSED AND NON-LICENSED UNITS)

REACTOR YEARS OF EXPERIENCE

	YEARS	1ST ELEC GENERATE	UNIT
**********			
* LICENSED *	12.00	08/01/74	ARKANSAS 1
* OPERATING *	23.65	12/08/62	BIG ROCK POINT 1
* ELECTRICAL *	9.88	09/12/76	BROWNS FERRY 3
* PRODUCING *	1.42	03/01/85	BYRON 1
* UNITS *	9.65	12/07/76	CALVERT CLIFFS 2
**********	8.36	03/22/78	COOK 2
	8.93	08/28/77	DAVIS-BESSE 1
	16.30	04/13/70	DRESDEN 2
	8.95	08/18/77	FARLEY 1
	12.93	08/25/73	FORT CALHOUN 1
	1.78	10/20/84	GRAND GULF 1
	7.86	09/22/78	HATCH 2
	12.31	04/08/74	KEWAUNEE
	2.28	04/20/84	LASALLE 2
	5.09	06/30/81	MCGUIRE 1
	10.73	11/09/75	MILLSTONE 2
	16.73	11/09/69	NINE MILE POINT 1
	13.24	05/06/73	OCONEE 1
	16.85	09/23/69	OYSTER CREEK 1
	.20	05/20/86	PALO VERDE 2
	14.03	07/19/72	PILGRIM 1
	12.66	12/04/73	PRAIRIE ISLAND 1
	14.19	05/23/72	QUAD CITIES 2
	15.85	09/26/70	ROBINSON 2
	19.04	07/16/67	SAN ONOFRE 1
	6.03	07/22/80	SEQUOYAH 1
	3.13	06/13/83	ST LUCIE 2
	13.39	03/10/73	SURRY 2
	12.12	06/19/74	THREE MILE ISLAND 1
	13.11	06/21/73	TURKEY POINT 4
	1.37	03/18/85	WATERFORD 3
	13.09	06/28/73	ZION 1
TOTAL 934.01 YRS			

YEARS	1ST ELEC GENERATE	UNIT
7.60	12/26/78	ARKANSAS 2
12.79	10/15/73	BROWNS FERRY 1
9.66	12/04/76	BRUNSWICK 1
1.77	10/24/84	CALLAWAY 1
1.52	01/22/85	CATAWBA 1
12.23	05/10/74	COOPER STATION
1.72	11/11/84	DIABLO CANYON 1
15.03	07/22/71	DRESDEN 3
5.19	05/25/81	FARLEY 2
9.64	12/11/76	FORT ST VRAIN
18.98	08/07/67	HADDAM NECK
13.10	06/26/73	INDIAN POINT 2
18.26	04/26/68	LA CROSSE
1.30	04/13/85	LIMERICK 1
3.19	05/23/83	MCGUIRE 2
.47	02/12/86	MILLSTONE 3
8.29	04/17/78	NORTH ANNA 1
12.65	12/05/73	OCONEE 2
14.58	12/31/71	PALISADES
12.45	02/18/74	PEACH BOTTOM 2
15.73	11/06/70	POINT BEACH 1
11.61	12/21/74	PRAIRIE ISLAND 2
11.80	10/13/74	RANCHO SECO 1
9.60	12/25/76	SALEM 1
3.86	09/20/82	SAN ONOFRE 2
4.61	12/23/81	SEQUOYAH 2
3.71	11/16/82	SUMMER 1
3.71	11/16/82	SUSQUEHANNA 1
10.61	12/23/75	TROJAN
13.86	09/20/72	VERMONT YANKEE 1
1.14	06/12/85	WOLF CREEK 1
12.60	12/26/73	ZION 2

YEARS	1ST ELEC GENERATE	UNIT
10.13	06/14/76	BEAVER VALLEY 1
11.93	08/28/74	BROWNS FERRY 2
11.26	04/29/75	BRUNSWICK 2
11.58	01/03/75	CALVERT CLIFFS 1
11.47	02/10/75	COOK 1
9.50	01/30/77	CRYSTAL RIVER 3
.78	10/20/85	DIABLO CANYON 2
12.20	05/19/74	DUANE ARNOLD
11.50	02/01/75	FITZPATRICK
16.66	12/02/69	GINNA
11.72	11/11/74	HATCH 1
10.26	04/27/76	INDIAN POINT 3
3.91	09/04/82	LASALLE 1
13.73	11/08/72	MAINE YANKEE
15.67	11/29/70	MILLSTONE 1
15.41	03/05/71	MONTICELLO
5.93	08/25/80	NORTH ANNA 2
11.92	09/01/74	OCONEE 3
1.14	06/10/85	PALO VERDE 1
11.92	09/01/74	PEACH BOTTOM 3
14.00	08/02/72	POINT BEACH 2
14.30	04/12/72	QUAD CITIES 1
.66	12/03/85	RIVER BEND 1
5.16	06/03/81	SALEM 2
2.85	09/25/83	SAN ONOFRE 3
10.23	05/07/76	ST LUCIE 1
14.08	07/04/72	SURRY 1
2.08	07/03/84	SUSQUEHANNA 2
13.74	11/02/72	TURKEY POINT 3
2.18	05/27/84	WASHINGTON NUCLEAR 2
25.72	11/10/60	YANKEE-ROWE 1

	YEARS	1ST ELEC GENERATE	SHUTDOWN DATE	UNIT
**********				
* PERMANENTLY *	3.80	08/14/64	06/01/68	BONUS
* OR *	18.54	04/15/60	10/31/78	DRESDEN 1
* INDEFINITELY*	6.32	08/05/66	11/29/72	FERMI 1
* SHUTDOWN *	13.21	04/18/63	07/02/76	HUMBOLDT BAY
* UNITS *	1.19	07/25/66	10/01/67	PATHFINDER
**********	2.16	11/04/63	01/01/66	PIQUA
TOTAL 74.77 YRS				

YEARS	1ST ELEC GENERATE	SHUTDOWN DATE	UNIT
3.04	12/18/63	01/01/67	CVTR
4.44	08/24/63	02/01/68	ELK RIVER
1.26	05/29/63	09/01/64	HALLAM
12.12	09/16/62	10/31/74	INDIAN POINT 1
7.76	01/27/67	11/01/74	PEACH BOTTOM 1
.93	04/21/78	03/28/79	THREE MILE ISLAND 2

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* RESEARCH *
* REACTORS *
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NON-POWER REACTORS IN THE U.S.

STATE	CITY	LICENSEE	REACTOR TYPE	DOCKET	LICENSE NUMBER	DATE OL ISSUED	AUTHORIZED POWER LEVEL (KW)
ALABAMA	TUSKEGEE	TUSKEGEE INSTITUTE	AGN-201 #102	50-406	R-122	08-30-74	0.0001
ARIZONA	TUCSON	UNIVERSITY OF ARIZONA	TRIGA MARK I	50-113	R-52	12-05-58	100.0
CALIFORNIA	BERKELEY	UNIVERSITY OF CALIFORNIA, BERKELEY COLLEGE	TRIGA MK. III	50-224	R-101	08-10-66	1000.0
	CANOGA PARK	ROCKWELL INTERNATIONAL CORP.	L-85	50-375	R-188	01-05-72	0.003
	HAWTHORNE	NORTHROP CORP. LABORATORIES	TRIGA MARK F	50-187	R-90	03-04-63	1000.0
	IRVINE	UNIVERSITY OF CALIFORNIA, IRVINE	TRIGA MARK I	50-326	R-116	11-24-69	250.0
	LOS ANGELES	UNIVERSITY OF CALIFORNIA, L.A.	ARGONAUT	50-142	R-71	10-03-60	100.0
	SAN DIEGO	GENERAL ATOMIC COMPANY	TRIGA MARK F	50-163	R-67	07-01-60	1500.0
	SAN DIEGO	GENERAL ATOMIC COMPANY	TRIGA MARK I	50-089	R-38	05-03-58	250.0
	SAN JOSE	GENERAL ELECTRIC COMPANY	NTR	50-073	R-33	10-31-57	100.0
	SAN LUIS OBISPO	CALIFORNIA STATE POLYTECHNIC COLLEGE	AGN-201 #100	50-394	R-121	05-16-73	0.0001
	SAN RAMON	AEROTEST OPERATIONS, INC.	TRIGA (INDUS)	50-228	R-98	07-02-65	250.0
	SANTA BARBARA	UNIVERSITY OF CALIFORNIA, SANTA BARBARA	L-77	50-433	R-124	12-03-74	0.01
COLORADO	DENVER	U.S. GEOLOGICAL SURVEY DEPARTMENT	TRIGA MARK I	50-274	R-113	02-24-69	1000.0
DELAWARE	NEWARK	UNIVERSITY OF DELAWARE	AGN-201 #113	50-098	R-43	07-03-58	0.0001
DIST OF COLUMBIA	WASHINGTON	THE CATHOLIC UNIVERSITY OF AMERICA	AGN-201 #101	50-077	R-31	11-15-67	0.0001
FLORIDA	GAINESVILLE	UNIVERSITY OF FLORIDA	ARGONAUT	50-083	R-56	05-21-59	100.0
GEORGIA	ATLANTA	GEORGIA INSTITUTE OF TECHNOLOGY	AGN-201 #104	50-276	R-111	04-19-68	0.0001
	ATLANTA	GEORGIA INSTITUTE OF TECHNOLOGY	HEAVY WATER	50-160	R-97	12-29-64	5000.0
IDAHO	POCATELLO	IDAHO STATE UNIVERSITY	AGN-201 #103	50-284	R-110	10-11-67	0.0001
ILLINOIS	URBANA	UNIVERSITY OF ILLINOIS	LOPRA	50-356	R-117	12-27-71	10.0
	URBANA	UNIVERSITY OF ILLINOIS	TRIGA	50-151	R-115	07-22-69	1500.0
	ZION	WESTINGHOUSE ELECTRIC CORP.	NTR	50-087	R-119	01-28-72	10.0
INDIANA	LAFAYETTE	PURDUE UNIVERSITY	LOCKHEED	50-182	R-87	08-16-62	10.0
IOWA	AMES	IOWA STATE UNIVERSITY	UTR-10	50-116	R-59	10-16-59	10.0
KANSAS	LAWRENCE	UNIVERSITY OF KANSAS	LOCKHEED	50-148	R-78	06-23-61	250.0
	MANHATTAN	KANSAS STATE UNIVERSITY	TRIGA	50-188	R-88	10-16-62	250.0
MARYLAND	BETHESDA	ARMED FORCES RADIOBIOLOGY RESEARCH INSTITUTE	TRIGA	50-170	R-84	06-26-62	1000.0
	COLLEGE PARK	UNIVERSITY OF MARYLAND	TRIGA	50-166	R-70	10-14-60	250.0

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* RESEARCH *
* REACTORS *
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NON-POWER REACTORS IN THE U.S.

STATE	CITY	LICENSEE	REACTOR TYPE	DOCKET	LICENSE NUMBER	DATE OL ISSUED	AUTHORIZED POWER LEVEL (KW)
MASSACHUSETTS	CAMBRIDGE	MASSACHUSETTS INSTITUTE OF TECHNOLOGY	HWR REFLECTED	50-020	R-37	06-09-58	5000.0
	LOWELL	UNIVERSITY OF LOWELL	GE	50-223	R-125	12-24-74	1000.0
	WORCESTER	WORCESTER POLYTECHNIC INSTITUTE	GE	50-134	R-61	12-16-59	10.0
MICHIGAN	ANN ARBOR	UNIVERSITY OF MICHIGAN	POOL	50-002	R-28	09-13-57	2000.0
	EAST LANSING	MICHIGAN STATE UNIVERSITY	TRIGA MARK I	50-294	R-114	03-21-69	250.0
	MIDLAND	DOW CHEMICAL COMPANY	TRIGA	50-264	R-108	07-03-67	100.0
MISSOURI	COLUMBIA	UNIVERSITY OF MISSOURI, COLUMBIA	TANK	50-186	R-103	10-11-66	10000.0
	ROLLA	UNIVERSITY OF MISSOURI	POOL	50-123	R-79	11-21-61	200.0
NEBRASKA	OMAHA	THE VETERANS ADMINISTRATION HOSPITAL	TRIGA	50-131	R-57	06-26-59	18.0
NEW MEXICO	ALBUQUERQUE	UNIVERSITY OF NEW MEXICO	AGN-201M #112	50-252	R-102	09-17-66	0.005
NEW YORK	BRONX	MANHATTAN COLLEGE - PYHSICS DEPT.	TANK	50-199	R-94	03-24-64	0.0001
	BUFFALO	STATE UNIVERSITY OF NEW YORK	PULSTAR	50-057	R-77	03-24-61	2000.0
	ITHACA	CORNELL UNIVERSITY	TRIGA MARK II	50-157	R-80	01-11-62	500.0
	ITHACA	CORNELL UNIVERSITY	ZPR	50-097	R-89	12-11-62	0.1
	NEW YORK	COLUMBIA UNIVERSITY IN THE CITY OF NEW YORK	TRIGA MARK II	50-208	R-128	04-14-77	250.0
	TUXEDO	UNION CARBIDE CORP	POOL	50-054	R-81	09-07-61	5000.0
NORTH CAROLINA	RALEIGH	NORTH CAROLINA STATE UNIVERSITY AT RALEIGH	PULSTAR	50-297	R-120	08-25-72	1000.0
OHIO	COLUMBUS	OHIO STATE UNIVERSITY	POOL	50-150	R-75	02-24-61	10.0
OKLAHOMA	NORMAN	THE UNIVERSITY OF OKLAHOMA	AGN-211 #102	50-112	R-53	12-29-58	0.100
OREGON	CORVALLIS	OREGON STATE UNIVERSITY	TRIGA MARK II	50-243	R-106	03-07-67	1000.0
	PORTLAND	REED COLLEGE	TRIGA MARK I	50-288	R-112	07-02-68	250.0
PENNSYLVANIA	UNIVERSITY PARK	PENNSYLVANIA STATE UNIVERSITY	TRIGA MK. III	50-005	R-2	07-08-55	1000.0
RHODE ISLAND	NARRAGANSETT	RHODE ISLAND NUCLEAR SCIENCE CENTER	GE POOL	50-193	R-95	07-21-64	2000.0
TENNESSEE	MEMPHIS	MEMPHIS STATE UNIVERSITY	AGN-201 #108	50-538	R-127	12-10-76	0.0001
TEXAS	AUSTIN	UNIVERSITY OF TEXAS	TRIGA MARK I	50-192	R-92	08-02-63	250.0
	COLLEGE STATION	TEXAS A&M UNIVERSITY	AGN-201M #106	50-059	R-23	08-26-57	0.005
	COLLEGE STATION	TEXAS A&M UNIVERSITY	TRIGA	50-128	R-83	12-07-61	1000.0
UTAH	PROVO	BRIGHAM YOUNG UNIVERSITY	L-77	50-262	R-109	09-07-67	0.01

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UTAH	SALT LAKE CITY	THE UNIVERSITY OF UTAH	TRIGA MARK I	50-407	R-126	09-30-75	100.0
	SALT LAKE CITY	UNIVERSITY OF UTAH	AGN-201M #107	50-072	R-25	09-12-57	0.005
VIRGINIA	BLACKSBURG	VIRGINIA POLYTECHNIC INSTITUTE	UTR-10	50-124	R-62	12-18-59	100.0
	CHARLOTTESVILLE	UNIVERSITY OF VIRGINIA	CAVALIER	50-396	R-123	09-24-74	0.1
	CHARLOTTESVILLE	UNIVERSITY OF VIRGINIA	POOL	50-062	R-66	06-27-60	2000.0
	LYNCHBURG	BABCOCK & WILCOX COMPANY	LPR	50-099	R-47	09-05-58	1000.0
WASHINGTON	PULLMAN	WASHINGTON STATE UNIVERSITY	TRIGA	50-027	R-76	03-06-61	1000.0
	SEATTLE	UNIVERSITY OF WASHINGTON	ARGONAUT	50-139	R-73	03-31-61	100.0
WISCONSIN	MADISON	UNIVERSITY OF WISCONSIN	TRIGA	50-156	R-74	11-23-60	1000.0
********** * EXPERIMENTAL AND TEST REACTORS * **********							
CALIFORNIA	SAN JOSE	GENERAL ELECTRIC COMPANY	GETR	50-070	TR-1	01-07-59	50,000.0
DIST OF COLUMBIA	WASHINGTON	NATIONAL BUREAU OF STANDARDS	TEST	50-184	TR-5	06-30-70	10,000.0
********** * CRITICAL EXPERIMENT FACILITIES * **********							
NEW YORK	TROY	RENSSELAER POLYTECHNIC INSTITUTE		50-225	CX-22	07-03-64	0.0
VIRGINIA	LYNCHBURG	BABCOCK & WILCOX COMPANY		50-013	CX-10	10-22-58	0.0
WASHINGTON	RICHLAND	BATTELLE MEMORIAL INSTITUTE		50-360	CX-26	11-29-71	0.0

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Status Summary Report

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Status Summary Report

14 ABSTRACT (200 words or less)

The OPERATING UNITS STATUS REPORT - LICENSED OPERATING REACTORS provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Resource Management from the Headquarters staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. The three sections of the report are: monthly highlights and statistics for commercial operating units, and errata from previously reported data; a compilation of detailed information on each unit, provided by NRC's Regional Offices, IE Headquarters and the utilities; and an appendix for miscellaneous information such as spent fuel storage capability, reactor-years of experience and non-power reactors in the U.S. It is hoped the report is helpful to all agencies and individuals interested in maintaining an awareness of the U.S. energy situation as a whole.

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