

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)  
Hope Creek Generating StationDOCKET NUMBER (2)  
0 5 0 0 0 3 5 4 1 OF 0 3TITLE (4)  
Inadvertent Opening of "P" Safety Relief Valve

| EVENT DATE (5) |     |      | LER NUMBER (6) |                   |                 | REPORT DATE (7) |     |      | OTHER FACILITIES INVOLVED (8) |   |                  |
|----------------|-----|------|----------------|-------------------|-----------------|-----------------|-----|------|-------------------------------|---|------------------|
| MONTH          | DAY | YEAR | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH           | DAY | YEAR | FACILITY NAMES                |   | DOCKET NUMBER(S) |
| 1              | 2   | 0    | 5              | 8                 | 6               | 8               | 6   | 0    | 9                             | 1 | 0 5 0 0 0        |
|                |     |      |                |                   |                 |                 |     |      |                               |   | 0 5 0 0 0        |

| OPERATING MODE (9) |       | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11) |  |                  |   |                      |  |  |  |  |  |
|--------------------|-------|---|--|------------------|---|----------------------|--|--|--|--|--|
| 1                  |       | 20.402(b)   |  | 20.406(e)        | X | 50.73(a)(2)(iv)      |  | 73.71(b)   |  |  |  |
| POWER LEVEL (10)   | 6 7 0 | 20.406(a)(1)(i)   |  | 50.36(c)(1)      |   | * 50.73(a)(2)(v)     |  | 73.71(c)   |  |  |  |
|                    |       | 20.406(a)(1)(ii)  |  | 50.36(c)(2)      |   | 50.73(a)(2)(vii)     |  | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |  |  |  |
|                    |       | 20.406(a)(1)(iii)   |  | 50.73(a)(2)(i)   |   | 50.73(a)(2)(viii)(A) |  |  |  |  |  |
|                    |       | 20.406(a)(1)(iv)  |  | 50.73(a)(2)(ii)  |   | 50.73(a)(2)(viii)(B) |  |  |  |  |  |
|                    |       | 20.406(a)(1)(v)   |  | 50.73(a)(2)(iii) |   | 50.73(a)(2)(ix)      |  |  |  |  |  |

LICENSEE CONTACT (12) THIS LER (12)  
NAME  
Ann M. Ervin, Lead Engineer-TechnicalTELEPHONE NUMBER  
AREA CODE  
6 0 9 3 3 9 - 5 2 3 9

| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) |        |           |              |                   |       |        |           |              |                   |
|--|--------|-----------|--------------|-------------------|-------|--------|-----------|--------------|-------------------|
| CAUSE  | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NRC |
|  |        |           |              |                   |       |        |           |              |                   |
|  |        |           |              |                   |       |        |           |              |                   |

SUPPLEMENTAL REPORT EXPECTED (14)  
YES (If yes, complete EXPECTED SUBMISSION DATE) X NO  
EXPECTED SUBMISSION DATE (15)  
MONTH DAY YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single space typewritten lines) (16)

On December 6, 1986 at 0953 the "P" Safety/Relief Valve opened and remained open for 45 seconds due to a Nuclear Control Operator (NCO) error in setting the SRV switch while performing the Test and Trip Point Calibration of Safety Relief Valve Low-Low Set Pressure Channel Procedure. The NCO was counselled on the necessity of carefully reading and following each procedure step. This occurrence and its significance has been discussed with all NCO's and will be discussed with all I&C Technicians at future meetings.

8701080311 870102  
PDR ADOCK 05000354  
S PDR

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## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO 3150-0104

EXPIRES 8/31/85

|  |  |                |                   |                 |          |    |     |
|--|--|----------------|-------------------|-----------------|----------|----|-----|
| FACILITY NAME (1)<br><br>Hope Creek Generating Station | DOCKET NUMBER (2)<br><br>0 5 0 0 0 3 5 4 8 6 | LER NUMBER (6) |                   |                 | PAGE (3) |    |     |
|  |  | YEAR           | SEQUENTIAL NUMBER | REVISION NUMBER |          |    |     |
|  |  | 0 9            | 1                 | 0 0             | 0 2      | OF | 0 3 |

TEXT (If more space is required, use additional NRC Form 368A's) (17)

PLANT AND SYSTEM IDENTIFICATION

General Electric - Boiling Water Reactor (BWR/4)  
Safety/Relief Valve System (EIIIS Designator:SB)

IDENTIFICATION OF OCCURRENCE

Inadvertent opening of "P" Safety Relief Valve.  
Event Date: 12/06/86  
Event Time: 0953  
This LER was initiated by Incident Report No. 86-262.

CONDITIONS PRIOR TO OCCURRENCE

Plant in Operational Condition 1. Test and Trip Point Calibration of SRV Low-Low Set Pressure Channel in progress.

DESCRIPTION OF OCCURRENCE

The "P" Safety/Relief Valve opened and remained open for 45 seconds due to an operator error in the performance of a SRV functional test.

APPARENT CAUSE OF OCCURRENCE

This occurrence was caused by a personnel error while performing a test procedure.

ANALYSIS OF OCCURRENCE

The "P" Safety Relief Valve (PSV-F013P) lifted for 45 seconds while procedure IC-FT.BB-038(Q), Test and Trip Point Calibration of Drywell Pressure Channel B21-N694G, was in progress. The I&C Technician performing the procedure requested that the NCO place the SRV switch (B21C-S36D) in the CLOSE position (Procedure step 5.1.4). This position positively prevents the opening of the SRV during the performance of the balance of this procedure. The NCO misinterpreted the procedure step and verified that the SRV was closed rather than placing the switch in the CLOSE position. The switch remained in AUTO thereby closing contacts which align trip unit (B21-N760D) to permit pressure cycling of the SRV. With the Master Trip Unit (B21-N707D) verified as being in trip status, per procedure step 5.2.1.4, relay B21H-K115H is energized and relay B21H-K116D is energized and sealed in. The SRV was then in a condition to cycle with the cycling of K117D only. Step 5.2.1.15 tripped MTU B21-N706D which energized relay B21H-K117D, sending an OPEN signal to the SRV. The following alarms and/or indications were observed in the Control Room:

## LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES 8/31/85

|  |   |                |                      |                    |          |  |  |
|--|---|----------------|----------------------|--------------------|----------|--|--|
| FACILITY NAME (1)<br><br>Hope Creek Generating Station | DOCKET NUMBER (2)<br><br>0 5 0 0 0 3 5 4 8 6 - 0 9 1 - 0 0 0 3 OF 0 3 | LER NUMBER (6) |                      |                    | PAGE (3) |  |  |
|  |   | YEAR           | SEQUENTIAL<br>NUMBER | REVISION<br>NUMBER |          |  |  |
|  |   |                |                      |                    |          |  |  |

TEXT (If more space is required, use additional NRC Form 388A's) (17)

ANALYSIS OF OCCURRENCE CONT'D

ADS/Safety-Relief Valve open  
Suppression Pool temp.- high  
Suppression Pool temp. Monitoring System - 95 degrees, A&B  
Channels  
~40 MWE load loss

The Control Room staff initially assumed that these alarms occurred as a consequence of the testing in progress, however upon observing the elevated Suppression Pool temperature, the SRV open condition was recognized and the SRV was closed. The SRV discharged to the Suppression Pool per plant design therefore the health and safety of the public and the plant staff was not compromised by this occurrence.

CORRECTIVE ACTION

The procedure steps involved were reviewed and it was determined that no corrections were needed.

The NCO was counselled concerning the necessity of carefully reading and following each procedure step. This occurrence and its consequences were discussed with all NCO's on each shift and will also be discussed with all I&C Technicians at their weekly informational meeting.

Sincerely,

R. S. SALVESEN *RS*

R. S. Salvesen  
General Manager -  
Hope Creek Operations

AME:dyp

SORC Mtg. 87-001





**PSEG**

Public Service Electric and Gas Company P.O. Box L Hancocks Bridge, New Jersey 08038

Hope Creek Operations

January 2, 1987

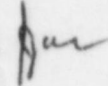
U.S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, DC 20555

Dear Sir:

HOPE CREEK GENERATING STATION  
DOCKET NO. 50-354  
UNIT NO. 1  
LICENSEE EVENT REPORT 86-091-00

This Licensee Event Report is being submitted pursuant to the requirements of 10CFR50.73(a)(2)(iv).

Sincerely,

*R. S. SALVESEN* 

R. S. Salvesen  
General Manager -  
Hope Creek Operations

AME:dyp

Attachment  
SORC Mtg. 87-001

C Distribution

*TE22*  
*1/1*