

NRC MONTHLY OPERATING REPORT

DOCKET NO. 50-528
UNIT NAME PVNGS-1
DATE 03/20/86
COMPLETED BY M. P. Richardson
TELEPHONE 602-932-5300
Ext. 6593

OPERATING STATUS

1. Unit Name: Palo Verde Nuclear Generating Station, Unit 1
2. Reporting Period: January 1986
3. Licensed Thermal Power (Mwt): 3800
4. Nameplate Rating (Gross MWe): 1403
5. Design Electrical Rating (Net MWe): 1270
6. Maximum Dependable Capacity (Gross MWe): To be determined
7. Maximum Dependable Capacity (Net MWe): To be determined
8. If Changes Occur In Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: _____
Recalculated According to definition _____
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons For Restrictions, If Any: _____

	This Month	Yr.-to-Date	Cumulative
11. Hours In Reporting Period	744	744	9504
12. Number Of Hours Reactor Was Critical	385.5	385.5	2812.5
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	363.8	363.8	2383.5
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	1,265,571	1,265,571	5,659,762
17. Gross Electrical Energy Generated (MWH)	428,000	428,000	1,764,700
18. Net Electrical Energy Generated (MWH)	383,457	383,457	1,511,107
19. Unit Service Factor	N/A	N/A	N/A
20. Unit Availability Factor	N/A	N/A	N/A
21. Unit Capacity Factor (Using MDC Net)	N/A	N/A	N/A
22. Unit Capacity Factor (Using DER Net)	N/A	N/A	N/A
23. Unit Forced Outage Rate	N/A	N/A	N/A
24. Shutdowns Scheduled Over Next 6 months (Type, Date, and Duration of Each):	Surveillance Test Outage - 3/86, 49 days		
25. If Shutdown At End Of Report Period, Estimated Date of Startup:			
26. Units In Test Status (Prior To Commercial Operation):			

INITIAL CRITICALITY
INITIAL ELECTRICITY
COMMERCIAL OPERATION

Forecast	Achieved
5/85	5/25/85
6/85	6/10/85
11/85	N/A

REFUELING INFORMATION

DOCKET NO. 50-582

UNIT PVNGS-1

DATE 02/08/86

COMPLETED BY M. P. Richardson

TELEPHONE 602-932-5302

Ext. 6533

1. Scheduled date for next refueling shutdown.

03/01/87

2. Scheduled date for restart following refueling.

04/19/87

3. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment?

Not Yet Determined
What will these be?

Not Yet Determined

4. Scheduled date for submitting proposed licensing action and supporting information.

Not Yet Determined

5. Important Licensing considerations associated with refueling, e.g. new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.

Not Yet Determined

6. The number of fuel assemblies.

a) In the core. 241

b) In the spent fuel storage pool. 0

7. Licensed spent fuel storage capacity. 1329

Intended change in spent fuel storage capacity. None

8. Projected date of last refueling that can be discharged to spent fuel storage pool assuming present capacity.

2002 (w/annual reloads and full core discharge capability).

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-520

UNIT PVNGS-1

DATE 02/08/86

COMPLETED BY M.D. Richardson

TELEPHONE 602-232-5300

Ext. 6533

MONTH: January 1986

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

1	1,253
2	1,256
3	1,180
4	1,263
5	1,266
6	1,161
7	1,147
8	1,236
9	658
10	(75)
11	(77)
12	(79)
13	(76)
14	(72)
15	(72)
16	(72)

DAY AVERAGE DAILY POWER LEVEL
(MWe-Net)

17	(78)
18	(72)
19	(66)
20	855
21	1,132
22	1,221
23	1,199
24	564
25	(74)
26	(74)
27	(61)
28	(55)
29	(63)
30	581
31	1,078

SUMMARY OF OPERATING EXPERIENCE FOR THE MONTH

DOCKET NO. 50-526
UNIT PVNGS-1
DATE 02/08/86
COMPLETED BY M.P. Richardson
TELEPHONE 602-233-5300
Ext. 6593

- 01/01 0000 - Reactor power at 100%, main turbine 1307 MWe.
- 01/03 1125 - Commenced power decrease for surveillance testing.
- 01/03 1132 - Reactor power at 85%.
- 01/03 1202 - Reactor power at 71%.
- 01/03 1219 - Commenced power increase to 95%.
- 01/03 1320 - Reactor power at 94.9%, main turbine 1245 MWe.
- 01/06 1025 - Transferred house loads from auxiliary transformers to startup transformers in preparation for loss of feedwater pump test.
- 01/06 1238 - Reactor power at 62% power.
- 01/06 1314 - Transferred house loads from startup transformers to auxiliary transformers in recovery from loss of feedwater pump test.
- 01/06 1455 - Reactor power at ~100%.
- 01/07 1533 - Performed reactor power cutback test for load reject. Tripped main generator.
- 01/07 1546 - Placed main generator on line.
- 01/07 1656 - Reactor power at 47% due to gas stripper malfunction.
- 01/07 1850 - Reactor power at 96%.
- 01/09 1325 - Initiated Load Reject Test, main turbine trip via master trip. Reactor trip on to DNER caused by Reactor Coolant Pump coastdown on 13.8 KV load shed due to failure of fast transfer. One main steam safety valve lifted.

01/29 1327 - MSIS on low Steam Generator pressure.

01/29 1329 - Started opening Steam Generator atmospheric dump valves.

01/29 1930 - Plant in Mode 3.

01/14 0205 - Completed plant cooldown/depressurization to approximately 400 degrees F/520 psia to allow repairs to the isolation valves to the pressurizer spray valves.

01/15 0130 - Plant in Mode 2.

01/19 0834 - Reactor is critical.

01/19 1042 - Entered Mode 1.

01/19 2247 - Main Turbine on line.

01/19 2250 - Commenced raising Rx power to 15%.

01/21 0720 - Plant in Mode 1, Rx power at 50%, main turbine at 1157 MWe.

01/22 1930 - Rx power at 53.2%, main turbine at 1229 MWe.

01/24 1130 - Performed Loss of Load without Reactor Power Cutback System. Rx tripped on HI pressurizer pressure. All four RCPs tripped for Natural Circulation Cooldown Test. During this test a secondary safety lifted.

01/24 2225 - Entered Mode 4.

01/25 0115 - A steam void was intentionally drawn in the Rx vessel head during RCS depressurization. The steam void was collapsed with use of the reactor coolant gas vent system venting from the vessel head to the RDT.

01/29 0250 - Entered Mode 3.

01/29 1538 - Entered Mode 2.

01/29 1536 - Reactor critical.

01/29 1550 - Entered Mode 1.

01/29 2213 - Main Turbine on line at 40 MWe.
01/30 0700 - Mode 1, Reactor power at 40%, Main generator at 438 MWe.
01/30 1520 - Reactor power at 69%.
01/31 0425 - Stabilized Reactor power at 99%.
01/31 0822 - Started the 120 hour continuous run for the Arizona
Corporation Commission.
01/31 1520 - Started borating to ~60% power due to H₁ Vibration on B
FWPT.
01/31 1555 - Stopped power decrease at ~70%.
01/31 1928 - Mode 1, Reactor power at 72%, Main Generator at 911 MWe.

UNIT SHUTDOWNS AND LER REDUCTIONS

DOCKET NO. 50-528
 UNIT NAME PVNGS-1
 DATE Feb. 10, 1986
 COMPLETED BY H.P. Richardson
 TELEPHONE 932-5300
 Pwr 6593

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	LER No.	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
29	01/03/86	S	0	B	5				Surveillance testing.
30	01/06/86	S	0	B	5				Loss of feedwater pump test.
31	01/07/86	S	.2	S	N/A				Reactor power cutback test.
32	01/07/86	F	0	A	N/A		CB	DGS	Gas stripper malfunction.
33	01/09/86	S	249.3	B	3	1-86-0006	EA	78	Load Reject Test.
34	01/24/86	S	130.7	B	3	1-86-015			Performed Loss of Load without Reactor Power Cutback System.
35	01/31/86	F	0	A	5		SJ	P	Decrease power due to Hi vibration on B FWPT. Fixed pump.

¹
 F-Forced
 S-Scheduled

² Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulatory Restriction
 E-Operator Training & License Examination
 F-Administrative
 G-Operational Error (Explain)

³ Method:
 1-Manual
 2-Manual Scram.
 3-Automatic Scram.
 4-Continuation from
 Previous Month
 5-Reduction of 20%
 or Greater in the

⁴ Exhibit F - Instructions
 for Preparation of Data
 Entry Sheets for Licensee
 Event Report (LER) File
 (NUREG 0161)

⁵ Exhibit H - Same Source