

Mr. Joseph V. Sipek  
Director - Licensing  
Clinton Power Station  
P.O. Box 678  
Mail Code V920  
Clinton, IL 61727

May 28, 1999

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION RELATED TO GENERIC LETTER 95-07, "PRESSURE LOCKING AND THERMAL BINDING OF SAFETY-RELATED POWER-OPERATED GATE VALVES," CLINTON POWER STATION, UNIT 1 (TAC NO. M93448)

Dear Mr. Sipek:

On August 17, 1995, the NRC issued Generic Letter (GL) 95-07, "Pressure Locking and Thermal Binding of Safety-Related Power-Operated Gate Valves," to request that licensees take actions to ensure that safety-related power-operated gate valves that are susceptible to pressure locking or thermal binding are capable of performing their safety functions.

In a letter dated February 9, 1996, Illinois Power Company submitted its 180-day response to GL 95-07 for the Clinton Power Station. The NRC staff reviewed your submittal and requested additional information in a letter dated May 23, 1996. In letters dated June 27, and August 5, 1996, you provided the additional information.

A telephone call was held with your staff on May 5, 1999, to discuss your responses. From our review to-date, including the telephone call, we find that additional information is needed to complete our review. The desired information was discussed with your staff. Enclosed is a request for additional information (RAI) regarding the GL 95-07 program at Clinton. Please provide the requested information within 45 days of your receipt of this letter.

Contact me if you have any questions.

Sincerely,

Original signed by

Jon B. Hopkins, Senior Project Manager, Section 2  
Project Directorate III  
Division of Licensing Project Management  
Office of Nuclear Reactor Regulation

Docket No. 50-461

Enclosure: Request for Additional Information

cc w/encl: See next page

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Joseph V. Sipek  
Illinois Power Company

Clinton Power Station, Unit 1

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REQUEST FOR ADDITIONAL INFORMATION FOR RESOLUTION OF  
GL 95-07 ISSUES AT THE CLINTON POWER STATION  
(TAC NO. M93448)

1. In a letter dated May 23, 1996, the NRC requested your analysis or evaluation that demonstrated that the high pressure core spray pump suppression pool suction valve, 1E22-F015, and the reactor core isolation cooling pump suppression pool suction valve, 1E51-F031, were not susceptible to thermal induced pressure locking following postulated accidents that cause the suppression pool temperature to increase. Your responses dated June 27, and August 5, 1996, use engineering judgment to determine that the temperature of these valves would not increase; and therefore, these valves are not susceptible to thermal induced pressure locking. The NRC staff considers that it is not appropriate to use engineering judgment to determine that these valves are not susceptible to pressure locking due to the geometry of the piping between the valves and the suppression pool. Provide the calculations that demonstrate that heat will not be transferred from the suppression pool to the valves during a postulated accident.
2. During a postulated accident, the containment would be initially pressurized to a peak pressure and the bonnets of 1E22-F015 and 1E51-F031 could also be pressurized to containment peak pressure. When transferring to the recirculation phase of a postulated accident, containment pressure could be lower than the initial peak pressure but the pressure in the bonnets of 1E22-F015 and 1E51-F031 could still be at containment peak pressure. Discuss if the pressure in the bonnets of the valves could be higher than upstream and downstream pressure due to changes in containment pressure when the valves are required to open and, if applicable, if the valves will open during this pressure locking condition.

Enclosure