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MEMORANDUM FOR: Jesse L. Funches, Director, Planning and Program Analysis

Staff, NRR

FROM: Charles E. Norelius, Director, Division of Reactor Projects,

RIII

SUBJECT: NTOL PLANT LICENSING STATUS - REGION III

Enclosed is the updated plant licensing status for Braidwood 1. This information is current as of April 15, 1987. The Commission was briefed by the Staff on April 10, 1987, regarding the Clinton Power Station. At the conclusion of the briefing the Commission voted 5 to 0 to authorize issuance of a full power license. Therefore, updating information on Clinton is not being submitted as agreed to in discussion between S. Black of your staff and R. F. Warnick of my staff on April 15, 1987. Should you or your staff have any further questions, please contact R. C. Knop for Clinton and W. S. Little for Braidwood.

ORIGINAL SIGNED BY C. E. NORELIUS

Charles E. Norelius, Director Division of Reactor Projects

Enclosure: Updated Plant Licensing Status

cc w/enclosure:
A. B. Davis, RIII
J. A. Hind, RIII
C. J. Paperiello, RIII
N. J. Chrissotimos, RIII
SRI, Clinton
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RIII Knop/jp RII)

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Norellius 4/16/87

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LICENSING STATUS OF BRAIDWOOD 1

A license authorizing fuel loading and precritical testing for Braidwood Unit 1 was issued on October 17, 1986. Fuel loading began on October 25, 1986, and was completed on November 3, 1986. The FES was issued in June 1984. The SER was issued in November 1983, and SSERs 1 and 2 were issued in September and October, 1986, respectively. The letter from the ACRS supporting issuance of an OL was issued on February 11, 1985. The ASLB hearings commenced on October 29, 1985, and were completed on November 26, 1986. The hearing record was closed on December 17, 1986. A final ASLB decision is not expected until April 1987.

FSAR Review

The application for the Byron and Braidwood Stations was submitted and accepted for review under the Commission's standardization policy statement of March 5, 1973. This application was for two duplicate units at each of two sites; therefore, the staff's review of the duplicate design portions of these four units was conducted simultaneously and documented in the Byron SER. The staff's review of the Braidwood site-specific portion of the Byron/Braidwood FSAR is essentially complete. There are three outstanding and four confirmatory items still under review.

Hearings

The Braidwood application is contested; the OL evidentiary hearing was completed on November 26, 1986. The hearing was closed on December 17, 1986. Initial hearings on the emergency preparedness contention were conducted on October 29, 1985; hearings were completed for this contention on March 12, 1986. The parties have submitted proposed findings on this contention and the matter is pending before the Licensing Board for its decision. In its June 21, 1985 Order, the ASLB admitted a complex contention alleging inadequate QA/QC during the construction of Braidwood Station. Directed certification to the Appeal Board of this Order was requested by the applicant and supported by the NRC staff in its July 23, 1985 filing. On September 6, 1985, the Appeal Board denied the grant of directed certification by a 2 to 1 majority. The applicant filed a request to waive the regulations prohibiting an appeal to the Commission of the Appeal Board denial of the grant of directed certification. This request was supported by the staff and opposed by the intervenor. On April 23, 1986, the Commission dismissed the quality assurance contention. An additional contention was accepted by the ASLB on July 23, 1985 concerning harassment, intimidation, retaliation and other discrimination. The hearing on this contention has been completed and spanned 97 hearing days between May 6, 1986 and November 26, 1986.

On August 18, 1986, the applicant filed a request pursuant to 10 CFR 50.57(c) for authorization of fuel load and precritical testing. The staff submitted a response supporting the request; the intervenors opposed the request. On September 18, 1986, the ASLB issued an Order granting the applicant's motion.

Construction Appraisal Team (CAT) Inspection

A CAT Inspection was conducted by the Office of Inspection and Enforcement during December 10-20, 1984 and January 7-18, 1985. The Inspection Report

(50-456/84-44; 50-457/84-40) was issued February 20, 1985. Although construction was generally found to be in accordance with requirements, the team identified a number of hardware deficiencies and program weaknesses as potential enforcement action items. These included vendor tanks and heat exchangers with deficient welds, vendor radiographs not meeting required weld or film quality requirements, failure to identify the required weld sizes for pipe supports and restraints, installation of unqualified switchboard wire and bolts of indeterminate material, failure to identify deficient seismic pipe support and restraint installations, and failure to provide adequate electrical separation criteria in quality control procedures. The potential enforcement actions were issued as violations by the Region in April 1985. Seven of the nine violations have been closed by the Region.

Independent Design Review (IDR)

On April 24, 1985, the applicant submitted its plans for ensuring that all applicable corrective actions resulting from the Byron 1 IDI and IDR, and the Clinton 1 IDR are incorporated into the Braidwood design. The NRC review of this information has been completed; an NRC inspection was conducted on November 18-22, 1985 to verify the proper implementation of these corrective actions. The inspection report was issued on April 2, 1986. All open items have been closed by the Region.

Region III Inspections

There are approximately 24 and 25 matters which require resolution by the applicant prior to initial criticality and 5% power, respectively. These include open items, violations, 10 CFR 50.55(e) reportable deficiencies, LERs, allegations, unresolved items, generic letters, IE Circulars, IE Bulletins, and 10 CFR 21 reports. There are also a number of normal inspection modules that remain to be completed in the following areas (percent completion for each area is included):

- Operations OA Program (74%)
- Safety Committee Inspection (50%)
- ° Fire Protection (75%)
- Radiation Protection (95%)
- ° Radwaste (30%)
- SER Review and Followup (75%)
- Radiological and Chemical Confirmatory Measurements (90%)
- Emergency Preparedness (95%)
- Startup Test Procedure Review (50%)
- Startup Test Results (0%)
- Startup Test Witnessing (0%)
- Overall Startup Test Program (80%)
- Low Power Test Procedure Review (0%)
- Power Ascension Test Procedure Review (0%)
- Precritical Data Review (0%)
- Initial Criticality Witnessing (0%)

Security and Safeguards

The licensee has satisfied the minimum requirements of their security plans required for licensing.

Radiation Protection and Radwaste

The most recent inspection was conducted October 6-8, 1986. There are presently four open items.

Operator Licensing

Confirmation that the senior reactor operators have obtained the requisite experience on shift is required. The first group of operator examinations was completed in April 1986. Sixteen of seventeen Senior Reactor Operators, and six of six Reactor Operators passed. The second group of operation examinations was completed in August 1986. Twelve of seventeen Senior Reactor Operators and seven of eight Reactor Operators passed. The third group of operator examinations were conducted during the weeks of November 10 and November 17, 1986. Fifteen of seventeen Senior Reactor Operators and six of seven Reactor Operators passed.

Allegations

There are seven open allegation files which require Region III action.

OI Investigations

The Office of Investigations has four (three applicant plus one vendor) open investigation files.

Fire Protection

The NRR Site Audit and the Region III Appendix R Team inspection were conducted the week of August 18, 1986, which resulted in several items of concern being raised. During subsequent discussions between the licensee and NRR, all NRR concerns were addressed adequately and are planned to be documented in Supplemental Safety Evaluation Report No. 3. Followup of the Region III concerns was performed by NRC contractors (EG&G) during an onsite inspection conducted February 23-March 6, 1987. The majority of Region III concerns were satisfactorily resolved during this inspection. Closure of the Region III concerns are planned to be documented in Inspection Report (456/87007; 457/87006). The remaining open items are scheduled to be followed by regional personnel.

Braidwood Construction Assessment Program (BCAP)

For background information on this issue, please refer to the November 15, 1985 report. The final presentation to the NRC concerning BCAP was conducted on October 15, 1985; the final BCAP report was issued on November 14, 1985, by

Commonwealth Edison Company for NRC review. The NRC review of the final BCAP report is documented in Inspection Report Nos. 456/86003; 457/86003 dated June 9, 1986. The staff presented testimony regarding its evaluation of BCAP at the November 20, 21, and 26, 1986 hearing sessions.

Emergency Preparedness

All onsite and offsite emergency preparedness licensing requirements have been completed satisfactorily.

By letter dated August 22, 1986, the applicant requested an exemption from the requirement to conduct an emergency preparedness exercise, involving the full scale participation of the State of Illinois and all three counties within the 10-mile EPZ, within one year of exceeding 5% power. The applicant has been informed that the 1987 exercise must also involve participation by the State of Indiana to satisfy the requirements of 10 CFR 50, Appendix E, IV.F.1. FEMA has determined that the State of Indiana's involvement in the 1985 exercise was minimal, and inadequate to satisfy FEMA's exercise evaluation criteria associated with the aforementioned Appendix E requirement. The State of Indiana successfully participated in the March 1987 exercise which satisfies the regulatory requirements. The applicant's first exercise was conducted in November 1985, and the last exercise was conducted in mid-March 1987. This exemption request is currently under review.

As part of FEMA's 44 CFR 350 approval process for offsite emergency planning a public meeting has been scheduled for May 20, 1987. The meeting will take place in the town of Braidwood. It will be hosted by the State of Illinois, and will include presentations by regional FEMA and NRC representatives on the offsite and licensee emergency preparedness programs for the Braidwood Station, respectively.

Plant: Braidwood Station, Unit 1 PM: J. A. Stevens (x 27702)

Technical Specifications:
Proof and Review: Completed
Final (Appendix A Version): Completed

FSAR OPEN ISSUES		Responsible Org.	Category	SSER Schedule Input Issued	Comments
1.	Control room human factors review	EICSB	3		Applicant's Summary Report of the DCRDR dated December 1, 1986, is under review.
2.	Safety parameter display system	EICSB			Applicant will submit V/V Report & correct identification of the wide-and narrow-range displays prior to startup fol- lowing the first refueling outage.
3.	Fire Protection Program	PSB	3		Applicant's submittal under review.

Plant: Braidwood Station, Unit 1 PM: J. A. Stevens (x 27702)

Technical Specifications:
Proof and Review: Completed
Final (Appendix A Version): Completed

FSAR Confirmatory Issues		Responsible Org.	Category	Schedule Issued	Comments
1.	Inservice testing pumps and valves TMI Action Plan Item II.D.1	RSB	3		Submittal dated 8/20/86 under review. Applicant's submittal
3.	Anticipated transients without scram (ATWS) Steam generator tube failure				under review. ATWS implementation design under review. Applicant to submit all plant-specific information at least 6 months prior to the first refueling outage.

Plant: Braidwood Station, Unit 1 PM: J. A. Stevens (x 27702)

Technical Specifications:
Proof and Review: Completed
Final (Appendix A Version): Completed

FSAR License Conditions		SSER Schedule					
		Responsible Org.	Category	Input	Issued	Comments	
1.	Inservice inspection program	EB	3			Applicant will submit ISI program 12 months prior to 1st refueling outage.	
2.	TMI Item II.F.1: Iodine/Particu- late Sampling	PSB	3			Applicant has installed an NRR approved modifi- cation. A review of installation and testing data will be conducted during a future Region III	
3.	Fire Protection Program Emergency diesel engine auxiliary support systems	PSB	3			inspection. Standard L. C. Applicant will resolve this matter prior to startup following the first refueling outage.	