UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-369

McGUIRE NUCLEAR STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 70
License No. NPF-9

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the McGuire Nuclear Station, Unit 1 (the facility) Facility Operating License No. NPF-9 filed by the Duke Power Company (the licensee) dated May 14, 1986, as revised or supplemented July 14, and November 21, 1986, and March 12, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-9 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 70, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

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Darl Hood, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment:
Technical Specification
Changes

Date of Issuance: April 10, 1987

PWR#4/DPWR-A	PWR#4/DPWR-A	OGC/BETH	PWR#4/DPWR-A
MDuncan:mac	DHood	MYoung	BJYoungblood
04/ /87	04/ /87	04/3 /87	04/ /87

UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

DUKE POWER COMPANY

DOCKET NO. 50-370

McGUIRE NUCLEAR STATION, UNIT 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 51
License No. NPF-17

1. The Nuclear Regulatory Commission (the Commission) has found that:

   A. The application for amendment to the McGuire Nuclear Station, Unit 2 (the facility) Facility Operating License No. NPF-17 filed by the Duke Power Company (the licensee) dated May 14, 1986, as revised or supplemented July 14, and November 21, 1986, and March 12, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations as set forth in 10 CFR Chapter I;

   B. The facility will operate in conformity with the application, as amended, the provisions of the Act, and the rules and regulations of the Commission;

   C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;

   D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and

   E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.

2. Accordingly, the license is hereby amended by page changes to the Technical Specifications as indicated in the attachments to this license amendment, and Paragraph 2.C.(2) of Facility Operating License No. NPF-17 is hereby amended to read as follows:

   (2) Technical Specifications

   The Technical Specifications contained in Appendix A, as revised through Amendment No.51, are hereby incorporated into the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.

3. This license amendment is effective as of its date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Darl Hood, Project Manager
PWR Project Directorate #4
Division of PWR Licensing-A

Attachment:
Technical Specification
Changes

Date of Issuance: April 10, 1987

PWR#4/DPWR-A	PWR#4/DPWR-A	OGC/BETH	PWR#4/DPWR-A
MDuncan:mac	DHood	MYoung	BJYoungblood
04/ /87	04/ /87	04/13 /87	04/ /87

ATTACHMENT TO LICENSE AMENDMENT NO. 70

FACILITY OPERATING LICENSE NO. NPF-9

DOCKET NO. 50-369

AND

TO LICENSE AMENDMENT NO. 51

FACILITY OPERATING LICENSE NO. NPF-17

DOCKET NO. 50-370

Replace the following pages of the Appendix "A" Technical Specifications with the enclosed pages. The revised pages are identified by Amendment number and contain vertical lines indicating the areas of change.

Amended Page
3/4 6-5
3/4 6-9a
3/4 6-29
3/4 6-30

TABLE 3.6-1

SECONDARY CONTAINMENT BYPASS LEAKAGE PATHS

PENETRATION NUMBER	SERVICE	RELEASE LOCATION	TEST TYPE
M216	Pressurizer Relief Tank Makeup	Auxiliary Building	Type C
M212	Nitrogen to Pressurizer Relief Tank	Auxiliary Building	Type C
M259	Reactor Makeup Water Tank to NV System	Auxiliary Building	Type C
M373	Ice Condenser Glycol In	Auxiliary Building	Type C
M372	Ice Condenser Glycol Out	Auxiliary Building	Type C
M330	Nitrogen to Accumulators	Auxiliary Building	Type C
M321	Safety Injection Test Line	Auxiliary Building	Type C
M348	Upper Head Injection Test Line# Post Accident Liquid Sample Discharge#	Auxiliary Building	Type C
M374	Containment Floor Sump Incore Instrument Sump Discharge	Auxiliary Building	Type C
M360	Reactor Coolant Drain Tank Gas Space to Waste Gas System	Auxiliary Building	Type C
M375	Reactor Coolant Drain Tank Heat Exchanger Discharge	Auxiliary Building	Type C
M356	Equipment Decontamination	Auxiliary Building	Type C
M235	Pressurizer Sample	Auxiliary Building	Type C
M309	Reactor Coolant Hot Leg Sample	Auxiliary Building	Type C
M322	Component Cooling to Component Cooling Drain Tank	Auxiliary Building	Type C

TABLE 3.6-1 (Continued)

SECONDARY CONTAINMENT BYPASS LEAKAGE PATHS

PENETRATION NUMBER	SERVICE	RELEASE LOCATION	TEST TYPE
---	Cont. Press. Monitor Narrow Range	Auxiliary Building	Type C
M354	Fuel Transfer Tube	Auxiliary Building	Type B

#Upon the deactivation of the Upper Head Injection System by removal of related components and piping and modifications to the Cold Leg Accumulators, this penetration is utilized for Post Accident Liquid Sample discharge.

TABLE 3.6-2 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	MAXIMUM ISOLATION TIME (SEC)
1. Phase "A" Isolation (continued)		
WL-1B	NCDT Pumps Discharge Outside Containment Isolation	≤10
WL-2A	NCDT Pumps Discharge Inside Containment Isolation	≤10
WL-39A	NCDT Vent Inside Containment Isolation	≤10
WL-41B	NCDT Vent Outside Containment Isolation	≤10
WL-64A	RB Sump Pump Discharge Inside Containment Isolation	≤15
WL-65B	RB Sump Pump Discharge Outside Containment Isolation	≤15
WL-321A	Containment Vent Unit Drains Inside Containment Isolation	≤15
WL-322B	Containment Vent Unit Drains Outside Containment Isolation	≤15
WL-1301B##	PALS Discharge Outside Containment Isolation	≤15
WL-1302A##	PALS Discharge Inside Containment Isolation	≤15
YM-115B	Demin. Water Containment Outside Isolation	≤15
2. Phase "B" Isolation		
KC-338B	NC Pump Supply Header Pent. Isolation (outside)	≤40
KC-424B	NC Pumps Return Hdr. Pent Inside Isolation	≤40
KC-425A	NC Pumps Return Hdr. Outside Isolation	≤40
RN-252B	Nonessential Supply to RB Penetration Outside Isolation	≤30
RN-253A	Nonessential Supply to RB Penetration Inside Isolation	≤30
RN-276A	Nonessential Return to RB Penetration Inside Isolation	≤30
RN-277B	Nonessential Return to RB Penetration Outside Isolation	≤30
RV-32A	Lower Containment Vent. Unit Supply Containment Isolation (outside)	≤60
RV-33B	Lower Containment Vent. Unit Supply Containment Isolation (inside)	≤60
RV-76A	Lower Containment Vent. Unit Discharge Containment Isolation (inside)	≤60
RV-77B	Lower Containment Vent. Unit Discharge Containment Isolation (outside)	≤60
VI-129B	"A" Header Containment Outside Isolation	≤15
VI-150B	Instrument Air Lower Containment Outside Isolation	≤15
VI-160B	"B" Header Containment Outside Isolation	≤15

TABLE 3.6-2 (Continued)

CONTAINMENT ISOLATION VALVES

VALVE NUMBER	FUNCTION	MAXIMUM ISOLATION TIME (SEC)
2. Phase "B" Isolation (continued)		
RV-79A	Upper Containment Vent. Unit Supply Containment Isolation Outside	≤30
RV-80B	Upper Containment Vent. Unit Supply Containment Isolation Inside	≤30
RV-101A	Upper Containment Vent. Unit Discharge Containment Isolation Inside	≤30
RV-102B	Upper Containment Vent. Unit Discharge Containment Isolation Outside	≤30
3. Main Steam Isolation		
SM-1AB#	Main Steam D Isolation	≤5
SM-3AB#	Main Steam C Isolation	≤5
SM-5AB#	Main Steam B Isolation	≤5
SM-7AB#	Main Steam A Isolation	≤5
SM-9AB#	Main Steam D Isolation Bypass Ctrl.	≤5
SM-10AB#	Main Steam C Isolation Bypass Ctrl.	≤5
SM-11AB#	Main Steam B Isolation Bypass Ctrl.	≤5
SM-12AB#	Main Steam A Isolation Bypass Ctrl.	≤5
4. Manual		
NC141*	NC Pump Motor Oil Drain	N.A.
NC142*	NC Pump Motor Oil Drain	N.A.
WE13*	Equipment Decontamination	N.A.
WE23*	Equipment Decontamination	N.A.
VX34*	Containment H_2 Sample	N.A.
VX40*	Containment H_2 Sample	N.A.
FW11*	Refueling Water	N.A.
FW13*	Refueling Water	N.A.
FW4*	Refueling Water	N.A.

*May be opened on an intermittent basis under administrative control.

**Valve also receives a High Radiation (H) isolation signal.

#Not subject to Type C leakage tests.

##Upon the deactivation of the Upper Head Injection System by removal of related components and piping and modifications to the Cold Leg Accumulators, this valve is utilized to isolate Post Accident Liquid Sample discharge.

NOTE: Times are for valve operation only, and do not include any sensor response or circuit delay times. See Specification 3/4.3.2 for system actuation response times.