

GPU NUCLEAR CORPORATION  
OYSTER CREEK NUCLEAR GENERATING STATION

Provisional Operating  
License No. DPR-16

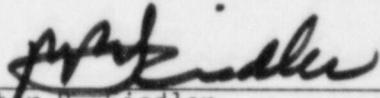
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Technical Specification  
Change Request No. 151  
Docket No. 50-219

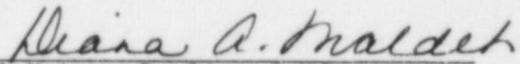
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Applicant submits, by this Technical Specification Change Request No. 158 to the Oyster Creek Nuclear Generating Station Technical Specifications, a change to page 3.3-2.

By

  
Peter B. Fiedler  
Vice President and Director  
Oyster Creek

Sworn and Subscribed to before me this 17<sup>th</sup> day of March 1987.

  
A Notary Public of NJ  
Expiration 6/5/91

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UNITED STATES OF AMERICA  
NUCLEAR REGULATORY COMMISSION

In the matter of  
GPU Nuclear Corporation

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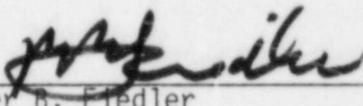
Docket No. 50-219

CERTIFICATE OF SERVICE

This is to certify that a copy of Technical Specification Change Request No. 158 for Oyster Creek Nuclear Generating Station Technical Specifications, filed with the U.S. Nuclear Regulatory Commission on March 17, 1987, has this day of March 17, 1987, been served on the Mayor of Lacey Township, Ocean County, New Jersey by deposit in the United States mail, addressed as follows:

The Honorable Christopher Connors  
Mayor of Lacey Township  
818 West Lacey Road  
Forked River, NJ 08731

By

  
Peter B. Fiedler  
Vice President and Director  
Oyster Creek

OYSTER CREEK NUCLEAR GENERATING STATION  
PROVISIONAL OPERATING LICENSE NO. DPR-16  
DOCKET NO. 50-219  
TECHNICAL SPECIFICATION CHANGE REQUEST NO. 158

Applicant hereby requests the Commission to change Appendix A to the above captioned license as below, and pursuant to 10CFR50.91, an analysis concerning the determination of no significant hazards considerations is also presented:

1. Section to be Changed

Section 3.3

2. Extent of Change

Revise Section 3.3.D.1.c to limit the unidentified leakage for the reactor coolant system to a maximum leak rate increase of 2 gpm within any 24 hour period while operating at steady state power.

3. Changes Requested

The requested change is shown on attached Technical Specification page 3.3-2.

4. Discussion

By a letter dated September 5, 1986, the NRC requested GPU Nuclear (GPUN), the licensee for the Oyster Creek Nuclear Generating Station (OCNGS) to justify its Technical Specification (TS) requirements (TS3.3D.1.c) for unidentified leakage against the requirements of Generic Letter (GL) 84-11, attachment 1, item B. Briefly, OCNGS TS3.3D.1.c states that reactor coolant system leakage shall be limited to a 2 gpm increase in unidentified leakage rate within any 4 hour period while operating at steady state power. GL 84-11 states that a maximum increase in unidentified leakage of 2 gpm within any 24 hour period while operating would be sufficiently restrictive to ensure timely investigation of potential through-wall cracks in austenitic stainless steel piping.

GPUN agrees that the OCNGS TS3.3D.1.c for the unidentified leakage is not as restrictive as the requirements of GL 84-11 and is proposing this amendment to revise the time interval from 4 hours to 24 hours. Since this proposed amendment increases the time interval for the same flow rate (2 gpm), this change places a more restrictive requirement on the maximum allowable rate of increase for the unidentified reactor coolant system leakage.

This proposed change would increase the margin of safety through more restrictive limiting conditions of operation for the unidentified reactor coolant system leakage.

Likewise, the more restrictive limiting conditions of operation would provide for the timely detection of potential cracks, and would decrease the probability of a design basis loss-of-coolant accident.

5. Determination

Based upon the hereinbefore discussion, we have evaluated that this change request involves no significant hazards considerations. In summary, we have determined that the proposed amendment would not:

- a. Involve a significant increase in the probability or consequences of an accident previously evaluated;

This proposed change would place more restrictive limiting conditions of operation for the unidentified reactor coolant system leakage which provides for the timely detection of potential through-wall cracks in austenitic stainless steel piping. By this timely detection the probability of a design basis loss-of-coolant accident would decrease. This proposed change would not have any effect on the consequences of the design bases accidents previously evaluated.

- b. Create the probability of a new or different kind of accident from any accident previously evaluated;

This proposed change only places more restrictive limiting conditions of operation for the unidentified reactor coolant system leakage, so this proposed change does not create the probability of a new or different kind of accident.

- c. Involve a significant reduction in a margin of safety;

This proposed change would increase the margin of safety by the initiation of timely actions for the detection of potential through-wall cracks in austenitic stainless steel piping, and thereby reduces the probability of a design basis loss-of-coolant accident.