

## LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Brunswick Steam Electric Plant Unit 1										DOCKET NUMBER (2) 0 5 0 0 0 3 2 5										PAGE (3) 1 OF 0 1																													
TITLE (4) Inadequate Channel Functional Testing of Reactor Building Ventilation Exhaust Radiation Monitors																																																	
EVENT DATE (5)										LER NUMBER (6)										REPORT DATE (7)										OTHER FACILITIES INVOLVED (8)																			
MONTH			DAY			YEAR			YEAR			SEQUENTIAL NUMBER			REVISION NUMBER			MONTH			DAY			YEAR			FACILITY NAMES Brunswick Unit 2										DOCKET NUMBER(S) 0 5 0 0 0 3 2 4												
0 3			1 8			8 5			8 5			0 1			0 0			0 0			4 1			6 8			5			0 5 0 0 0																			
OPERATING MODE (9) 1										THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following.) (11)																																							
POWER LEVEL (10) 0 7 0										20.402(b)										20.406(c)										50.73(a)(2)(iv)										72.71(b)									
										20.406(a)(1)(i)										50.36(c)(1)										50.73(a)(2)(v)										73.71(e)									
										20.406(a)(1)(ii)										50.36(c)(2)										50.73(a)(2)(vii)										OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
										20.406(a)(1)(iii)										X 50.73(a)(2)(i)										50.73(a)(2)(viii)(A)																			
										20.406(a)(1)(iv)										50.73(a)(2)(ii)										50.73(a)(2)(viii)(B)																			
										20.406(a)(1)(v)										50.73(a)(2)(iii)										50.73(a)(2)(ix)																			
LICENSEE CONTACT FOR THIS LER (12)																																																	
NAME M. J. Pastva, Jr., Regulatory Technician																				TELEPHONE NUMBER 9 1 1 9 4 5 7 - 2 3 1 5																													
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																																																	
CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC			CAUSE			SYSTEM			COMPONENT			MANUFACTURER			REPORTABLE TO NRC																						
SUPPLEMENTAL REPORT EXPECTED (14)																																																	
YES (If yes, complete EXPECTED SUBMISSION DATE)																				X NO										EXPECTED SUBMISSION DATE (15)																			
																														MONTH DAY YEAR																			

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 3-18-85 at 2325, the Units 1 and 2 Reactor Buildings' ventilation exhaust radiation monitors, 1(2)-D12-RM-N010A and B, were declared inoperable per Technical Specifications (T/S) Table 4.3.2-1 following a determination that their monthly operability test, periodic test PT-04.1.1, did not adequately perform a functional test of their radiation high trip setpoint. Unit 1 was operating at 70%; Unit 2 was in cold shutdown.

The PT did not functionally verify that the electrical contacts of the high radiation trip relay, K2, open on a test input high radiation signal. Opening of the K2 contacts deenergizes the K82 relay to initiate a primary containment Group 6 isolation, automatic isolation of the Reactor Building Ventilation System, and automatic starting of the Reactor Building Standby Gas Treatment System. The PT placed the monitor mode switch to the "trip test" position which caused electrical contacts in series with the K2 contacts to open and deenergize the K82 relay. When a test input signal was applied to the K2 relay logic circuitry, the K82 relay was already deenergized due to the prior positioning of the mode switch. Therefore, the instrument channel trip function, which is to deenergize the K82 relay, was not adequately verified.

Similar problems have been addressed in CP&L responses to IERs 84-30 and 84-31 and in LERs 1-84-015 and 1-85-005. Proper operation of the monitors' K2 relay contacts was verified on 3-19-85 on both units and the monitors were returned to service. PT-04.1.1 is being revised to satisfy testing requirements.

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Carolina Power & Light Company

Brunswick Steam Electric Plant  
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Southport, NC 28461-0429

April 16, 1985

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NRC Document Control Desk  
U.S. Nuclear Regulatory Commission  
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT UNIT 1  
DOCKET NO. 50-325  
LICENSE NO. DPR-71  
LICENSEE EVENT REPORT 1-85-010

Gentlemen:

In accordance with Title 10 to the Code of Federal Regulations, the enclosed Licensee Event Report is submitted. This report fulfills the requirement for a written report within thirty (30) days of a reportable occurrence and is in accordance with the format set forth in NUREG-1022, September 1983.

Very truly yours,

C. R. Dietz, General Manager  
Brunswick Steam Electric Plant

MJP/shb

Enclosure

cc: Dr. J. N. Grace

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