

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Waterford 3 Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 1 8 2	PAGE (3) 1 OF 0 1 3
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TITLE (4)
Inadvertent Actuation of Reactor Protection System

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0	3	1 4 8 5	8 5	0 0 7		0	0	4 1 1 8 5	N/A		0 5 0 0 0
									N/A		0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9) 3	20.402(b)	20.406(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 0 1 0 1 0	20.406(a)(1)(i)	50.36(c)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	73.71(c)
	20.406(a)(1)(ii)	50.36(c)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.406(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME O.D. Hayes, Operations Superintendent	TELEPHONE NUMBER 5 0 4 4 6 4 - 3 1 1 8
AREA CODE	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH DAY YEAR
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

At 1016 hours Central Standard Time on March 14, 1985, while performing a Reactor Shutdown, Waterford 3 Steam Electric Station experienced an inadvertent actuation of the Reactor Protection System due to noise in the Core Protection Calculator channels C and D. Waterford 3 was in mode 3 at the time of the event.

This event was reported to the Commission pursuant to 10CFR50.72(b)(2)(ii).

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

NARRATIVE

At 1016 hours Central Standard Time on March 14, 1985 Waterford 3 Steam Electric Station was in mode 3 when an inadvertent actuation of the Reactor Protection System occurred due to a LOW DNBR and a HIGH LOCAL POWER DENSITY trip in channels C and D of the Core Protection Calculators. At the time of the event all Regulating Groups, Part-Length, and Shutdown Bank B Control Element Assemblies were fully inserted, and Operations Personnel were commencing the insertion of Shutdown Bank A.

Upon insertion of the Part-Length Control Element Assemblies the LOW DNBR and HIGH LOCAL POWER DENSITY trips came in for all four channels of the Core Protection Calculators. However, the trips are manually bypassed by Operations Personnel when reactor power decreases below 1.0E-4 percent, as defined in procedure OP-10-001, General Plant Operations.

Electrical noise within the Excore Nuclear Instrumentation caused channels C and D of the Core Protection Calculators to momentarily spike above 1.0E-4 percent power. This in turn removed the 1.0E-4 percent bypass, per design, for the above channels. When the bypass for channels C and D was removed, the two-out-of-four trip logic was completed.

SAFETY CONSEQUENCES AND IMPLICATIONS

The above event resulted in an inadvertent actuation of the Reactor Protection System during Phase III Testing in which no primary system parameters were exceeded. Since the Control Element Assemblies and the Reactor Protection System functioned as designed, the subject event in no way placed Waterford 3 in a degraded safety condition.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

CORRECTIVE ACTIONS

The above actuation was caused by a spurious electrical spike within the Excore Nuclear Instrumentation. Spikes such as these have not been a problem in the past and have not reoccurred since the above event. However, further investigations will be undertaken should this event reoccur.

SIMILAR EVENTS

Licensee Event Report, LER-85-004, reported a reactor trip.

PLANT CONTACT

O. D. Hayes, Operations Superintendent, 504/464-3118



LOUISIANA
POWER & LIGHT

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April 11, 1985

W3P85-1215
A4.05

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-007-00 for the Waterford 3 Steam Electric Station. This Licensee Event Report is submitted per 10CFR50.73(a)(2)(iv).

Very truly yours,

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, D.M. Crutchfield, NRC Resident Inspectors
Office, INPO Records Center (J.T. Wheelock), E.L. Blake,
W.M. Stevenson

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