٠, . Update Report - Previous Report Date 6/24/83 NRC CORM 366 (12-81)-10 CFR 50 APPROVED BY OMB U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT 3150-0011 CONTROL BLOCK (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION) ASES120 0 1 P 0 0 0 0 0 - 0 0 3 4 1 1 1 1 00 CON'T REPORT L 6 0 5 0 0 0 3 8 7 7 0 5 2 7 8 3 8 SOURCE 10 DOCKET NUMBER 59 59 EVENT DATE 74 75 0 1 (9) EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10) While at 40% power, the 'A' Containment H2/02 Analyzers were found to have erratic 0 2 0 3 readings. The analyzers were declared inoperable in accordance with Technical Specification 3.3.7.5. Accident Monitoring Instrumentation. There were no adverse 0 4 consequences in that the 'B' Analyzers were available and operating properly. 0 5 This event is similar to LER 83-053. 0 6 0 7 0 8 80 SYSTEM CAUSE CAUSE COMP VALVE CODE COMPONENT CODE SUBCODE SUBCODE 15 (11) | E | (12) C Z (16) B (13) 0 9 SE PUMPXX14 19 OCCURRENCE SEQUENTIAL REPORT NO. REPORT REVISION NO CODE LER/RO REPORT REPORT 8 (17) 13 0 8 01 1 31 28 22 ATTACHMENT EFFECT ON PLANT ACTION FUTURE TAKEN ACTION SHUTDOWN PRIME COMP ER(26) HOURS FORM SUB. METHOD SUPPLIER LY 23 N 24 Z 21 LA 25 20 A (18 X (19 Z 0 0 0 D 0 9 10 6 CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27) Upon inspection of the 'A' analyzer components, the pressure control valve and 1 0 sample pump were found to be degraded to a point where proper flow could not be 1 1 maintained. The failed components were replaced and the system calibrated and declared operable. An engineering review has been tasked with determining the 3 4 long term resolution. 1 METHOD OF FACILIT (30) OTHER STATUS DISCOVERY DESCRIPTION (32) STATUS A 3 Operator Observation 5 B 28 0 4 0 29 1 NA A 9 10 ACTIVITY CONTENT RELEASED OF RELEASE 13 44 80 12 45 AMOUNT OF ACTIVITY (35) LOCATION OF RELEASE (36) Z 33 Z 34 1 6 NA NA 80 10 11 PERBONNEL EXPOSURES NUMBER TYPE DESCRIPTION (39) 8608080235 860804 ADOCK 05000387 NA PDR PDR 80 PERSONNEL INJURIES NUMBER 0 0 0 0 0 NA 1 8 11 80 12 TEZZ LOSS OF OR DAMAGE TO FACILITY Z (42) NA 9 1 PUBLICITY ISSUED DESCRIPTION (45) NRC USE ONLY LN (4) 2 0 NA NAME OF PREPARER L.A. Kuczynski (717) 542-3759 PHONE

## ATTACHMENT

## LER # 83-087/03X-1

## Pennsylvania Power & Light Company Susquehanna Steam Electric Station Docket Number: 50-387

While at 40% power, the 'A' Containment  $H_2/O_2$  analyzers were found to have erratic readings. The analyzers were declared inoperable in accordance with Technical Specification 3.3.7.5, Accident Monitoring Instrumentation. There were no adverse consequences in that the "B" analyzers were operable and operated properly. This event is similar to LER 83-053, which was written about the "B" containment  $H_2/O_2$  analyzers.

The suspect components, the pressure control value and sample pump, were inspected and found to be degraded to a point where proper flow could not be maintained. The value and pump were replaced and the "A" Containment  $H_2/O_2$  analyzers were returned to service and declared operable.

It is believed that previous operation of the  $H_2/O_2$  and/yzers in a nonierted containment during the Power Ascension testing program caused accelerated degradation of the system components. With the higher than normal concentratin of oxygen in the containment, abnormal amounts of moisture were present in the system. This coupled with the amount of outage activities in the containment (grinding, welding, insulation work, etc.) allowed quantities of particulate matter to accumulate in various components and restricted flow.

Sample pumps have been sent to Comsip-Delphi for failure analysis. As a result of the valve disc/valve body valve guide wear noted in their analysis, Comsip developed a new valve disc design which provides for more bearing surface on the valve disc. The Nuclear Plant Engineering group has been continuing their investigation and evaluation of the life expectancy of the  $H_2/0_2$  analyzers sample pumps and other system components. The questin of equipment design versus equipment application is also a part of the engineering review.

The method of operation has been changed to keep one channel of analyzers in the standby mode while the other channel is kept in continuous analyze. Until a permanent fix is developed, system malfunctions will continue to be corrected as they arise. Since the problems associated with the continuous operation of the  $H_2/O_2$  analyzers are intrinsic and this topic is presently tracked as an Inspection Report Unresolved Item (Reference: Inspection Report 50-387/UNR 85-01-02) no further updates to this LER are planned. Future problems with the analyzers will be reported when they meet the requirements of 10 CFR 50.73.



## Pennsylvania Power & Light Company

Susquehanna Steam Electric Station P.O. Box 467 • Berwick, PA 18603 • 717 / 542-2181

August 4, 1986

U.S. Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

SYSQUEHANNA STEAM ELECTRIC STATION LICENSEE EVENT REPORT 83-087/03X-1 FILE R41-2 PLA- 190

Docket No. 50-387 License No. NPF-14

Attached please find a copy of Licensee Event Report No. 83-087/03X-1. This event was determined to be reportable per Technical Specification 6.9.1.9.b, in that the 'A' channel of Containment Hydrogen/Oxygen analyzers was found to have erratic readings and declared inoperable. This event is similar to LER 83-053 and an engineering review is in progress to determine proper system design versus application.

T.M. Crimmins, Jr. Superintendent of Plant-Susquehanna

LAK/cdn

cc: Dr. Thomas E. Murley Regional Administrator, Region I U.S. Nuclear Regulatory Commission 631 Park Avenue King of Prussia, PA 19406

> Mr. Loren Plisco Resident Inspector U.S. Nuclear Regulatory Commission P.O. Box 52 Shickshinny, PA 18655

