

3 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE JO 57 CAT 58 59

REPORT SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

## EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

On 3/24/83 at 1120 hrs., the Shift Control Room Engineer was informed that a Local Leak Rate Test on the C.R.D. Removal Hatch Double Gasket Seals had been incorrectly performed prior to startup on the same day. At the time of the occurrence the reactor was in the startup mode with only one C.R.D. withdrawn. The leakage found when the test was performed properly at 39.6 psig was 0.0 scfh. The health and safety of the public was preserved at all times.

SYSTEM CODE		CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE				COMP. SUBCODE	VALVE SUBCODE
R	B	A	X	Z	Z	Z	Z	Z	Z
9	10	11	12	13	14	15	16	17	18
EVENT YEAR			SEQUENTIAL REPORT NO.			OCCURRENCE CODE		REPORT TYPE	
8	3		0	3	0		0	3	L
21	22	23	24	25	26	27	28	29	30
ACTION TAKEN		FUTURE ACTION		EFFECT ON PLANT		SHUTDOWN METHOD	HOURS	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.
E	G	Z	Z	0	0	0	0	Y	N
33	34	35	36	37	38	39	40	41	42
CAUSE DESCRIPTION AND CORRECTIVE ACTIONS				PRIME COMP. SUPPLIER		COMPONENT MANUFACTURER			
				Z	Z	9	9	9	9
				43	44	45	46	47	48

## CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

The cause of the occurrence was attributed to personnel error on the part of the engineer performing the test. The leak rate test was re-performed expeditiously and completed at 11150 hours on 3/24/83.

FACILITY STATUS		% POWER		OTHER STATUS		METHOD OF DISCOVERY		DISCOVERY DESCRIPTION	
5	B	28	0	0	0	29	C	31	Leak Rate Test
ACTIVITY CONTENT		AMOUNT OF ACTIVITY		LOCATION OF RELEASE					
6	Z	33	Z	34	NA	35	NA	36	
PERSONNEL EXPOSURES		TYPE		DESCRIPTION					
7	0	0	0	37	Z	38	NA	39	
PERSONNEL INJURIES		TYPE		DESCRIPTION					
8	0	0	0	40			NA	41	
LOSS OF OR DAMAGE TO FACILITY		TYPE		DESCRIPTION					
9	Z	42			NA	43	8305020090 830421 PDR ADOCK 05000373 S PDR	44	
PUBLICATION		TYPE		DESCRIPTION					
10	N	45			NA	46		47	

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NAME OF PREPARER

R. W. Houston

PHONE: (815) 357-6761

- I. LER NUMBER: 83-030/03L-0
- II. LASALLE COUNTY STATION: Unit 1
- III. DOCKET NUMBER: 050-373
- IV. EVENT DESCRIPTION:

On March 24, 1983, at 1120 hours the Unit 1 Station Control Room Engineer was informed that a Local Leak Rate Test on the C.R.D. Removal Hatch Double Gasket Seals had been incorrectly performed prior to startup on the same day. The test pressure used was 10 psig with a resultant 0.0 scfh leakage, while the proper test pressure is 39.6 psig.

- V. PROBABLE CONSEQUENCES OF THE OCCURRENCE:

At the time of the occurrence, the reactor was in the Startup Mode with only one CRD withdrawn. The leakage found when the test was performed properly at 39.6 psig was 0.0 scfh. The health and safety of the public was preserved at all times.

- VI. CAUSE:

The cause of the occurrence was attributed to personnel error on the part of the engineer performing the test. A test on the Personnel Access Doors Double Gasket Seals had been performed just prior to this test. The required test pressure for these seals is 10 psig.

- VII. CORRECTIVE ACTION:

The leak rate test was re-performed expeditiously and completed at 1150 hours on 3/24/83.

A new procedure, LTS 100-8, has been initiated to outline steps necessary to perform a leak rate test on the Personnel Access Doors Double Gasket Seals. This is the only leak test that requires a test pressure of 10 psig, hence, confusion associated with proper test pressures will be eliminated.

Prepared by: R. W. Houston