

March 12, 1999

LICENSEES: Saxton Nuclear Experimental Corporation (SNEC) and
GPU Nuclear Corporation (GPUN)

FACILITY: Saxton Nuclear Experimental Facility (SNEF)

SUBJECT: SUMMARY OF MEETING BETWEEN SNEC, GPUN AND THE NRC STAFF

On March 11, 1999, representatives of the NRC staff met at NRC headquarters with representatives of SNEC and GPUN, the licensees for the SNEF. Enclosure one is a list of meeting attendees. Enclosure two is the briefing material provided by the licensees at the meeting.

The purpose of the meeting was to present an overview of the Saxton decommissioning project and the recently submitted license termination plan (LTP). The licensees gave a brief description of the operational and licensing history of the facility. The licensees then presented an overview of their LTP and their final status survey including release criteria, final status survey methodology, determination of derived concentration guideline levels, quality assurance, training, conduct of the final status survey and reporting of final status survey results. The NRC staff commented that the first step in the NRC review of the LTP would be an acceptance review to determine if all required topics were discussed in the licensees' submittal.

In response to a question from the NRC staff, the licensees discussed the issue of buoyancy of the containment vessel (CV). The water table at the SNEF site is near the surface. The CV extends about 50 feet below ground level. There is a concern that as the weight of the CV is reduced by removal of contaminated and activated material, the CV may become buoyant and shift in position, raising structural stability concerns. The licensees are planning to drill soil cores near the CV to determine soil properties. Friction between the CV and soil could increase significantly the amount of material that could be removed from the CV without buoyancy becoming an issue. The licensees also plan to study the internal structure of the CV to understand the effect of the polar crane on the CV structure and the effect of removing concrete on the internal structural stability of the CV.

Docket No. 50-146
Enclosures: As stated
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ORIGINAL SIGNED BY:

Alexander Adams, Jr., Senior Project Manager
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Project Directorate

Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

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SCollins/RZimmerman (SJC1/RPZ) CBassett (CHB1)
BBoger (BAB2) TBurdick (TMB)
DMatthews (DBM) PDoyle (PVD)
SWeiss (SHW) TDragoun (TFD)
TMartin (SLM3) Weresian (EJW)
LPittiglio (CLP) SHolmes (SWH)
BMCabe (BCM) MMendonca (MMM)
TMichaels (TSM1)

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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A handwritten signature in cursive script, reading "Alexander Adams, Jr.", is positioned above the typed name.

Alexander Adams, Jr., Senior Project Manager
Non-Power Reactors and Decommissioning
Project Directorate
Division of Regulatory Improvement Programs
Office of Nuclear Reactor Regulation

Cocket No. 50-146

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Division of Regulatory Improvement Programs
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Saxton Nuclear
Experimental Corporation

Docket No. 50-146
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Saxton Nuclear
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Docket No. 50-146
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MEETING BETWEEN THE NRC STAFF AND SAXTON

March 11, 1999

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MEETING BETWEEN THE NRC STAFF AND SAXTON

March 11, 1999

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MEETING BETWEEN THE NRC STAFF AND SAXTON

March 11, 1999

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TITLE
ORGANIZATION

PHONE

WILLIAM HEYSEK

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Saxton Nuclear Experimental
Corporation (SNEC)
Facility Site
License Termination Plan
Meeting

USNRC
Rockville MD
March 11, 1999

SNEC License Termination Plan

- SNEC Overview:
 - SNEC Organization
 - Transition Information
 - Citizens Task Force
 - Saxton Independent Inspector Program

SNEC License Termination Plan

- SNEC Overview:
 - Decommissioning Funding:
 - Schedule:
 - Concrete Removal: till Spring 2000
 - Site Remediation: Spring- Summer 2000
 - Final Survey: Fall - Winter 2000/01
 - License Termination: Spring 2001

SNEC License Termination Plan

- Brief History:
 - Westinghouse PWR Design
 - 23.5 MWTh/ ~ 7 MWE
 - Single Loop
 - Construction Authorization February 1960
 - Initial Critically April 1962

SNEC License Termination Plan

- Brief History:

- Operational Period 1962 - 1972

Three Fuel cycles
Mixed Oxides Fuels
Failed Fuel Operation
Unplanned Releases

SNEC License Termination Plan

- Brief History:

- Final Shutdown May 1972
- Spent Fuel Removed 1972
- Placed in "SAFSTOR" February 1975

SNEC License Termination Plan

- Brief History:
 - Phased Remediation:
 - Decontamination and Demolition of Support Buildings (1986-1989)
 - Soil Remediation Project (1994)
 - CV Characterization (1995-)
 - Asbestos Abatement (1996-1997)
 - Electrical System Modifications (1996-1997)

SNEC License Termination Plan

- Brief History:
 - Decommissioning T/S Approved April 1998
 - Major Decommissioning Activities:
 - CV Ventilation System
 - System Dismantlement
 - Large Component Removal
 - Demins Disposal
 - Shield Plug / Concrete Removal

License Termination Plan

- Covers Issues in the Standard Review Plan (NUREG-1700):
 - Site Characterization
 - Remaining Dismantlement Activities
 - Site Remediation Plans

License Termination Plan

- Covers Issues in the Standard Review Plan (NUREG-1700):
 - End use of site
 - Project Funding
 - Environmental Changes
 - Final Status Survey Plan

License Termination Plan

- LTP process:
 - The LTP has been submitted as a supplement to the SNEC Facility Updated Safety Analysis Report (USAR)

Final Status Survey

- Purpose:
 - Demonstrates compliance to the NRC that the SNEC Facility Site is below the unrestricted radiological release criteria.

Final Status Survey

- Site Specific Release Criteria:
 - Below USNRC Unrestricted Site Release Criteria (10 CFR Part 20). Less than 25 mrem per year (TEDE) to the average member of the critical group
 - Below the EPA National Primary Drinking Water Standard for Radioactivity (40 CFR Part 141) Less than 4 mrem per year

Final Status Survey

- Site Specific Release Criteria:
 - Exposure rates from gamma emitters:
 - 5 micro- rem per hour @ 1 meter (average)
 - 10 micro- rem per hour (maximum)
 - Below the applicable Derived Concentration Guideline Levels (DCGL) values for soil, concrete and other structural materials.

Final Status Survey

- FSS Methodology:
 - Taken mainly from MARSSIM (NUREG-1575)
 - Using a form of the Data Quality Objectives (DQO) Process
 - Seven steps per MARSSIM
 - Use to plan the FFS process

Final Status Survey

- FSS Methodology:
 - Impacted:
 - Areas that have the potential for radioactive contamination.
 - Impacted areas are further divided into three classifications.
 - Initial screening indicates possibly as much as 29 acres

Final Status Survey

- FSS Methodology:

- Class 1 Areas:

- Areas or locations that have a potential for radioactive contamination based on knowledge of site operating history, of known contamination levels or previous radiological surveys.

Final Status Survey

- FSS Methodology:

- Class 2 Areas:

- These areas have or had a potential for radioactive contamination that is not expected to exceed the DCGI_w.

Final Status Survey

- FSS Methodology:
 - Class 3 Areas:
Any impacted areas that are not expected to contain any residual radioactivity, or are expected to contain levels of residual radioactivity at a very small fraction of the $DCGL_W$.

Final Status Survey

- FSS Methodology:
 - Non-impacted:
Areas that have no reasonable potential for residual contamination
 - Background reference areas will be selected from non-impacted areas

Final Status Survey

- Contamination Identification:
 - Based on continued site characterization process
 - Radiological surveys
 - Historical Site Assessment (HSA)

Final Status Survey

- Determination of Derived Concentration Guideline Levels (DCGL)
 - Current work in progress
 - Establishing background level
 - Establishing Site Modeling

Final Status Survey

- Quality Assurance:
 - Instrumentation Selection Calibration and operation
 - Survey Documentation
 - Quality Control Surveys (internal)
 - Written Procedures

Final Status Survey

- Quality Assurance:
 - Chain of Custody
 - Records Management
 - Access Controls
 - Control of Vendor Supplied Services
 - Independent Review of Survey Results

Final Status Survey

- Training:
 - Overview and Objectives
 - Procedures
 - Instrumentation
 - Sample Collection
 - Documentation

Final Status Survey

- Final Survey Phases:
 - CV Structure
 - Remaining Structures
 - All On-site areas
 - Remaining Impacted Areas

Final Status Survey

- Area Turnover for FSS:
 - Decommissioning activities that have the potential to re-contaminate the survey area must be completed
 - Housekeeping effort completed

Final Status Survey

- Turnover for FSS:
 - Final Remediation support surveys are completed and may consist of:
 - Scan surveys
 - Smear (loose surface) surveys
 - Access control measures in place

Final Status Survey

- Access Control Measures (one or more of the following):
 - Site personnel awareness
 - Barriers
 - Postings
 - Locking entrances

Final Status Survey

- FFS Performed:
 - Basically per MARSSIM
 - Per FSS Table 4-4

Final Status Survey

- Final Survey Results Report:

- Consist of:

- Detailed and summary data reporting for each survey unit.

- Final TEDE Evaluation

- Independently Verified

SNEC License Termination Plan

- Conclusions:

- No On-site spent fuel

- Decommissioning Costs are covered

- Unrestricted Release Criteria and Drinking Water Limits)

SNEC License Termination Plan

- Conclusions:
 - MARSSIM Methodology
 - LTP Ongoing Process
 - License Termination: Spring 2001