U.S. NUCLEAR REGULATORY COMMISSION IRC Form 366 APPROVED OMB NO. 3150-0104 EXPIRES 8-31/48 LICENSEE EVENT REPORT (LER) DOCKET NUMBER (2) FACILITY NAME (1) 0 | 5 | 0 | 0 | 0 | 3 | 2 | 7 | 1 OF 0 | 4 Sequoyah, Unit 1 "A' Relay Deenergized When Bumped During Modification Work Resulting In An Auxiliary Building Isolation Train 'B' Actuation OTHER FACILITIES INVOLVED (8) LER NUMBER (6) REPORT DATE 17 VOILITY NAMES DOCKET NUMBERS SEQUENTIAL NUMBER YEAR MOSTH DAY YEAR MONTH YEAR 0 | 5 | 0 | 0 | 0 | 3 | 2 | 8 Sequoyah, Unit 2 101488 0 | 5 | 0 | 0 | 0 | 0 0 0 9 2 9 3 8 8 8 0 3 TO THE REQUIREMENTS OF 10 CFR & (Check one or more of the following) (11 THIS REPORT IS SUBMITTED PURSUANT MODE IS 73.71(b) 20.4021b1 20 408 (c) 80 73(a)(2)(iv) 86.73(a)(2)(v) 73,73(e) 20.406(41(1))) 50 38(e)(1) OTHER (Specify in Addrect below and in Text, NRC Form 366A) 0 010 20.405(4)(1)(0) 50.73(e)(2)(vii) 60.38(c)(2) 85 750 (1911) (III) 20.406(*)(*)(*)(!!) 80.73(4)(2)(1) 20.408 (a) (1) (iv) 60.75(a)(2)(ii) 86.73(a) (2) (viii1/8) 50: A08:(41/11/11 50.73 W1(21(0)) 50.73(4)(2)(4) LICENSEE CONTACT FOR THIS LER (12) ELEPHONE NUMBER AREA CODE 8 | 7 | 0 | - | 7 | 5 | 8 | 5 6 1 1 5 T. K. Phifer, Plant Reporting Section COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) TO NAMES TO NERDS MANUFAC MANUFAC TURER CAUSE EVETEM COMPONENT CAUSE SYSTEM COMPONENT SUPPLEMENTAL REPORT EXPECTED (14) EXPECTED SUBMISSION DATE (15)

At 1359 EDT, on September 29, 1988, with unit 1 in mode 4 (0 percent power, 355 psig, and 246 degrees F) and unit 2 in mode 1 (65 percent power, 2235 psig, and 565 degrees P), a train 'B' Auxiliary Building Isolation (ABI) (EIIS code VF) occurred. The Auxiliary Building (AB) is a common building for units 1 and 2 and is the secondary containment enclosure for both units.

Defore this event, modifications personnel were performing work inside O-M-12 under Work Plan (WP)-11266 R1. WP 11266 R1 was generated to implement a design change which installs a seal-in circuit for an ABI caused by a high radiation condition in the AB. The cause of this event has been investigated, and it has been concluded that while performing the work inside O-M-12, an electrician inadvertently bumped a time delay relay (TDR) for O-RM-90-103. O-M-90-103 is the train B spent fuel pool area radiation monitor. Two recommendations from the investigation of this event were made to prevent recurrence of this type of event. One was to require the appropriate monitors to be blocked when work is performed inside a main control room panel in O-M-12 and the second was to install a protective cover for appropriate TDRs. TVA will evaluate both options and revise this report by December 31, 1988 to provide the course of action to be taken.

8810210412 881014 PDR ADOCK 05000327

X YES IT YES COMPANY EXPECTED SUBMISSION DATE:

ABSTRACT IS only to 1400 spaces, i.e., approximately fifteen single-space systemation lines (14)

1erz

1 2 3 1 1 8 18

NRC Form 386A (9-83) LICENSEE EV	ENT REPORT (LER) TEXT CONTIN	v	U.S. NUCLEAR DEGULATORY COMMISSIO APPROVED OMB NO. 3150-0104 EXPIRES 8:31/86								
FACILITY NAME (1)	DOCKET NUMBER (2)	T	Li	R NUMBER (6)	NUMBER (6)				(3)		
		YEAR		SEQUENTIAL NUMBER		REVISION NUMBER		T			
Sequoyah, Unit 1	0 5 0 0 0 3 2 7	8 8						OF	0 4		

EXT IR more space is required, use additional NRC Form 366A's: 117)

DESCRIPTION OF EVENT

At 1359 EDT, on September 29, 1988, with unit 1 in mode 4 (0 percent power, 355 psig, and 246 degrees F) and unit 2 in mode 1 (65 percent power, 2235 psig, and 565 degrees F), 3 train 'B' Auxiliary Building Isolation (ABI) (EIIS code VF) occurred. The Auxiliary Building (AB) is a column building for units 1 and 2 and is the secondary containment enclosure for both units.

Before this event, modifications personnel were performing work inside O-M-12 under Work Plan (WP)-11266 Rl. O-M-12, which is a common panel for units 1 and 2, is located between each units main control room (MCR) boards and primarily contains radiation monitoring instrumentation (EIIS code IL). WP-11266 Rl was generated to implement a design change which installs a seal-in circuit for an ABI caused by a high radiation condition in the AB. This circuit seals in the high radiation ABI signal requiring a manual reset to remove the signal.

A high radiation time delay relay (TDR) for radiation monitor (RM) O-RM-90-103 is also housed inside O-M-12. This TDR is a plug-in type which deenergizes to trip and is located in the vicinity of the work being performed under the WR identified above. O-RM-90-103 is the ABI train 'B' RM for the spent fuel pool area which is common to both units and located within the AB.

A 1359 EDT, on September 29, 1988, the O-RM-90-103 high radiation annunicator alarmed and the unit 1 operations personnel observed that the radiation analyzer for O-RM-90-103 did not indicate a high radiation level. Operations personnel proceeded to identify if any equipment had been affected based on the alarm. The operators observed that a train 'B' ABI actuation had occurred and immediately, initiated an investigation in accordance with SQA-186, "Root Cause Assessment for Adverse Actions/Conditions". In addition, the operators were aware of the work being performed in O-M-12 and immediately requested that the work be stopped until further notice.

CAUSE OF EVENT

The cause of this event has been investigated including the possible causes of an ABI train 'B' actuation, and it has been concluded that while performing the work inside O-M-12, an electrician inadvertently bumped the time delay relay (TDR) for O-RM-90-103.

The normal initiating sources which will actuate an ABI are as follows:

- Manual initiation from either unit by handswitches HS-30-101A and -101B on panel M-6 (requires both handswitches from either unit to initiate both trains of ABI).
- 2. Unit 1 or 2 containment phase A isolation 'ignal (both trains A and B).

NRC Form 366A (9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION
APPRO ED OMB NO. 3180-0104

FACILITY NAME (1)

DOCKET NUMBER (2)

LER NUMBER (6)

FAGE (3)

VEAR SEGUENTIAL REVISION NUMBER
NUMBER
NUMBER

Sequeyah, Unit 1

0 | 5 | 0 | 0 | 0 | 3 | 2 | 7 | 8 | 8 --- | 0 | 0 | 0 | 3 | 0 | 0 | 4

FEXT /If more space is required, use additional NRC Form 3664's) (17)

- Outside air inlet temperature to AB exceeding 115 degrees F (TS-30-103 and -103A for unit 1 or TS-30-104 and 104a for unit 2). This requires actuation of both temperature switches from either unit to get both trains of ABI.
- High radiation in the AB general exhaust vent sensed by O-RM-90-101A, -101B, or 101C. (Any one channel will initiate both trains of ABI).
- High radiation in the spent fuel pool area sensed by O-RM-90-102 (Train A) and O-RM-90-103 (Train B). Note: These monitors do not start the various AB space cooler units.

Immediately after the ABI actuation, the radiation monitor (RM) recorder chart (RR-90-102) for the spent fuel pool RMs (RM-90-102 and -103) was reviewed and verified that no high radiation signal was received from these two monitors. Also, the (RM) recorder (1-RR-90-101) chart was reviewed to confirm that a high radiation condition did not exist in the AB general exhaust vent (RM-90-101A, -101B and -101C). This confirmed that no actual high radiation condition existed. Also, the operator did receive a high radiation alarm, however, the radiation analyzer for 0-RM-90-103 did not indicate that the high radiation setpoint had been exceeded by the numpresence of a red light. This combination of indications can be generated by the circuitry associated with the TDR.

Interviews with the personnel performing the work subsequent to the event supported the conclusion that the TDR was bumped resulting in the ABI train 'B' actuation.

ANALYSIS OF EVENT

An ABI is an engineered safety features (ESF) actuation and is reportable pursuant to the criteria established in 10 CFR 52.73, a.2.iv.

Subsequent to the ABI signal, all equipment functioned as designed as the general supply and exhaust fans shut off, the fuel handling area fans shut off, the appropriate dampers operated, and train 'B' of Auxiliary Building gas treatment system started. Since no high radiation levels actually existed, then this unplanned operation of an ESF posed no safety consequence.

CORRECTIVE ACTION

Immediate corrective action taken by the operators subsequent to the ABI was to verify there was not an actual high radiation condition in the spent fuel pool area. Operators then performed SOI-30.5D, "Recovery From Auxiliary Building Isolation", for the restoration of the ventilation system, and an immediate investigation of the event ensued. The investigation was implemented in accordance with SQA-186. The following two items reflect the type of recommendations resulting from the investigation to prevent recurrence of this type of event:

	Form 368A
(9-83)	

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OME NO. 3150-0104 EXPIRES: 8/31/88

								EAPINEDI: 6/31/86													
FACILITY NAME (1)		DOCKET NUMBER (2)								LER NUMBER (6)									PAGE (3)		
	F									YE	4.9		BEC	UNI	TIAL ER		REVISION NUMBER		T		
Sequoyah, Unit 1	0	15	10	10	10		3]	2	7	8	8	neme	0	1 3	14	_	010	0 4	OF	0	14

TEXT (If more space is required, use additional NRC Form 356A's) (17)

- Require the appropriate monitors to be blocked when work is performed inside a MCR panel housing a TDR.
- Install a protective cover for TDRs associated with radiation monitors that generate ESF actuations.

TVA will evaluate both options and revise this report by December 31, 1988 to provide the course of action to be taken.

ADDITION INFORMATION

There have been two previous occurrences of ABIs caused by a TDR being bumped SQRO-50-327/84068 and 85023. These events involved modifications personnel and electrical maintenance personnel; respectively.

COMMITMENTS

 TVA will evaluate both recommendations resulting from the investigation of this event and will revise this report by December 31, 1988 to provide NRC with the course of action to be taken and a schedule for implementation.

00760

TENNESSEE VALLEY AUTHORITY Sequoyah Nuclear Plant Post Office Box 2000 Soddy-Daisy, Tennessee 37379

October 14, 1988

U. S. Nuclear Regulatory Commission Pocument Control Desk Washington, DC 20555

Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNITS 1 AND 2 - DOCKET NOS. 50-327 AND 50-328 - FACILITY OPERATING LICENSE DPR-77 AND -79 - REPORTABLE OCCURRENCE REPORT SQR0-50-327/88034 REVISION 0

The enclosed licensee event report provides details concerning an inadvertent Auxiliary Building isolation which occurred during modification work inside again control room panel. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.iv.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

S. J. Smith Plant Manager

Enclosure c (Enclosure):

> J. Nelson Grace, Regional Administrator U. S. Nuclear Regulatory Commission Suite 2900 101 Marietta Street, NW Atlanta, Georgia 30323

Records Center Institute of Nuclear Power Operations Suite 1500 1100 Circle 75 Parkway Atlanta, Georgia 30339

NRC Inspector, Sequoyah Muclear Plant

TE2