



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

UNIVERSITY OF VIRGINIA

DOCKET NO. 50-62

AMENDMENT TO FACILITY LICENSE

Amendment No. 12
License No. R-66

1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by the University of Virginia (the licensee) dated July 14, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and
 - F. Publication of notice of this amendment is not required since it does not involve a significant hazards consideration nor amendment of a license of the type described in 10 CFR Section 2.106(a) (2).

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2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 3(2) of Facility License No. R-66 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 12, are hereby incorporated in the license. The University of Virginia shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of its issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Morton B. Fairclough

Robert W. Reid, Chief
Operating Reactors Branch #4
Division of Operating Reactors

Attachment:
Changes to the Technical
Specifications

Date of Issuance: December 19, 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 12

FACILITY LICENSE NO. R-66

DOCKET NO. 50-62

Replace page 24 of the Appendix A Technical Specifications with the attached revised page. The change on the revised page is indicated by a marginal line.

4.0 SURVEILLANCE REQUIREMENTS

4.1 Safety Rods

Applicability

This specification applies to the surveillance requirements for the safety rods.

Objective

To assure that the safety rods are capable of performing their function and that no significant physical degradation in the rods has occurred.

Specification

- a. Safety rod drop times shall be measured at intervals not to exceed five months. Safety rod drop times shall also be measured if the control assembly is moved to a new position in the core or if maintenance is performed on the mechanism.
- b. The safety rod reactivity worths shall be measured whenever the rods are installed in a new core configuration.
- c. The safety rods shall be visually inspected at intervals not to exceed thirteen months, and when rod drop times exceed the limiting conditions for operation, Section 3.9 of these specifications. If the safety rod is found to be deteriorated and to have a crack of more than 1/4 inches in length, it shall be removed from service.

Bases

The reactivity worth of the safety rods is measured to assure that the required shutdown margin is available and to provide means for