NORTHEAST UTILITIES

THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

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March 20, 1987

Docket No. 50-336
B12473
Re: 10 CFR 50.90

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, D.C. 20555

Gentlemen:

Millstone Nuclear Power Station, Unit No. 2
Additional Information
Control Rod Drive Mechanism Operability

In a letter dated February 6, 1987[1], Northeast Nuclear Energy Company (NNECO) proposed to amend its Operating License, DPR-65, by incorporating changes to the Technical Specifications of Millstone Unit No. 2. These proposed changes correct an inconsistency between the Safety Analysis and the Technical Specifications by placing restrictions on the operability of the control rod drive mechanisms (CRDMs).

This license amendment request has been discussed with the NRC Staff in telephone conversations since that time. As a result, NNECO is hereby documenting the following additional information in support of the license amendment request submitted in Reference (1).

- With respect to current Technical Specification 3.9.1, maintaining the refueling Reactor Coolant System (RCS) boron concentration (MODE 6) ensures that there is adequate shutdown margin even if all control rods are removed from the core. That is why the "high power" reactor trip is not required to be available nor are the CRDMs required to be deenergized if the boron concentration is greater than or equal to the refueling concentration.

- The "high power" reactor trip is only required in Mode 3 to provide mitigation for the "Uncontrolled Rod Withdrawal from Subcriticality" accident. Therefore this trip is not necessary if the RCS boron concentration is greater than or equal to the refueling concentration or if the CRDMs are deenergized.

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(1) E.J.Mroczka letter to the U.S. Nuclear Regulatory Commission, "Proposed Changes to Technical Specifications, Control Rod Drive Mechanism Operability," dated February 6, 1987.

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o The evaluation of the "Uncontrolled Rod Withdrawal from Subcriticality" accident for Millstone Unit No. 2 performed by the fuel vendor used as assumptions the conditions specified in the "Note" included in proposed Technical Specification 3.1.3.7:

- 4 RCPs Operating
- RCS Temperature >500 °F
- RCS Pressure >2000 psia
- High Power Reactor Trip Operable

This ensures that the CRDMs are energized only during those conditions which have been analyzed in the Safety Analysis.

We trust that you will find this information satisfactory.

Very truly yours,

NORTHEAST NUCLEAR ENERGY COMPANY

E.J.Mroczka
Senior Vice President

cc: Dr. T.E.Murley, Region I Administrator
D.H.Jaffe, NRC Project Manager, Millstone Unit No. 2
T.Rebelowski, Resident Inspector, Millstone Unit Nos. 1 & 2