

Log # TXX-6332 File # 10110 910.3 Ref # 10CFR50.55(e)

William G. Counsil Executive Vice President

March 20, 1987

U. S. Nuclear Regulatory Commission Attn: Document Control Desk Washington, D.C. 20555

SUBJECT: COMANCHE PEAK STEAM ELECTRIC STATION (CPSES) DOCKET NOS. 50-445 AND 50-446 INSTRUMENTATION INSTALLATIONS SDAR: CP-86-19 (INTERIM REPORT)

Gentlemen:

On March 21, 1986, we notified you of a reportable item involving the installation of steam service pressure transmitters (see TXX-4824). This is a follow-up interim report to status the corrective actions for a reportable item under the provisions of 10CFR50.55(e). Our last interim report was logged TXX-6235 dated January 21, 1987.

The scope of this item has been expanded to include corrective action implementation for SDARS: CP-86-16, "Fire Effects on Instrumentation Tubing," CP-86-5C, "Unistrut Spring Nuts on Instrument Supports," CP-86-70, "Elevated Temperature Effects on Instrument Supports and Tubing," and CP-86-77, "Instrument Tubing Minimum Wall Thickness"; all of which were previously reported under the provisions of 10CFR50.55(e).

We have completed a comprehensive evaluation of instrument installations with proposals to relocate them as necessary. The design engineering package is in an internal review cycle.

Engineering is in the process of evaluating correspondence, design documentation and technical chemistry reports covering zinc contamination of and fire effects on stainless steel tubing. The scope of corrective action will be forthcoming after completion of this study.

We are implementing a program to assure proper alignment and torquing of unistrut nuts. Installation specifications and design drawings are being revised and retraining of affected personnel will be soluted.

We are in the process of revising instrument tubing layout design criteria to include the provision that makes the layout compatible with high environmental temperatures caused by a postulated Loss of Coolant Accident (LOCA) event. This criteria is based on ASME Code Case N-47 which specifies the strain limits for a "one-time" event.

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We are currently evaluating the instrumentation tubing minimum allowable wall thickness criteria previously established. After completion of the evaluation, a walkdown and reinspection of tubing is planned. Procedures and checklists are being established to implement the walkdown.

We are currently evaluating instrument support types for load capacity and the ability to accommodate a High Energy Line Break (HELB) load.

All the above instrumentation evaluations are scheduled to be completed on or before June 30, 1987. Our next report will be submitted on or before July 31, 1987.

Very truly yours,

W. G. Counsel

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