

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Sequoyah, Unit 1	DOCKET NUMBER (2) 0 5 0 0 0 3 2 17	PAGE (3) 1 OF 0 3
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TITLE (4)
Failure to Perform the Correct Monthly Channel Check for the Steam Generator Level

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																																																							
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																																																					
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<table border="1" style="width:100%; border-collapse: collapse;"> <tr> <td style="width:15%;">OPERATING MODE (9) 5</td> <td colspan="11">THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)</td> </tr> <tr> <td rowspan="5">POWER LEVEL (10) 0 0 0</td> <td>20.402(b)</td> <td></td> <td>20.406(c)</td> <td></td> <td>50.73(a)(2)(iv)</td> <td></td> <td>73.71(b)</td> </tr> <tr> <td>20.406(a)(1)(ii)</td> <td></td> <td>50.36(c)(1)</td> <td></td> <td>50.73(a)(2)(v)</td> <td></td> <td>73.71(c)</td> </tr> <tr> <td>20.406(a)(1)(iii)</td> <td></td> <td>50.36(c)(2)</td> <td></td> <td>50.73(a)(2)(vii)</td> <td></td> <td rowspan="3">OTHER (Specify in Abstract below and in Text, NRC Form 366A)</td> </tr> <tr> <td>20.406(a)(1)(iii)</td> <td>XX</td> <td>50.73(a)(2)(i)</td> <td></td> <td>50.73(a)(2)(viii)(A)</td> <td></td> </tr> <tr> <td>20.406(a)(1)(iv)</td> <td></td> <td>50.73(a)(2)(ii)</td> <td></td> <td>50.73(a)(2)(viii)(B)</td> <td></td> </tr> <tr> <td>20.406(a)(1)(v)</td> <td></td> <td>50.73(a)(2)(iii)</td> <td></td> <td>50.73(a)(2)(ix)</td> <td></td> <td></td> </tr> </table>												OPERATING MODE (9) 5	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)											POWER LEVEL (10) 0 0 0	20.402(b)		20.406(c)		50.73(a)(2)(iv)		73.71(b)	20.406(a)(1)(ii)		50.36(c)(1)		50.73(a)(2)(v)		73.71(c)	20.406(a)(1)(iii)		50.36(c)(2)		50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)	20.406(a)(1)(iii)	XX	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)		20.406(a)(1)(iv)		50.73(a)(2)(ii)		50.73(a)(2)(viii)(B)		20.406(a)(1)(v)		50.73(a)(2)(iii)		50.73(a)(2)(ix)		
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LICENSEE CONTACT FOR THIS LER (12)

NAME Glenn E. Duggin, Compliance Staff Engineer	TELEPHONE NUMBER 6 1 5 8 7 0 - 6 5 4 8
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COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC

SUPPLEMENTAL REPORT EXPECTED (14)

<input type="checkbox"/> YES (if yes, complete EXPECTED SUBMISSION DATE) <input checked="" type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15) MONTH: DAY: YEAR:
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ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

A technical specification (TS) review committee was established temporarily at Sequoyah to ensure that all surveillance requirements (SR) were correctly performed by and included in the plant's surveillance instructions (SI). On April 1, 1986, this committee discovered that a monthly channel check for the steam generator (S/G) (IEEE AB system) narrow range level remote shutdown instrumentation was not properly performed on SI-3, "Daily, Weekly, and Monthly Logs." SI-3 had been incorrectly written to check the wide range level instruments. The wide range instruments are on the same local panel as the narrow range instruments, but there is only one channel for each S/G for the wide range. The TS indicates that two channels exist, and this is only true for the two narrow range instruments, one on a local panel and the other in the auxiliary control room. The channel determination of operability will be done by comparing the two channels for each S/G and making sure they agree within six percent of each other. If they do not agree, they would be compared to the main control room indicators to determine which one is incorrect. Calibrations on these instruments were done correctly in the past.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

DESCRIPTION OF EVENT

During the discovery of this occurrence, unit 1 was in mode 5 (0 percent power, 280 psig, 138 degrees F) and unit 2 was in mode 5 (0 percent power, 260 psig, 132 degrees F).

A technical specification (TS) review committee was established temporarily at Sequoyah to ensure that all surveillance requirements (SR) were correctly performed by and included in the plant's surveillance instructions (SI). On April 1, 1986, at 1300 CST, this TS review committee discovered that a monthly channel check for the steam generator (S/G) (IEEE AB system) narrow range level remote shutdown instrumentation was not properly performed on SI-3, "Daily, Weekly, and Monthly Logs." SI-3 had been incorrectly written to check the wide range level instruments. The wide range instruments are on the same local panel as the narrow range instruments, but there is only one channel for each S/G for the wide range. The TS indicates that two channels exist, and this is only true for the two narrow range instruments, one on a local panel and the other in the auxiliary control room. There are separate SI-3s for both units, and the information enclosed applies to both units. The monthly channel check is required by SR 4.3.3.5.

CAUSE OF EVENT

SI-3 was written with the interpretation that the wide range level indicators at the auxiliary feedwater turbine control panel (panel L-381, EL 669) were the proper indicators to check. This was found to be incorrect since there are no redundant channels for the wide range indicators. The TS requires one of the channels checked to be in the auxiliary control room (panel L-11); therefore, the narrow range level indicators must be used. The other channels to be checked are still located on panel L-381 (one channel per S/G at each location per unit).

Originally, the calibration SI for the remote shutdown instruments only included four out of eight per unit, and the other four were in another SI. However, all of the instruments were calibrated correctly and at the correct frequency. The four instruments were moved to the correct SI before the SI-3 discrepancy was discovered.

ANALYSIS OF EVENT

Since SI-3 did not perform a channel check on the correct channels, the TS SR was not met. This occurrence is an operation prohibited by TS, and it is reported as such. This condition is assumed to have existed since the remote shutdown instrumentation was included in SI-3.

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The remote shutdown instruments were calibrated correctly; therefore, even though the channel check was not performed, they should have been available for use. Also, the narrow range and wide range instruments in the main control room would have been available, and the wide range instruments that were channel checked would have been available. There was no effect on public health or safety because of the availability of other indicators. The mode of operation would not have influenced the safety consequences if these indicators had been lost.

Also, since the remote shutdown wide range indicators were beside the narrow range indicators on the local control panel, they could have been compared to determine operability.

The channel determination of operability in SI-3 will be done by comparing the two channels for each S/G and making sure they agree within six percent of each other. If they do not agree, they will be compared to the main control room indicators to determine which one is incorrect. A work request would then be prepared, and the inoperable channel repaired and recalibrated.

CORRECTIVE ACTION

SI-3 will be revised to include the correct remote shutdown instrumentation by July 31, 1986. No further action is deemed necessary.

Previous occurrence - SQRO-50-327/86007.

TENNESSEE VALLEY AUTHORITY

Sequoyah Nuclear Plant
Post Office Box 2000
Soddy-Daisy, Tennessee 37379

May 1, 1986

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

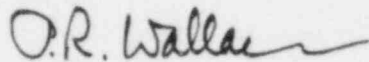
Gentlemen:

TENNESSEE VALLEY AUTHORITY - SEQUOYAH NUCLEAR PLANT UNIT 1 - DOCKET NO.
50-327 - FACILITY OPERATING LICENSE DPR-77 - REPORTABLE OCCURRENCE REPORT
SQRO-50-327/86008

The enclosed licensee event report provides details concerning the failure to perform the correct monthly channel check for the steam generator narrow range level remote shutdown instrumentation. This event is reported in accordance with 10 CFR 50.73, paragraph a.2.i.

Very truly yours,

TENNESSEE VALLEY AUTHORITY



P. R. Wallace
Plant Manager

Enclosure
cc (Enclosure):

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NRC Inspector, Sequoyah Nuclear Plant

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