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and the surveillance test procedure was successfully completed with no further problems occurring. There was no impact on the safe operation of the plant or the health and safety of the public.

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NRC Form 386 (9 83)

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NUCLEAR REGULATORY COMMISSION

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Reported Condition

AC Form JEEA

On 3/26/86 at 2332 with the unit at cold shutdown during a maintenance outage, a Division I Residual Heat Removal (RHR) system isolation occurred. This was caused during the performance of Surveillance Test Procedure STP-207-4209 "Main Steam Line Temperature Quarterly Channel Calibration" when a bypass jumper was inadvertently shorted to an adjacent test lead.

Division I of the RHR system was restored to service at 2357 and STP-207-4209 was successfully completed with no further problems occurring.

Investigation

Surveillance Test Procedure STP-207-4209 requires test leads to be connected to terminals 1 and 2 of Reactor Core Isolation Cooling (RCIC) main steam tunnel temperature timer E31-R617E. The procedure also requires a bypass jumper to be installed between terminal 1 of timer E31-R617E and terminal M1 of relay E31A-K29A. During the process of installing the jumper on terminal 1 of E31-R617E, the jumper connector was inadvertently shorted to the test lead which had previously been installed on terminal 2 of E31-R617E. This caused leak detection system power supply fuse E31-F2A to blow and RHR isolation valve E12*MOVF008 to isolate.

An investigation of the timer terminal blocks on Division I and II found that they do not provide adequate electrical separation when using anything other than lugged wires. While investigating a method

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placing the jumper, it was	concluded that	the	timers	coul	ld 1	be	
permanently isolated from the	logic.						

Corrective Action

Based on the timing action of timers E31-R617E and E31-R617F being set at zero (0) time delay, Modification Request 86-0539 was initiated to eliminate the timers from the logic. Implementation of the modification will prevent recurrence of this condition by eliminating the need for STP-207-4209 and similar Surveillance Test Procedure STP-207-4210.

Similar isolations of E12*MOVF008 were reported in LERS 85-016 and 85-017. In LER 85-016 a jumper slipped off a terminal and shorted to ground. Corrective action was aimed at making a more secure connection. In LER 85-017 a technician inadvertently grounded a recorder lead because of difficulty in accessing the terminal point. In this later case corrective action taken was to bring all signal points for STP-204-0602 "RHR System Isolation 18 Month Logic System Functional Test..." to a more accessible terminal block. The isolation reported here could not have been anticipated from the events reported in LERS 85-016 and 85-017.

Safety Assessment

There was no impact on the safe operation of the plant or the health and safety of the public.



RIVER BEND STATION POST OFFICE BOX 220 ST. FRANCISVILLE, LOUISIANA 70775 AREA CODE 504 635-6094 346-8651

> April 25, 1986 RBG-23582 File Nos. G9.5, G9.25.1.3

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Dear Sir:

River Bend Station - Unit 1 Docket No. 50-458

Please find enclosed Licensee Event Report No. 86-025 for River Bend Station - Unit 1. This report is submitted pursuant to 10CFR50.73.

Sincerely,

EBooher

J. E. Booker Manager-Engineering, Nuclear Fuels & Licensing River Bend Nuclear Group

JEB/TFP/DRG/BEH/je

cc: U.S. Nuclear Regulatory Commission 611 Ryan Plaza Drive, Suite 1000 Arlington, TX 76011

> INPO Records Center 1100 Circle 75 Parkway Atlanta, GA 30339-3064

