|                                | TABLE 3.1.1 REACTOR                         | PROTECTION SYSTEM (SCRAM)            |                         | ATION REQUIR           |        |        |
|--------------------------------|---|--------------------------------------|-------------------------|------------------------|--------|--------|
| Minimum Number                 |   |                                      | Modes in Which Function |                        |        |        |
| Operable Inst.                 | Trip Function                               | Trip Level Setting                   | Must                    | Must Be Operable       |        | Action |
| Channels per<br>Trip(1) System |   |                                      | Refuel (7)              | Startup/Hot<br>Standby |        |        |
| 1                              | Mode Switch in Shutdown                     |                                      | X                       | Х                      | X      | A      |
| 1                              | Manual Scram                                |                                      | x                       | ×                      | x      | A      |
|                                | IRM   |                                      |                         |                        |        |        |
| 3                              | High Flux                                   | <120/125 of full scale               | X                       | X                      | (5)    | Α      |
| 3                              | Inoperative                                 |                                      | X                       | Х                      | (5)    | A      |
|                                | APRM  |                                      | 1.6.1.1.1.5.            |                        |        |        |
| 2                              | High Flux                                   | * (14) (15)                          | (17)                    | (17)                   | X      | A OF B |
| 2                              | Inoperative                                 | (13)                                 | X                       | X(9)                   | X      | A or B |
| 2                              | Downscale                                   | >2.5 Indicated on Scale              | (11)                    | (11)                   | X(12)  | A or B |
| 2                              | High Flux (15%)                             | <15% of Design Power                 | X                       | x                      | (16)   | A or P |
| 2                              | High Reactor Pressure                       | <1085 psig                           | X(10)                   | X                      | X      | A      |
| 2                              | High Drywell Pressure                       | ≤2.5 psig                            | X(8)                    | ×(8)                   | x      | A      |
| 2                              | Peactor Low Water Level                     | 29 In. Indicated Lage:               | X                       | x                      | X      | A      |
| 2                              | High Water Level in Scram<br>Discharge Tank | <39 Gallons                          | X(2)                    | x                      | x      | A      |
| 2                              | Turbine Condenser Low<br>Vacuum             | ≥23 In. Hg Vacuum                    | X:3)                    | X(3)                   | x      | A or C |
| 2                              | Main Steam Line High                        | <7X Normal Full Power                | art to a                |                        | See. 1 |        |
| and a most                     | Radiation                                   | Background (18)                      | X                       | Х                      | X(18)  | A or C |
| 4                              | Main Steam Line Isolation<br>Valve Closure  | ≤10% Valve €losure                   | X(3)(6)                 | X(3)(6)                | X(6)   | A or C |
| 2                              | Turb. Cont. Valve Fast                      | ≥150 psig Control Oil<br>Pressure at | P Ch                    | 123                    |        |        |
|                                | Ciosure                                     | Acceleration Relay                   | X(4)                    | X(4)                   | X(4)   | A or D |
| 4                              | Turbine Stop Valve Closure                  | 10% Valve Closure                    | X(4)                    | X(4)                   | X(4)   | A or D |
|                                | *APRM high flux scram setpo                 | oint ((.53W + 62%) FRP_              | Two recirc              | . pump opera           | tion   |        |
|                                | R40923021                                   | READEN MELPD                         | and Zara                | Pat                    |        |        |
| Amendment No. 9                | 2 PDR ADOC                                  | k 05000293                           |                         |                        |        | 27     |

## TABLE 2 1 1 DEACTOR PROTECTION SYSTEM (SCRAM) INSTRUMENTATION REQUIREMENT

## NOTES FOR TABLE 3.1.1 (CONT'D)

- Not required to be operable when the reactor pressure vessel head is not bolted to the vessel.
- The APRM downscale trip function is only active when the reactor mode switch is in run.
- 12. The APRM downscale trip is automatically bypassed when the IRM instrumentation is operable and not high.
- 13. An APRM will be considered inoperable if there are less than 2 LPRM inputs per level or there is less than 50% of the normal complement of LPRM's to an APRM.
- 14. W is percent of drive flow required to produce a rated core flow of 69 Mlb/hr. Trip level setting in percent of design power (1938 MWt).
- 15. See Section 2.1.A.1.

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- 16. The APRM (15%) high flux scram is bypassed when in the run mode.
- The APRM flow biased high flux scram is bypassed when in the refuel or startup/hot standby modes.
- 18. Within 24 hours prior to the planned start of hydrogen injection with the reactor power at greater than 20% rated power, the normal full power radiation background level and associated trip setpoints may be changed based on a calculated value of the radiation level expected during the injection of hydrogen. The background radiation level and associated trip setpoints may be adjusted based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be determined and associated trip setpoints shall be set within 24 hours of re-establishing normal radiation levels after completion of hydrogen injection and prior to withdrawing control rods at reactor power levels below 20% rated power.

|                 |      | PNI       | PS      |             |           |  |
|-----------------|------|-----------|---------|-------------|-----------|--|
| TABLE 3.2.A     |      |           |         |             |           |  |
| INSTRUMENTATION | THAT | INITIATES | PRIMARY | CONTAINMENT | ISOLATION |  |

| Ope | Minimum # of<br>rable Instrument<br>s Per Trip System (1) | Instrument  | Trip Level Setting  | Action (2) |
|-----|---|---|---|------------|
|     | 2(7)  | Reactor Low Water Level                                 | $\geq$ 9" indicated level (3)                             | A and D    |
|     | 1   | Reactor High Pressure                                   | ≤110 psig   | D          |
|     | 2   | Reactor Low-Low Water Level                             | at or above -49 in.<br>indicated level (4)                | A          |
|     | 2   | Reactor High Water Level                                | ≤48" indicated level (5)                                  | В          |
|     | 2(7)  | High Drywell Pressure                                   | <2.5 psig   | A          |
|     | 2   | High Radiation Main Steam<br>Line Tunnel (9)            | <pre>&lt;7 times normal rated full power background</pre> | В          |
|     | 2   | Low Pressure Main Steam Line                            | ≥880 psig (8)   | В          |
|     | 2(6)  | High Flow Main Steam Line                               | <140% of rated steam flow                                 | В          |
|     | 2   | Main Steam Line Tunnel<br>Exhaust Duct High Temperature | <u>≤</u> 170°F  | В          |
|     | 2   | Turbine Basement Exhaust<br>Duct High Temperature       | <u>≺</u> 150°F  | В          |
|     | 1   | Reactor Cleanup System<br>High Flow                     | <300% of rated flow                                       | с          |
|     | 2   | Reactor Cleanup System<br>High Temperature              | <u>≤</u> 150°F  | с          |

5 S S 6

## NOTES FOR TABLE 3.2.A

 Whenever Primary Containment integrity is required by Section 3.7, there shall be two operable or tripped trip systems for each function.

## 2. Action

If the first column cannot be met for one of the trip systems, that trip system shall be tripped. If the first column cannot be met for both trip systems, the appropriate action listed below shall be taken.

- A. Initiate an orderly shutdown and have the reactor in Cold Shutdown Condition in 24 hours.
- B. Initiate an orderly load reduction and have Main Steam Lines isolated within eight hours.
- C. Isolate Reactor Water Cleanup System.
- D. Isolate Shutdown Cooling.
- 3. Instrument set point corresponds to 129.5" above top of active fuel.
- 4. Instrument set point corresponds to 78.5" above top of active fuel.
- 5. Not required in Run Mode (bypassed by Mode Switch).
- 6. Two required for each steam line.
- These signals also start SBGTS and initiate secondary containment isolation.
- 8. Only required in Run Mode (interlocked with Mode Switch).
- 9. Within 24 hours prior to the planned start of hydrogen injection with the reactor power at greater than 20% rated power, the normal full power radiation background level and associated trip setpoints may be changed based on a calculated value of the radiation level expected during the injection of hydrogen. The background radiation level and associated trip setpoints may be adjusted based on either calculations or measurements of actual radiation levels resulting from hydrogen injection. The background radiation level shall be determined and associated trip setpoints shall be set within 24 hours of re-establishing normal radiation levels after completion of hydrogen injection and prior to withdrawing control rods at reactor power levels below 20% rated power.