

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Virgil C. Summer Nuclear Station

DOCKET NUMBER (2)
0 5 0 0 0 3 9 5 1 OF 0 1

PAGE (3)
1 OF 0 1

TITLE (4)
Turbine Trip / Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 7	2 7	8 6	8 6	0 1 4	0 0	0 8	2 6	8 6			0 5 0 0 0
<p>THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)</p>											

OPERATING MODE (9) 1	20.402(b)	20.406(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10) 90	20.406(a)(1)(i)	50.36(e)(1)	50.73(a)(2)(v)	73.71(c)
	20.406(a)(1)(ii)	50.36(e)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.406(a)(1)(iii)	50.73(a)(2)(i)	60.73(a)(2)(viii)(A)	
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	60.73(a)(2)(viii)(B)	
	20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME: **W. R. Higgins, Assoc. Manager, Regulatory Compliance**

TELEPHONE NUMBER: **8 0 3 3 4 5 - 5 2 0 9**

AREA CODE: **8 0 3**

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS
X	T, G			N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On July 27, 1986, the plant was operating at approximately 90 percent power when, at 0145 hours, a reactor trip occurred. Power had been reduced so that the monthly test of the Turbine Control Valves (CV) could be accomplished in accordance with PTP-120.001. CV-1 was successfully tested at 0141, CV-2 at 0142, and CV-3 at 0144. During the test of CV-4, a reactor trip was initiated. The cause of the reactor trip has been attributed to the spurious operation of one of the two other CV pressure switches coincident with the test of CV-4 which completed the 2 out of 3 logic. It is suspected that the spurious actuation of the second CV pressure switch was due to the induced vibration caused by the closing of CV-4. Maintenance personnel examined the three pressure switches, verified calibration and operability, and found no significant problems. Following the investigation by maintenance personnel, a restart was authorized and criticality achieved at 1135 hours, July 27, 1986. Consideration is being given to relocating the CV pressure switches to a location that would not be susceptible to the vibration induced by the CV closure. There were no adverse consequences due to this event. No safety limits were exceeded, and the Reactor Protection System functioned properly.

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South Carolina Electric & Gas Company
P.O. Box 764
Columbia, SC 29218
(803) 748-3513

Dan A. Nauman
Vice President
Nuclear Operations

August 26, 1986

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

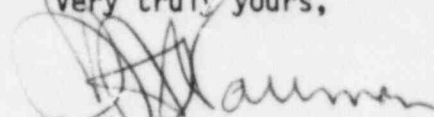
SUBJECT: Virgil C. Summer Nuclear Station
Docket No. 50/395
Operating License No. NPF-12
LER 86-014

Dear Sir:

Attached is Licensee Event Report #86-014 for the Virgil C. Summer Nuclear Station. This report is submitted pursuant to the requirements of 10 CFR 50.73(a)(2)(iv).

Should there be any questions, please call us at your convenience.

Very truly yours,



D. A. Nauman

RJB/DAN:dwf
Attachment

pc: O. W. Dixon, Jr./T. C. Nichols, Jr.
E. H. Crews, Jr.
E. C. Roberts
J. G. Connelly, Jr.
W. A. Williams, Jr.
J. Nelson Grace
Group Managers
O. S. Bradham
D. R. Moore
W. R. Baehr
K. W. Woodward
C. A. Price
W. T. Frady

J. F. Heilman
C. L. Ligon (NSRC)
R. M. Campbell, Jr.
K. E. Nodland
R. A. Stough
G. O. Percival
R. L. Prevatte
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