

Mr. O. J. Zeringue  
Chief Nuclear Officer  
and Executive Vice President  
Tennessee Valley Authority  
6A Lookout Place  
1101 Market Street  
Chattanooga, Tennessee 37402-2801

February 27, 1998

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION - GENERIC LETTER 96-06,  
ASSURANCE OF EQUIPMENT OPERABILITY AND CONTAINMENT  
INTEGRITY DURING DESIGN BASIS ACCIDENTS (TAC NOS. M96784,  
M96785, AND M96786)

Dear Mr. Zeringue:

By letter dated January 23, 1997, the Tennessee Valley Authority (TVA) responded to Generic Letter 96-06. In that response, TVA indicated that two pipelines penetrating containment and identified as susceptible to thermally-induced pressurization were evaluated using ASME Code Appendix F criteria. TVA letter dated October 23, 1997, revised the response. In the revised response, TVA indicates that further evaluation has demonstrated that these lines meet the design code criteria.

One of these lines is associated with the penetration for the sampling system. TVA's revised response indicates the sampling system inboard isolation valve will unseat and relieve pressure prior to exceeding the design criteria for the piping or valves. Please provide a drawing of the inboard isolation valve and the calculation(s) used to determine the pressure needed to unseat the valve. Additionally, provide a discussion of the uncertainties associated with this calculation.

The October 23, 1997, revision also identified additional lines where valve leakage is expected to prevent overpressure of the lines. These are the Drywell Floor and Equipment Drain Sump discharge lines. TVA indicated that it intends to implement a design change to provide overpressure protection for these lines. Please describe the intended design change which will provide overpressure protection for these lines.

Please provide your response no later than March 27, 1998.

Sincerely,

Original Signed by:  
Albert W. DeAgazio, Sr. Project Manager  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

Docket Nos. 50-259, 50-260,  
and 50-296

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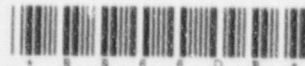
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DATE	02/27/98		02/27/98		02/27/98		02/26/98		02/ /98

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Dear Mr. Zeringue:

By letter dated January 28, 1997, the Tennessee Valley Authority (TVA) responded to Generic Letter 96-06. In that response, TVA indicated that two pipelines penetrating containment and identified as susceptible to thermally-induced pressurization were evaluated using ASME Code Appendix F criteria. TVA letter dated October 23, 1997, revised the response. In the revised response, TVA indicates that further evaluation has demonstrated that these lines meet the design code criteria.

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Please provide your response no later than March 27, 1998.

Sincerely,

A handwritten signature in black ink, reading "Albert W. De Agazio, Sr.", is positioned above the typed name.

Albert W. De Agazio, Sr. Project Manager  
Project Directorate II-3  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

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and 50-296

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Mr. O. J. Zeringue  
Tennessee Valley Authority

**BROWNS FERRY NUCLEAR PLANT**

cc:

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