NRC Form (9-83)	n 366										LIC	EN	SE	ES	VE	NT	RE	POR	RT (	LER	)			U.S. I	AP	PROV	REG(	0MB 1 31/88	NO 318	OMM	ISSION M
FACILITY	NAME	(1)							-				-									DO	CKET N	UMB	R (2)		-		F-	PAGE	E (3)
D.C	. Co	ok	Nuc	le.	ar	P1	ant	:, 1	Uni	it	1									100	ite da	0	151	010	010	B	1	115	1	OF	01
TITLE 14	F	ail	ure	e to	o V	er	ify	T	ime	e R	esp	ons	se	of	T	urb	ine	T	rip		Reac	to	r Tr	ip	Fu	nct	tic	on i	fron	n	
	S	tea	m (	Jen	era	to	rł	lig	n-ł	Hig	h d	ue	t	o P	re	ced	ura	11	Det	ici	ency	•									
NONTH	NT DAT	E (5)		VEAL	. 1	LEP	EQUE	NTIA	(6)	REV	ISION	1404	RE	PORT	DAT	E (7)			-	FA	OTHE	AMES	CILITIE	IS INV	DLV	OCKE	TN	MBER	(S)		
NUMBER NUMBER							HBER	A MONTH DAY YEAR										0	0 151010101 1 1												
11		1							1			ŀ.,					H								+	-	-	-			
0 3	2 6	8	6	8	6 -	-b	0	7	-	0	0	0	4	2	4	8	6								0	15	10	10	101	1	1
OPE	RATING			THIS	REPOR	RTI	S SUB	MITT	D P	URSU	ANT T	TO TH	ER	EQUI	REM	INTS	OF 10	CFR	§: (C	heck o	e ar ma	re af t	he faila	wing)	(11)		_				
MC	DE (9)		1	_	20.402	t(b)						20.	405	c)			-			60.73	(1.)(2)(iv)	1			L	7	3.71	(6)			
POWE	R		-	_	20.405	ile){	1)(i)				-	50.	36 (c	)(1)					_	50.73	(a)(2)(v)				1	- 7	3.71	(c)			
(10)	10	19	10	-	20.405	i(a)(*	1)(ii)					50.	36ic	)(2)				1	-	50.73	(a)(2)(vii	9			-		HOW	R (Spi and in	Text	NRC	Form
			ł	-	20.406	H#)(1	13(86)				X	50.	73(8)	)( <b>2</b> )(i)	Е. 11.					50.73	(a)(2)(vii	()(A)				3	00.A)				
			ł	-	20.405		11(4)				-	50	73(8)	1(2)(1				ł	-	50.73	(a)(2)(x)				1						
				-								ICEN	SEE	CON	TACT	FOR	THIS	LER	12)						-						
NAME						-																			TE	LEPH	ONE	NUM	BER		
																							AREA	COD	E						
L.S	. Gi	bso	n -	- Te	ech	ní	ca1	. Er	ngi	ine	eri	ng	De	epa	irti	nen	t S	Supe	eri	nte	nden	t	6	110	5 4	16	51 5	51-	15 1	91	011
		-					COMP	LETE	ONE	LINE	FOR	EAC	H CO	MPO	NENT	FAI	URE	DEBC	RIBE	DINT	HIS REP	ORT	13)					-			
CAUSE SYSTEM COMP			OMPO	ONENT MANUFAC			C-	RE	PORT	ABLE					CA	AUSE	SYST	EM	CON	PONENT		MANU	FAC		REPO	RTA	BLE				
		+			+				+							+			-			-	101	nen.	-	101	area.				
		Į.																	. 1				1		.						
		+	1_1	-	+	+	1	-	+							+			-		1.1	+		-	-			+			
	- 1	1	1	4		1		1	L		]										1.4	1	T	1							
		-				-	SUP	PLEM	ENTA	AL RE	PORT	EXPE	ECTE	ED (1	4)				-			+	-	-			TN	ONTH	DA	Y I	YEAR
													T				-					-	S	UBMIS	SION		F		-	-	
YES	i lit yes,	comple	ete EX	PECTI	ED SU	BMI	SS/ON	DATE	É)			, i la	2	X	NO								_	DATE	(15)			1			1
	Du SpPrth Th fri En fo tiu su A th sa	rin eci ote e n e d om me gin me rve rve rev is fe	g a fic cti equ isc Ste res eer per res ill isi par	a re ation ire por anc anc ation anc atic	evic ions and cemen se ion nse to cula tion	ew s, d l nt: cy ne: t ev. o i i re ar	to a Eng s o in hro iew f t s c qui ech ti or	en dis ine f T vol or ugh fc he nly rem nic me shu	ve le ve tu ve n tu ren tu	ire repared that well and the urb: require ts l Sp low	pro Sa ica tim l h y p tha ine uir · pec cns n a	oce y w feg l S e r igh art t r tr ed ifi e v t t	edu vas jua pe res i-h be	ira s n ard ci cr p - cr atif	l conse fic nse h. the redi	com ed System The control it is and it is hatint,	pli bet ten ion est irc was tor va as on. nc	and week Ti s. ing surt tr lue beek fr	y o cve try ake cip e i Bec urt	witt the Re of t ill s s pro aus	h Te sco spon urbi ance volv n th Ther peci e it act	chr pe se pr ed. e a e fo fie d v d v	tri tes tri acci bre, acci acci acci acci acci acci acci acc	l the t I dur der te n t th v n t	e Ropro	eac ced ana ing l d eop n t	tc iur itc ilu ilu ilu ilu ilu ilu ilu ilu ilu ilu	or te a or t ideo of t lete cdi: sen	ind rip the the ze	>	
	80 PI S	505 R	010 Al	011	17 CK	80	500	24 903 PD	R	5																					

#### LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/88

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)			
이 이 지수는 것 같은 것이 가지?	그는 한 가격한 것 것 같	YEAR SEQUENTIAL REVISION NUMBER NUMBER				
D.C. Cook Nuclear Plant, Unit l	0  5  0  0  0  3   1   5	5 8 6 - 0 0 7 - 0 0	0 2 0 0 0 3			

Title

RC Form 366A

Failure to verify time response of turbine trip - reactor trip function from Steam Generator high-high due to procedural deficiency.

## Date of Event

On March 26, 1986, the following was determined reportable.

## Conditions at Time of Event

Unit 1 in Mode 1, Power Operation at 90% RTP

### Description of Event

During a review to ensure procedural compliance with Technical Specifications, a discrepancy was noted between the scope of the Reactor Protection and Engineered Safeguards System Time Response test procedure and the requirements of Technical Specifications.

The discrepancy involved Technical Specification Table 3.3-5, Item 8a which requires the turbine trip - reactor trip response time to be less than or equal to 2.5 seconds after a Steam Generator water level hi-hi signal. The appropriate surveillance procedure includes the portion of the circuit fromthe origin of the signal, through the Solid State Protection System (SSPS), and up to the output of the turbine trip control master relay. It does not include the remainder of the circuit which is from the output of the turbine trip control master relay through the electro-hydraulic controls to the output of the oil pressure control relay, or the parallel part of the circuit from the output of the turbine trip control master relay through the steam stop valve circuitry to the output of the steam stop valve position switch.

### Analysis of Event

Engineering review of the discrepancy found that no credit was taken in the accident analysis for operation of the turbine trip - reactor trip. Therefore, testing of the time response of these trips is not required from an operational or safety standpoint. The circuit is tested functionally through the continuity of the turbine trip slave relay which satisfies Technical Specification requirements. This report is being submitted because we have not fully tested the time response of the turbine trip - reactor trip feature from Steam Generator level high-high as required by Technical Specification surveillance requirements.

UICENSEE EVENT REPORT (LER) TEXT CONTINUAT													N		U.S. NUCLEAR REGULATORY COMMISSION APPROVEL ONB NO. 3150-0104 EXPIRES: 8/54/88									
FACILITY NAME (1)						DOCKET NUMBER (2)						T	L	ER N		p		PAGE (3)						
												YEAR	T	SEC	UMBER	1	NUMBER	-	T					
D.C. Cook N	Nuclear	Plant,	Unit	1		0	15	0	0	0  3	1  5	81	6 -	-0	1017	-	0 0	0 3	OF	0	13			

Cause of Event

The probable cause of the event was that the procedure was based on the requirements of the Technical Specification bases which do not require verification of this response time. It therefore went undetected until the line-by-line Technical Specification review which was conducted to find discrepancies of this type.

## Corrective/Preventive Action

A revision to Technical Specifications has been proposed which will delete this particular time response verification. This proposed revision will make the Unit 1 and Unit 2 Technical Specifications consistent in this regard. Because it does not jeopardize safe operation or shutdown of the Plant, no further action has been taken.

# Previous Similar Events

None