NRC Form 366 19-83) LICENSEE EVENT REPORT (LER)														U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/85										
FACILITY NAME (1)												DOCKET NUMBER (2) PAGE (3)												
	F	rain	rie	Isl	an	dI	Uni	it	1							0 11	1 06	012						
TITLE (4	Bot	h D	iese	el G	en	er	ato	ors	s Star	ted N	When R	elay	Acti	uat	ion Cause	d I	055 0	E O	me					
	Sou	rce	of	Off	si	te	Po	we	er.															
EVENT DATE (II) LER NUMBER (6) REPORT DATE (7) OTHER S													FACIL	ITIES INVO	LVE	ED (8)								
MONTH	DAY	YEAR	R Y	EAR		SEGU	UENT	AL	REVISI	ON MON	TH DAY	YEAR			FACILITY NA	MES		00	DOCKET NUMBER(S)					
		-										T	Pra	air	ie Island	Ur	it 2	0 15 10 10 10 13 10 16						
0 3	2 8	8	6 8	6	-	0	0	4	- 0	001	4 2 5	8 6					1	0	151010	101				
OPI	RATING	1	TH	IS REPO	ORT	15 81	UBM	TTE	D PURSUAN	T TO TH	E REQUIRE	MENTS OF	10 CF	A 5 10	Check one as more	of the	following) (1	1)						
	002 (8)	1	- R	20.40	02(6)			-	20.	408(c)			X	60.73(a)(Z)(iv)			-	73.71(b)					
LEVE	R		-	20.406(a)(1)(i)				+	60.	36(c)(1)			-	50.73(e)(2)(v)			-	73,7%(c)	6)					
(10)	10	1010	20.406(a)(1)(ii)					-	50.	50.36(c)(2)			-	50,73(a)(2)(vii)			-	DIMER (So	n Text, NR	Text, NRC Form				
			H	- 20.44	20.406(4)(1)(1)				-	- 50	50.73(a)(2)(i)			-	50.73(a)(2)(viii)(A			306A/					
		-			20.406(s)(1)(iv)				H	- 80.	73(e)(Z)(a)				60.7318/(2/(VIH))			Ι.						
				20.4	0018	111114	"			80.	3(a)(2)(iii)			1.12	BU./3(8/(2/(8)			-						
NAME					-					LICEN	EE CONTA	LI FUR IN	no cen	111.45		T		TEL	EPHONE NUM	BER				
																A	REA CODE	1						
	Δ	rne	AI	luns	ta	d.	St	af	f Eng	inee	-					6	1112	13	18181-	1111	12 11			
				iuno		co	MPL	ETE	ONE LINE F	OR EAC	COMPONE	NT FAILU	RE DES	CRIBE	D IN THIS REPOR	AT (13	1-1-	1.0	10101	1-1-	1- 1-			
		T				ANU	FAC		REPORTAR	E			T			T	AND ISAC	T	cooperate of					
CAUS'.	SYSTEM	COM	MPONE	NT	1	TUR	ER		TO NPRO	S		CAU	SEST	STEM	COMPONENT	1.	TURER	ľ	TO NPRDS	×				
		1				-							-			+		+		******	*****			
i		1	1	1		1								1.1	1.1.1		1.1.1							
															and the second second second second	1	termine dan serie dan	1						
	11	1	1											1	111			1						
_						SI	UPPL	EME	NTAL REPO	AT EXPE	CTED (14)						EXCEPT	ED.	MONTH	DAY	YEAR			
																SUBMISSION								
YE	S (If ym,	complete	EXPE	CTED S	UBN	115510	OND	ATE	7	Reference and an	X NO					-								
	S	gener	ce o	of o prs.	ff	si Ca	te us:	po e o	ower and the	nd au rela	itomat ay act	ic st uatio	arti n wa	ing as 1	of both not found	saf	eguard	is	diesel					
	86(PDF S	250	100	94 CK	80	504	42 PI	5 28 28	2										ĪĒ	22				

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES 8/31/85

DOCKET NUMBER (2)						LER NUMBER (6)							
말 잘 다 같이 많을 수 있는				SEQUENTIAL REVISION NUMBER NUMBER			Za		Γ	•			
2 8 1	2 8	16	-	01	0 4	-	010	0	2	OF	0	12	
1	2 8	2 8 2 8	2 8 2 8 6	2 8 2 8 6 -	2 8 2 8 6 - 0	2 8 2 8 6 - 0 0 4	2 8 2 8 6 - 0 0 4 -	28286 - 004 - 00	2 8 2 8 6 - 0 0 4 - 0 0 0	2 8 2 8 6 - 0 0 4 - 0 0 0 2	2 8 2 8 6 - 0 0 4 - 0 0 0 2 OF	2 8 2 8 6 - 0 0 4 - 0 0 0 2 OF 0	

On March 28, 1986 Unit 1 was at cold shutdown for refueling and Unit 2 was at 100% power. Substation configuration was normal.

At 1321 the sudden pressure relay (RLY) for CT1 Transformer (XFMR) actuated, clearing 345KV Bus 1 (BU) sources and loads to allow isolation of CT1 Transformer via the CT 345KV MOD, 13.8KV ACB 1H4 and 13.8KV OCB 1H5. 345KV ACBs 8H7, 8H13 and 8H16 opened, deenergizing 345KV Bus 1, and 34.5KV GCBs 2RSX and 2RSY opened, deenergizing 2RX and 2RY Transformers. Both safeguards diesel generators (DG) started automatically but did not load since offsite power was still available to the safeguards buses. Bustie Breaker (BKR) 16-8 closed, restoring power to Bus 25, and Breaker 26-1 from CT12 Transformer closed, restoring power to Bus 26. Breaker 8H7 reclosed automatically following CT1 isolation, reenergizing 345KV Bus 1.

After some investigation, 2RX and 2RY Transformers were reenergized at about 1554 and Bus 25 and 26 were returned to 2RY Transformer. CTl Transformer remained isolated for further investigation. Due to scheduling considerations with the Unit 1 refueling outage, CTl was restored on April 11.

Doble testing of CT1 Transformer revealed no problems. Investigation revealed that the sudden pressure relay isolation valve was closed. Routing maintenance was performed on CT1 in February and the sudden pressure relay was checked as part of that maintenance. Apparently the isolation valve was left closed at that time. With the isolation valve closed, the relay should not have actuated. The isolation valve was opened and the relay was checked for proper operation; no reason for its actuation was found. The event was discussed with involved personnel. Investigation is continuing; if further information is found as to the cause of the relay actuation, a supplemental report will be submitted.

The Safeguards Bus Voltage Restoration Schemes responded properly to the loss of voltage on each safeguards bus. Both Buses 25 and 26 remained powered by offsite sources which could adequately support safe shutdown loads. Thus, there were no consequences from the standpoint of public health and safety.

NRC Form 366A



Northern States Power Company

IE22 111

414 Nicollet Mall Minneapolis, Minnesota 55401 Telephone (612) 330-5500

April 25, 1986

U S Nuclear Regulatory Commission Document Control Desk Washington, DC 20555

> PRAIRIE ISLAND NUCLEAR GENERATING PLANT Docket Nos. 50-282 License Nos. DPR-42 50-306 DPR-60

Both Diesel Generators Started When Relay Actuation Caused Loss of One Source of Offsite Power

The Licensee Event Report for this occurrence is attached.

This event was reported via Emergency Notification System per 10 CFR Part 72 on March 28, 1986.

Eugene Eckholt

for David Musolf Manager - Nuclear Support Services

DMM/EFE/efe

c: Regional Administrator-III, NRC NRE Project Manager, NRC Resident Inspector, NRC MPCA Attn: F W Ferman

Attachment



1