RC FOR	N 366 .		and the second	U.S. NUCLE	AR REC	BULATORY C	OMMIS	SION	anner all allere		EXPERES O	6/30/96			
	1	LICEN	(See rever	ENT REP	mber o				INFORMAT LEARNED A TO INDUST INFORMAT NUCLEAR	ION COLLEG	ER RESPONSE TO TION REQUEST: RATED INTO THE RD COMMENTS R RECORDS MANA Y COMMISSION, I CREDUCTION I L REDUCTION	BO.0 MORS. LICENSING P EGLARDING BU GEMENT BR WASHINGTON PROJECT (3)	REPOI ROCESS IRDEN ES ANCH (1. DC 20 160-0104	AND FE	ESSONS D BACK TO THE S). U.S.
AC BITY N		Read of the rest of the	And a state of the		ATT CREAMENTS	Cargonina an official and a cardina	an the root was	n Eldenin oper	BOCKET N	UMBER 121	0.4.6.1			OF 5	
linton	Powe	r Static	m							0500	0461			UF D	
		culation set Ger		al Generator	Cont	rol Circuitr	y Res	ults in	Insuffi	cient Vo	bitage and i	noperabi	lity of	the	
EVEN	TAGT	E (5)	L	ER NUMBER LE	1)	A REPOR		(7)	FACILITY	DD	HER FACILITTE	SINVOLV	ED (8)	IRER .	
HTNON	DAY	VEAR	YEAR	NUMBER	NUMB		DAY	YEAH	None			000	05000		
01	31	\$3	98	006	- 00	0 03	02	98	and the second s	IUTY NAME		Dec	RET NUS	STATE OF THE R. LANS.	
01	31	30	30	000	~		02	00	None	ne			05000		
OPERA	TING	and the second se	THIS REP	ORT IS SUBMIT	TED PU	RSUANT TO T	HE REC	UIREM	ENTS OF	10 CFR 8:	(Check one o	r mora) (1	12	and the second of	
MOD	MODE 191		100.0	201/61		100 2203	1-1/21/-	1	150.73(a)(2)(i)			50.73(a)(2)(viii)		viii)	
POWER LEVEL (10) 000		20.2201(b) 20.2203(a)(1)			20.2203(e)(2)(v) 20.2203(e)(3)(l)			X				50.73(a)(2)(x)			
										60.73(a	(5100)		73.71		
		000	20.2203(a)(2)(i) 20.2203(a)(2)(ii) 20.2203(a)(2)(ii) 20.2203(a)(2)(iii)			20.2203(a)(3)(ii) 20.2203(a)(4) 50.36(c)(1)			50.73(a)(2)(v) 50.73(a)(2)(v)			×	X OTHER Specify in Abetract belo		
												Sp			
		ALL ALL	-	203(a)(2)(iv)		50.36(c)(2)				50.73(a)(2)(vii)			or in NRC Form 366A		
		and the standard standard		CONTRACTOR AND A CONTRACTOR	LIG	ENSEE CONT	ACTE	OR TH	IS LER 11	2)	MEER Include Are	- Codel			
M. G.	McMe	enamin,	Design	Engineer							7) 935-888		nsion	3469)
	NAMES AND ADDRESS OF TAXABLE PARTY.		COMP	LETE ONE LINE		LACH COMPO	NENT	FAILUF	RE CESCA	IBED IN T		131			
CAUS	e s	YSTEM	COMPON	ENT MANUFAC	TURER	REPORTABLE TO NPRDS		CA	USE	SYSTEM	COMPONENT	MANUFAC	TURER		NPRDS
			<u> </u>												
elementa da la constanta da const	an a the second s		SUPPLEME	NTAL REPORT	EXPE	CTED (14)				SUB	ECTED MISSION TE (15)	MONTH	DA	Y	YEAR
	yes, ca			UBMISSION D			XA			(1.0)	and the second se				Alertos
ABSTR	Wit	h the liscre	plant pancy i	in Mode 4 in a Direction cables	(CO t Cu	LD SHUTD	OWN) ltage overe	and cal	the si culat: The ci	ixth re	the Div	ision 1 not inc	Die	the	8

existence of two cables that are fun to the Remote Shutdown Faher and their effect of the voltage drop to the Diesel Generator control panel. Preliminary evaluation of the impact of this error indicated that there has not been sufficient voltage to the control panel for all of its connected equipment. The cause for this event was due to an erroneous calculation provided by Sargent and Lundy design engineers. The engineer failed to include the cable lengths from and to the Remote Shutdown Panel in the voltage drop calculation for the Diesel Generator Control Panel. This error was overlooked in several subsequent revisions. The corrective actions for this event are to restore the Diesel Generator control panel voltage to an acceptable level, to review other voltage calculations to ensure no other circuits omitted cabling, to review the control circuits that pass through the Remote Shutdown Panel and to brief engineering personnel on lessons learned from this event. This issue is also reportable under 10CFR21.

9803040126 PDR ALOCK	980302 05000461 PDR
-------------------------	---------------------------

U.S. NUCLEAR REGULATORY COMMISSION

(4-95) LICENSEE EVENT REPORT (LER) TEXT CONTINUATION LER NUMBER (S) FACILITY NAME (1) DOCKET PAGE (3) VEAR SEQUENTIAL NUMBER NUMBER Clinton Power Station 98 006 00 OF 05000461 2 5

TEXT III more space is required, use additional copies of NRC Form 366A) (17)

DESCRIPTION OF EVENT

NRC FORM 366A

On January 31, 1998, the plant was in Mode 4 (COLD SHUTDOWN) for the sixth refueling outage (RF-5), and reactor [RCT] coolant temperature was being maintained within a band cf 95 to 115 degrees Fahrenheit (F) and pressure was zero pounds per square inch. A review was being performed by Niclear Station Engineering Department personnel of the Emergency Diesel Generator (EDG)[EK] control circuitry due to a supplier deficiency noted with the air start solenoids [SOL]. During this review, a discrepancy was identified with a calculation for the Direct Current (DC) feed to the Division I DG control panel (1PL12JA)(FL). The identified error was that certain cables [CBL3], which comprise part of the normal DC feed to the Division I DG control panel. An informal calculation of the resistances and voltage drop, with the additional cable lengths included, indicated that the acceptance criteria of the original calculation had not been met. Condition Report (CR) 1-98-01-452 was written to address this issue.

The control power feed to 1PL12JA has an alternate feed through the Remote Shutdown Panel (RSP). This alternate feed is routed through a transfer switch located on the RSP. On February 4, 1990, during the roview of the voltage requirements for 1PL12JA, it was noted that the amperage specifications for the transfer switch on the RSP were not adequate for the revised calculation. CR 1-98-02-060 was written to address this inadequaty.

The normal 125 volts (V) DC power is supplied to panel 1PL12JA through a feed cable (1DG01N) from the DC Motor Control Center (MCC). This cable is a three conductor, 1/0 gauge utilizing two of the conductors to carry the current to the panel. This cable choice appears to have treated the voltage drop issue as the limiting factor for this circuit and, at the time of installation, provided some margin in the design.

In order to provide the capability to transfer to an alternate source of DC power, the normal feed circuit passes through the remote shutdown panel. This was accomplished by the addition of cables to and from the RSP, connecting through the transfer switch. This cabling (IDGOIM and IDGOIP) consists of #6 American Wire Gauge (AWG) size conductors. This part of the design failed to adequately evaluate the loade that existed at the diesel panel versus the rating of the transfer switch contacts (20 Amps (A) continuous, 6.5A inductive interrupting). At the time of the initial design, the maximum continuous load of the panel would have been approximately 14A but the inductive load represented by two DG lube oil motors [MO] would have been approximately 12.6A. The cable selection was acceptable from an ampacity viewpoint but the voltage drop was not considered. The original design current of 14A results in a voltage drop that does not meet the criteria of Sargent and Lundy standards with respect to motor feeds.

The resultant configuration should have been included as part of calculation 19-D-28, "Review of Division I DC System 1A," (Revision 0 dated June 6, 1985). This calculation recognized that the DC lube oil motors at the dissel would be running. In the "voltage drop in DC feeder circuits" portion of calculation 19-D-28, the feeder circuit breaker to 1PL12JA was evaluated, but only with the three conductor 1/0 cable (1DG01N) included and not the alternate feed through 1DG01M and 1DG01P. Based on the calculation layout, the preparer of calculation 19-D-28 was most likely working from the MCC key diagram and wiring NRC FORM 366A

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT (LER)

TEA	CON	INUAI	ION

DOCKET		LER NUMBER (61	E	AGE 13	
	YEAR	SEQUENTIAL NUMBER	NUMBER			
05000461	98	006	00	3	OF	5
	05000461	05000461 98	YEAR SEQUENTIAL NUMBER	NUMBER NUMBER	YEAR SEQUENTIAL REVISION NUMBER NUMBER	YEAR SEQUENTIAL REVISION NUMBER NUMBER

TEXT III more spece is required, use additional copies of NRC Form 386A/ 1171

diagram as well as the cable database. These documents show 1DGO1N feeding 1PL12JA and terminating there. In order to identify the additional cables involved, it would be necessary to go to the wiring diagrams for the DG panel and the RSP, but there was nothing to specifically lead the preparer to lock at these documents.

After issuance of the initial calculation, it was unlikely that the error would be subsequently identified since changes would tend to be loads and this would only lead to revision of the numerical values and not re-verification of the base data. This proved to be the case when the diesel pre-lubrication modification (DG-OS, Engineering Change Notice 6644, dated November 19, 1985) was issued. In revision three of the calculation (issued February 18, 1987), the battery loading was revised to reflect the two new DC motors at the diesel. However, the "voltage drop in DC feeder circuits" was not revised. When this modification was issued, the transfer switch at the RSP was not re-examined for the additional load of two DC motors which took the continuous load to 26A, which is greater than the switch rating of 20A.

In revision eight to calculation 19-D-28 (issued June 15, 1991), the voltage drop portion was redone. This revision reformatted the calculation and examined not only the voltage drop to 1FL12JA, but also examined the addition of the two DC motors for pre-lubrication. However, the revision failed to identify the cabling through the RSP and so the acceptance values achieved were in fact not present at the motor terminale. As a result of this error, the Division I DG control panel would not have had sufficient voltage to operate all connected equipment in the event of an actual lose of Alternating Current(AC) power start.

No automatic or manually initiated safety system responses were necessary to place the plant in a safe and stable condition. This event was not effected by other inoperable equipment or components.

CAUSE OF EVENT

There was an initial error in the preparation of calculation 19-D-28 provided by Sargent and Lundy, which was not detected in the initial review nor subsequent revisions to the calculation. This error was further aggravated by the improper selection of a transfer switch that was marginal for its purpose and not further evaluated during later design changes.

CORRECTIVE ACTION

The corrective actions for this event includes a resolution to the voltage level required at the DG control panel. DC calculations for the safety-related batteries will be reviewed to ensure no other circuits emitted cabling. The control circuitry that runs through the RSP will also be analyzed for other components to ensure proper voltage/amperage is adequate for their operation. Engineering personnel will be briefed on the lessons learned from this event. U.S. NUCLEAR REGULATORY COMMISSION 14-951 LICENBEE EVENT REPORT (LER) TEXT CONTINUATION

EACILITY NAME (1)	DOCKET		LEB NUMBER U	51	F	AGE (3	1
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
Clinton Power Station	05000461	98	006	00	4	OF	5

TEXT IIf more space is required, use additional copies of NRC Form 366A) (17)

ANALYSIS OF EVENT

This event is reportable under the provisions of lOCFR50.73(a)(2)(ii)(B) due to the plant being in a condition outside of its design basis in that there could have been insufficient voltage to the DG equipment fed by the control panel such that they would not have performed their design function.

An assessment of the safety consequences and implications of this event concluded that this event was safety significant. In the event of a true Loss of Alternating Current (AC) Power situation, the lack of sufficient voltage could have had significant adverse effects to the operation of the Division I DG up to and including failure of the diesel to start within the required time period. This condition has existed since initial plant licensing. This cabling configuration only effects the Division I Diesel Generator. The DC feed to the Division II and III Diesel Generators do not have the extra cabling that runs through the RSP.

ADDITIONAL INFORMATION

No equipment or components failed during this event. Illinois Power has not reported any events of insufficient DC voltage or underrated control switches in the past two years.

For further information regarding this event, contact M. G. McMenamin, Nuclear Station Engineering, at (217) 935-8881, extension 3469

10CFR, FART 21 REPORT 21-98-014

On January 31, 1998, a review of the D' power to the control circuitry of the DG control panel revealed that there was an error in the base calculation for the required voltage to DC equipment associated with the Division I DG. This error in preparing the calculation should be reported under the provisions of 1005R21. The backs for the reportable conclusion is that the lack of sufficient voltage to the DG equipment could have adverse effects to its operation when required.

- Walter G. MacFarland, IV, Chief Nuclear Officer of IP, Clinton Power Station, Highway 54, 6 miles East, Clinton, Illinois, 61727, is informing the Nuclear Regulatory Commission of a condition reportable under the provisions of 10CFR, Part 21.
- (ii) The activity involved in this condition is the calculation 19-D-28, "Review of Division I DC System 1A," (Revision 0 dated June 6, 1985).
- (iii) This calculation was prepared and supplied by Sargent & Lundy.
- (iv) The error in the calculation was a failure to analyze the cabling run from the DG control panel to the Remote Shutdown Panel for its impact on the minimum required voltage required for operation of the DC loads off of the DG control panel. Insufficient voltage to this equipment could result in failure for the equipment to perform its desired function when required by plant conditions.

RC FOR	M 366A LICENSEE EVENT	REPORT (I		U.S. NUCLEAR				
	TEXT CONT							
and the second second	FACILITY SAME (1)	DOCKET	-	LER NUMBER (E) REVISION	P	AGE	
linton	Power Station	05000461	YEAR 98	NUMBER 006	MUMBER 00	5	OF	!
EXT In	more space is required, use additional contes of NRC Form 366A.	(17)	<u> </u>	C. Methics		Concession of the		
V)	The error in this calculation was discov be potentially reportable under the prov transfer switch was identified on Pebrua potentially reportable due to the error	risions of 10 ry 4, 1998, in the calcu	and walatic	as determ on 19-D-28	ined to	be	r.ne	
vi)	The scope of the generic applicability of A review of DC calculations will be per- configurations that pass through the Rem	formed and an note Shutdown	n ana. n Pane	lysis of o el will be	pe.for	med.		G
(vii)	Corrective actions being taken by Illin action section of the LER portion of th	ois Power ar is report.	e dis	oussed in	the cos	rrect	ive	
(viii	.) IP has no advice for other purchasers	or licensees	rega	rding this	issue			
i								
L	FORM 300A (4.95)	Carallel House and the second s			-	and the second s		-

.

POWER	REACTOR				E	VENT NUMBE	R: 33	819
UNIT:	.ITY: CLINTC [1] [PE: [1] GE-] []		REGION: STATE: II	L NO EV	TIFICATION TIFICATION ENT DATE:	TIME:	14:25 [ET] 03/02/98
	OTIFIED BY: S OFFICER:					ENT TIME: ST UPDATE		14:25[CST] 03/02/98
	ENCY CLASS:					NOTI	FICATI	ONS
10 CF	R SECTION: 21.21			ED PARAGRAPH	VE	RN HODGE (PCEB)	NRR
UNIT	SCRAM CODE	RX CRIT	INIT PWR	INIT RX MOI	DE	CURR PWR	CURR	RX MODE
1	N	N	0	COLD SHUTDOW	WN	0	COLD	SHUTDOWN

EVENT TEXT

INCORRECT CALCULATION IN DIESEL GENERATOR CONTROL CIRCUITRY RESULTS IN INSUFFICIENT VOLTAGE AND INOPERABILITY OF THE DIVISION 1 DIESEL GENERATOR.

A DISCREPANCY IN A DIRECT CURRENT VOLTAGE CALCULATION FOR THE DIVISION 1 DIESEL GENERATOR CONTROL CIRCUITRY WAS DISCOVERED. THE CALCULATION DID NOT INCLUDE THE EXISTENCE OF TWO CABLES THAT ARE RUN TO THE REMOTE SHUTDOWN PANEL AND THEIR EFFECT ON THE VOLTAGE DROP TO THE DIESEL GENERATOR CONTROL PANEL. PRELIMINARY EVALUATION OF THE IMPACT OF THIS ERROR INDICATED THAT THERE HAS NOT BEEN SUFFICIENT VOLTAGE TO THE CONTROL PANEL FOR ALL OF ITS CONNECTED EQUIPMENT. THE CAUSE FOR THIS EVENT WAS DUE TO AN ERRONEOUS CALCULATION PROVIDED BY SARGENT AND LUNDY DESIGN ENGINEERS. THE ENGINEER FAILED TO INCLUDE THE CABLE LENGTHS FROM AND TO THE REMOTE SHUTDOWN PANEL IN THE VOLTAGE DROP CALCULATION FOR THE DIESEL GENERATOR CONTROL PANEL. THIS ERROR WAS OVERLOOKED IN SEVERAL SUBSEQUENT REVISIONS. THE CORRECTIVE ACTIONS FOR THIS EVENT ARE TO RESTORE THE DIESEL GENERATOR CONTROL PANEL VOLTAGE TO AN ACCEPTABLE LEVEL, TO REVIEW OTHER VOLTAGE CALCULATIONS TO ENSURE NO OTHER CIRCUITS OMITTED CABLING, TO REVIEW THE CONTROL CIRCUITS THAT PASS THROUGH THE REMOTE SHUTDOWN PANEL AND TO BRIEF ENGINEERING PERSONNEL ON LESSONS LEARNED FROM THIS EVENT. LER WAS SUBMITTED BY THE LICENSEE.