NRC Form 366

U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104

EXPIRES 8/31/88

LICENSEE EVENT REPORT (LER)

			-	-	-	-	-	-	-	-	-	-	-				_			_	_	-	_	_	_	_	-	-	-	-	-
ACILITY	NAME	(1)			F	err	ni	2													0	OCKE	TNUN	MBER	(2)		h .	1	PA	GET	3)
TITLE (4)	In in	ad 1 R	lve	ert	an	t I on	Los	ss	of to	Po	owe	r t cal	to le	Reac	tor	Buil ESF	dir Ad	ng Exi	haus ions	t	Rad	iat	io	n M	lon	ito	r	Res	sult	s	13
EVENT DATE (5) LER NUMBER (6)					REPORT DATE (7) OTHER								FACILITIES INVOLVED (8)																		
MONTH DAY YEAR			A.FI	YE	AR		SEQUENTIAL REVISION				MONTH	DAY	FACILITY NAMES							DOCKET NUMBER(S)											
		T						NUMBER								1			1.11			22	1		0	151	0	0	0	1	1
0 7	0	5	8	6	8	6 - 0 1 8 - 0 0 0 7 3 1 8 6									0 15 10 10 10 1 1																
OPE	RATIN	G			THE	S RE	PORT	ISS	UBM	ITTE	D PUI	ASUA	NT 1	O THE R	EQUIREN	MENTS C	F 10	CFR § /C	heck one	or	nore al	t the t	0//0++1	ng) (1	1)				-		-
POWE				20.402(b) 20.405(a)(1)(i)					20.405(c) X 50.73(e)(2)(iv						(iv) (v)				73.71(b) 73.71(c)												
) 20.405(a)(1)(ii) 20.405(a)(1)(iii) 20.405(a)(1)(iv)					50.36(c)(2) 50.73(a)(2)(vii)						(vii)	Ē			OTHER (Specify in Abstract below and in Text, NRC Form						st prm						
									50.73(a)(2)(ii)					50.73(e)(2)(viii)(B)							.100										
					1	20.	405(a)(1)(¥)			_	_	50.73(a)(2)(iii) 50.73(a)(2)(x)							(x)				1		_				
NAME													-	ICENSEE	CONTAC	TFOR	THIS	ER (12)				-			TEI	EBUO	NE N				
	Lew	is	F	•	Bre	egr	ni,	C	om	pli	an	ce	En	gine	er							A	REAC	00E	6	.8	6	CIVIDI	5.2		1.2
								co	OMPL	ETE	ONE	LINE	FOR	EACH C	OMPONE	NT FAIL	URE	DESCRIBE	D IN TH	IS F	EPOR	T (13)	-	13	12	0	0	- 1	20	1	15
CAUSE	CAUSE SYSTEM COMPONENT					NT MANUFAC REPORTABI TURER TO NPROS			BLE			CA	USE:	SYSTEM	COMPONENT		ENT	MANUFAC- TURER			REPOR TO N		TABLE								
	1			1	1				1	1									1	1	1			1				T			******
				1	1	1		1	1	1									1	1	1			1				T			
				-	-			5	UPP	LEME	ENTA	. REP	PORT	EXPECT	ED (14)					-	-	-					MO	NTH	DAY	T	EAR
YE	5 i/f ye	es. co	mp)	ere t	XPE	CTED	SUB	MISS	ION	DATE	Ð			-	X NO							1	SUE	ATE IN	ED ION				1		1

At 1634 hours on July 5, 1986, while in Operational Condition 5 (Refueling) for a scheduled maintenance and modification outage, power to the Reactor Building ventilation exhaust gaseous effluent radiation monitor was lost actuating several Engineered Safety Feature (ESF) systems. Actuation of an ESF system is a reportable event. Investigation by operations personnel revealed the cause to be the result of Motor Control Center (MCC), 72B-4C, position 1CR, de-energizing.

It was discovered that the MCC 72B-4C, position 1CR, disconnect switch was not fully in the ON position. It is only possible for this position to be de-energized by either manually turning the switch from the ON position or by blowing a fuse. All fuses were found to be acceptable. The exact cause of the switch being turned from the ON position is not known. A drywell Health Physics (HP) and Security access point had been established near the MCC location, and it is likely that plant personnel may have accidently bumped or leaned on the switch. The access point was relocated to prevent recurrence.

8608050231 860731 PDR ADOCK 05000341 S PDR

NRC Form 366A (9-83)	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION										ULATORY COMMISSION MB NO. 3150-0104 /88					
FACILITY NAME (1)		DOCKET NUMBER (2)	DOCKET NUMBER (2)							PAGE (3)						
1. 1. 1. 1. 1.			YEAR		SEQUENTIA	-	NUMBER		T	1	-					
	Fermi 2	0 5 0 0 3 4	1 816	_	0111	3 -	010	0 12	OF	0	13					

TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1634 hours on July 5, 1986, while in Operational Condition 5 (Refueling) for a scheduled maintenance and modification outage, the control room received a "Division I/II Reactor Building Ventilation Exhaust Radiation Monitor Upscale Trip" alarm. Annunciation of this alarm window indicates a high (upscale) radiation alarm and trip of the Reactor Building ventilation exhaust gaseous effluent radiation monitor. Investigation by operating personnel revealed the cause to be the result of Motor Control Cabinet (MCC), 72B-4C, position 1CR, being de-energized; not an actual radiological release. An upscale failure of this radiation monitor upon a loss of power is in accordance with design criteria.

With the de-energization of the lCR position of MCC 72B-4C and the resulting upscale trip of the Reactor Building exhaust radiation monitor, the following Engineered Safety Feature (ESF) actuations occurred (Actuation of an ESF system is a reportable event):

- 1. Division I Standby Gas Treatment (SGTS) system auto-started.
- Division II Control Center heating, ventilation and air conditioning (HVAC) went into recirculation mode.

The following indications and actuations also occurred as a result of the MCC de-energizing:

- 1. Reactor Building HVAC system tripped.
- "Fire Alarm Panel Trouble" alarms for panels H11-P916A, P82-P430, P80-P101 and P80-P102 were received.
- Division 1 Control Center HVAC damper indications were lost.
- Operations and Security radio transmitters de-energized affecting site communications.

After the incident the disconnect switch was reset to the ON postion and all the affected systems and equipment were returned to normal operation without incident.

A plant operator dispatched to MCC 72B-4C found that the disconnect switch for position 1CR was not fully in the ON position. This MCC position is a fusible disconnect switch which is either ON or OFF; there is no tripped position. The only way for position 1CR to become de-energized is to manually move the switch from the ON position or to have a blown fuse. Inspection found all fuses to be acceptable.

NRC Form 366A (9-83)	LICENSEE EVENT REPORT (LER) TEXT CONTINUATION											MB NO. 3	LATORY COMMISSION 8 NO. 3150-0104 8				
FACILITY NAME (1)	0	DOCKET NUMBER (2)							RNU	MBER (6)			PAGE (3)				
								YEAR		SEQU	MBER		REVISION NUMBER				
Fermi 2		0 5	0	0	0	31	4 [1	816	-	01	1 8	_	010	013	OF	0	13

TEXT (If more space is required, use additional NRC Form 366A's) (17)

The MCC that powers the Reactor Building exhaust radiation monitor, responsible for the actuations, is physically located directly outside the drywell equipment hatch opening. To support the on-going outage work being completed inside the drywell, Health Physics (HP) and Security personnel established a personnel access point that was near the MCC location. The definite cause of this event is indeterminate. However, the probable cause is that the switch was accidently bumped or leaned against, and was moved slightly from the ON position. To prevent recurrence, the personnel access point to the drywell was relocated to preclude personnel from walking near the MCC.

The safety consequences of this event are miminal. The majority of equipment supplied by MCC 72B-4C, position 1CR, are Balance of Plant systems. The Reactor Building HVAC, Contol Center HVAC and SGTS system actuations and isolations caused by the MCC position de-energizing are conservative responses designed to protect plant personnel during a postulated radiological event in the Reactor Building.

All of the above system actuations and isolations that occurred in this event are designed to do so on a loss of power. These actuations and isolations were not detrimental to the affected equipment, nor did they reduce the availability of plant safety systems. Robert S. Lenart Plant Manager



Fermi-2 6400 North Dixie Highway Newport, Michigan 48166 (313) 586-5201

July 31, 1986 NP860378



Nuclear Operations

U. S. Nuclear Regulatory Commission Attention: Document Control Desk Washington, D.C. 20555

Gentlemen:

- Reference: Fermi 2 NRC Docket No. 50-341 Facility Operating License No. NPF-43
- Subject: Transmittal of Licensee Event Report 86-018-00

Please find enclosed LER No. 86-018-00, dated July 31, 1986, for a reportable event that occurred on July 5, 1986. As indicated below, a copy of this LER is being sent to the Administrator Region III.

If you have any questions, please contact us.

Sincerely,

Pl Lent

R. S. Lenart Plant Manager

Enclosure: NRC Forms 366, 366A

cc: M. D. Lynch W. G. Rogers

> Regional Administrator USNRC Region III 799 Roosevelt Rd. Glen Ellyn, IL 60137

Wayne County Emergency Management Division 1250 Middlebelt Road Detroit, MI 48242

Director/Coordinator Monroe City-County Office of Civil Preparedness 965 South Raisinville Road Monroe, MI 48161