



Carolina Power & Light Company
P.O. Box 10429
Southport, NC 28461-0429

FEB 20 1998

SERIAL: BSEP 98-0011

U. S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, DC 20555

BRUNSWICK STEAM ELECTRIC PLANT, UNIT NOS. 1 AND 2
DOCKET NOS. 50-325 AND 50-324/LICENSE NOS. DPR-71 AND DPR-62
WITHDRAWAL OF REQUEST REGARDING INTERGRANULAR STRESS CORROSION
CRACKING INSPECTION PLAN FOR REACTOR WATER CLEAN-UP SYSTEM PIPING
WELDS (NRC TAC NOS. M98742 AND M98743)

Gentlemen:

By letter dated May 2, 1997 (Serial: BSEP 97-0108), Carolina Power & Light (CP&L) Company submitted a request regarding intergranular stress corrosion cracking (IGSCC) inspections of Reactor Water Clean-Up (RWCU) system piping for the Brunswick Steam Electric Plant (BSEP), Unit Nos. 1 and 2. Specifically, CP&L requested NRC concurrence with plans to remove the piping welds outboard of the second containment isolation valve in the RWCU system from the inspection extent and frequency outlined in Generic Letter 88-01, "NRC Position on IGSCC in BWR Austenitic Stainless Steel Piping." Removal of these welds from the inspection scope was based on meeting the following criteria:

1. The piping outboard of the second containment isolation valve being made of resistant material.
2. Absence of IGSCC indications in RWCU system piping welds inboard of the second isolation valves.
3. Absence of IGSCC indications in RWCU system piping welds outboard of the second isolation valves after inspecting a minimum of 10 percent of the susceptible piping welds.

An additional criterion established during NRC review of a similar request for Philadelphia Electric Company'simerick Generating Station was completion of all actions outlined in Generic Letter 89-10, "Safety-Related Motor-Operated Valve Testing and Surveillance," for the RWCU system containment isolation valves.

9803030127 980220
PDR ADOCK 05000324
G PDR



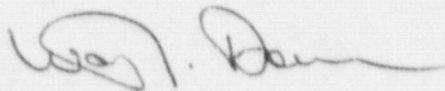
110
1001

Document Control Desk
BSEP 98-0011 / Page 2

On October 27, 1997, the NRC issued Inspection Report 97-11 for BSEP, Unit Nos. 1 and 2. The scope of inspections covered by this report included a review of CP&L's implementation of Generic Letter 89-10. In Inspection Report 97-11, the NRC concluded there was inadequate justification for valve thrust settings for Anchor Darling double disk gate valves (i.e., such as the RWCU system containment isolation valves). Therefore, as a result of the NRC findings documented in Inspection Report 97-11, CP&L is withdrawing the May 2, 1997, request.

Please refer any questions regarding this submittal to Mr. Warren J. Dorman, Supervisor - Licensing, at (910) 457-2068.

Sincerely,



Keith R. Jury
Manager - Regulatory Affairs
Brunswick Steam Electric Plant

WRM/wrm

Document Control Desk
BSEP 98-0011 / Page 3

cc: U. S. Nuclear Regulatory Commission, Region II
ATTN: Mr. Luis A. Reyes, Regional Administrator
Atlanta Federal Center
61 Forsyth Street, SW, Suite 23T85
Atlanta, GA 30303

U. S. Nuclear Regulatory Commission
ATTN: Mr. Charles A. Patterson, NRC Senior Resident Inspector
8470 River Road
Southport, NC 28461-8869

U. S. Nuclear Regulatory Commission
ATTN: Mr. David C. Trimble, Jr. (Mail Stop OWFN 14H22)
11555 Rockville Pike
Rockville, MD 20852-2738

The Honorable Jo A. Sanford
Chairman - North Carolina Utilities Commission
P.O. Box 29510
Raleigh, NC 27626-0510

Division of Boiler and Pressure Vessel
North Carolina Department of Labor
ATTN: Mr. Jack Giver., Assistant Director of Boiler & Pressure Vessels
4 West Edenton Street
Raleigh, NC 27601-1092