NRC FORM 366 (5-92) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3[50-0104 EXPIRES 5/31/95

LICEN E EVENT REPORT (LER)

(See reverse for required number of digits/characters for each block)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)
WOLF CREEK GENERATING STATION

DOCKET NUMBER (2) PAGE (3) 05000482 1 0

1 OF 5

TITLE (4)

Required 10-Year ISI VT-3 Examinations Not Performed on Pressurizer Safety Valves

EVE	NT DATE	(5)		LER NUMBER (6)	REPOR	T DATE	(7)	OTHER FACILIT	TES INV	OLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REV NUMBER	HTHOW	DAY	YEAR	FACILITY NAME		DOCKET NUMBER	
08	19	97	97	15	01	12	11	97	FACILITY NAME		DOCKET NUMBER	
OPER	ATING	NIGHT OF PLANSAGE	THIS RE	PORT IS SUBMITTE	D PURSUAN	T TO THE	REQUIR	EMENTS (OF 10 CFR \$: (Check one			
MO	DDE (9)	MODE 1	20.4	102(b)		20 405(c)			50.73(a)(2)(iv)	73	3.71(b)	
FOI	WER	PROFESS CONTROLLY	20.4	105(a)(1)(i)		50.36(c)(1)			50.73(a)(2)(v)	7:	3.71(c)	
LEVEL (10)		100 20 405(a)(1)(ii)			50.36(c)(2)			5u.73(a)(2)(vii)		OTHER		
	MET (10)	percent										
LEV LEV	PEL (10)	percent	20.4	405/~,(1)(iii)	×	50.73(a)(2)(i)		50.73(a)(2)(viii)(A)			
LEV	AEL (10)	percent	Services and the services	405(*,)(1)(iii) 405(a)(1)(iv)	arventurence and arrest and area.	50 73(a)(2)(50 73(a)(2)(transmit resident metalentalis		50.73(a)(2)(viii)(A) 50.73(a)(2)(viii)(B)	T		

LICENSEE CONTACT FOR THIS LER (12

NAME

Michael J. Angus

Manager Licensing and Corrective Action

TELEPHONE NUMBER (Include Area Code)

316-364-8831 X-4077

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CALTE	SYSTEM	COMPONENT	MANUFACTURE	R	REPORTABLE TO NPRDS
		N/A									
					THE REAL PROPERTY.				1		
NECT CHEST CONTRACTOR CONTRACTOR	ENERGINA ACESSE CINA	SUPPLEMENT	AL REPORT EX	PECTED (14)	uborasses		E	XPECTED	MONTH	DAY	YEAR
YES	3				X	NO		BMISSION ATE (15)			

ABSTRACT (16)

On August 19, 1997, at 1731 hours, WCNOC Engineering personnel determined that a 10-year Inservice Inspection (ISI) requirement for a VT-3 visual examination of the internal surfaces of a Pressurizer Safety Valve had not been performed during the first 10-year interval. As a result, the requirements of Technical Specifications 4.0.5 and 4.4.2.2 were not met and the pressurizer safety valves were declared inoperable. Technical Specification 4.0.3 was entered, allowing 24 hours to complete the required surveillance prior to taking the required actions of Specification 3.4.2.2. The required ISI VT-3 visual examination was completed satisfactorily, and the pre-surizer safety valves were returned to OPERABLE status on August 19, 1997 at 2346 hours. The cause of this event was inadequate program — program interface between the ISI program and Maintenance work planning with respect to work activities which fall outside the scope of the WCNOC ASME Section XI Repair/Replacement program. Until necessary procedure revisions are complete, interim corrective actions ensure ISI engineer review of work activities on Code component which would be outside of the WCNOC Repair/Replacement Program. Revisions of the Pressurizer Code Safety Valve Operability procedure and the work package task planning procedure will be completed by February 3, 1998.

NRC FORM 366

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 5/31/95

ESTIMATED BURDEN PER RESPONSE TO COMPLY
WITH THIS INFORMATION COLLECTION REQUEST:
50.0 HRS. FORWARD COMMENTS REGARDING

LICENSEE	EAEML	REPORT	(LER)
TE	XT CONTI	NOITAUM	

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORD REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1)	DOCKET NUMBER (2)		LER NUMBER (6)		PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
Wolf Creek Generating Station	05000482	97	015	01	2 OF	5

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

Plant Conditions Prior to the Event:

MODE = 1 Reactor Corlant Pressure = 2233 psig Reactor Power = 100 percent

Basis for Reportability:

Technical Specification (TS) Limiting Condition for Operation 3.4.2.2 requires that the pressurizer Code safety valves be OPERABLE during Modes 1, 2, and 3. Surveillance Requirement 4.4.2.2 indicates that there are no required surveillances for the pressurizer safety valves except for those required by TS 4.0.5. Technical Specification 4.0.5 requires that Inservice Inspection of ASME Class 1, 2, and 3 components be performed in accordance with Section XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda, as required by 10 CFR 50.55a. ASME Section XI contains specific requirements for Class 1 pressure retaining components. These requirements pertain to Pressurizer Safety Valves BB-8010A/B/C. ASME Section XI, 1980 Edition through Winter 1981 Addenda, Examination Category B-M-2, Valve Bodies, specifies for valves with a body greater than 4 inches NPS that a VT-3 examination be performed on the valve's internal surface once within each 10-year inspection interval.

WCNOC received approval of Relief Request IIR-40 on October 5, 1995, which allowed for not performing the VT-3 internal examination unless the valves were disassembled. On August 19, 1997, WCNOC personnel determined that past lift testing of the pressurizer safety valves entailed disassembly to accommodate lapping of the valve seat. However, a VI-3 examination was not requested or performed during any lift testing throughout the first 10-year interval.

Not performing the required VT-3 examination within the required time frame constitutes a failure to meet Technical Specification 4.0.5 Surveillance Requirements for the pressurizer safety valves, which, in accordance with the guidance of NUREG-1022, is an operation prohibited by Technical Specifications; therefore, this event is reportable per 10 CFR 50.73(a)(2)(i)(B).

Description of Event:

On August 19, 1997, at 1731 hours, WCNOC Engineering personnel determined that a 10-year Inservice Inspection (ISI) requirement for a VT-3 visual examination of the internal surfaces of one Pressurizer Safety Valve had not been performed during the first 10-year interval. As a result, the pressurizer safety valves were declared inoperable. Technical Specification 4.0.3 was entered, allowing 24 hours to complete the surveillance prior to taking the required actions of Specification 3.4.2.2.

WCNOC has a total of six Pressurizer Safety Valves, three of which are installed in the plant. All three installed valves are charged out each outage. The removed valves are then refurbished for replacement in the subsequent outage. The internal surface inspection

NRC FORM 366 (5-92) LICENSEE EVENT REPORT (LICENSEE EVENT REPORT)	ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORD REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGEY, WASHINGTON, DC 20503.							
FACILITY NAME (1)	DOCKET NUMBER (2)	THE STATE OF THE	LER NUMBER (6	,	P	AGE (3	3)	
THE RESIDENCE OF THE PROPERTY		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER				
Wolf Creek Generating Station	05000482	97	015	01	3	OF	5	

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

requirements for the Pressurizer Safety Valves are contained in ASME Section XI, 1930 Edition through Winter 1981 Addenda, Table IWB-2500-1, Category B-M-2, Val a Bodies. The The Footnotes for that section indicate that examination is limited to one valve within each group of valves that provide the same function and are designed, manufactured, etc., similarly. The WCNOC Pressurizer Safety Valves constitute a single group. Since only one VT-3 visual examination is required on a group of similar valves, satisfactory performance of a VT-3 visual examination on any of the 6 Pressurizer Safety Valves would constitute satisfactory performance of the required surveillance.

Following identification of the missed VT-3 examination August 19, 1997, at 1731 hours, an ISI VT-3 visual examination was completed on one of the 3 Pressurizer Safety Valves that had been removed from the plant during the last refuel outage (Refuel 8). The examination was satisfactory, and the pressurizer safety valves were returned to OPERABLE status on August 19, 1997 at 2346 hours.

Root Cause:

An investigation was conducted to determine the root cause and appropriate corrective actions for this event. The investigation determined that differences exist between the Pressurizer Safety Valves and other valves in the ISI Program which require VT-3 internals examinations. The majority of valves which require VT-3 internals examinations are not of the type that can readily be removed from the system and shipped off-site or to another location for the work.

The investigation also determined that the Pressurizer Safety Valve lift testing activities were not required to be performed under the ASME Section XI Repair/Replacement program. Therefore, the work did not receive the same type of reviews that would be performed on activities which are governed by this program. The Repair/Replacement program requires pre-work review of the repair/replacement plans by the ISI Engineer. This pre-work review has been effective in identifying the need to perform required examinations prior to implementation of the work package.

Based on the results of the investigation, the root cause of this issue was determined to be program to program interface deficiencies between the ISI program and Maintenance work planning with respect to work activities which fall outside of the scope of the WCNOC ASME Section XI Repair/Replacement Program.

During this investigation it was also determined that, in addition to the Pressurizer Safety Valves BB-8010A/B/C, KHR suction gate valves BB-PV8702A/B and EJ-HV8701A/B had not received an internals inspection. However, the RHR suction gate valves were not disassembled during the first 10-year interval. Therefore, Relief Request I1R-40 remains valid for the RHR suction gate valves during the first 10-year interval and no noncompliance existed for these valves.

05000482

97

015

01

OF

TEXT (If more space is required, use additional copies of NRC Form 366A) (17)

Wolf Creek Generating Station

Immediate Corrective Actions:

The required VT-3 visual examination on the internals of a pressurizer safety valve was completed on August 19, 1997 at 2346 hours.

Additional Corrective Actions:

Based on the investigation into this event, no additional deficiencies were identified with respect to the implementation of the first 10-year ISI requirements at WCNOC. Due to the nature of the deficiencies which led to this event, an interim corrective action is being taken with respect to work activities performed on primary system valves and other ASME Class 1 components that fall outside the Repair/Replacement Program. These interim actions ensure ISI engineer review of work activities on Code components which would be outside of the WCNOC Repair/Replacement Program and will remain in place until the procedure revisions are completed.

The 2nd 10-Year inspection interval for WCNOC began September 3, 1995, with the first period in this interval ending May, 1999, concluding with Refuel Outage 10 (this date includes the allowable extension). Prior to Refueling Outage 9, reviews of planned mechanical maintenance activities were performed to ensure that required inservice inspections were properly identified and scheduled for performance.

Procedure STS MT-005, Pressurizer Code Safety Valve Operability, will be revised to direct the planner to notify the ISI engineer for a determination of whether a VT-3 examination is required following valve disassembly, and to provide a copy of the results to the ISI engineer when a required examination is performed. This revision will be completed by February 3, 1998.

Two primary work activities outside of the Repair/Replacement program were identified which could require inservice inspection: disassembly of an ASME Class 1 component, and identification of a damaged support, which requires a surface examination of any corresponding integrally welded attachments. Procedure AP 16C-003, Work Package Task Planning, will be revise; to dire the planner to notify the ISI engineer when these work activities are being planned. This revision will be completed by February 3, 1998.

Safety Significance:

The potential safety significance of this issue is minimal. VT-3 examinations are intended to detect structural degradation of the valve internal surfaces. Structural degradation of the Pressurizer Safety Valves, such as a decrease in wall thickness of the valve body, could result from conditions such as valve leakage. Each outage, the three installed Pressurizer Safety Valves are removed and the alternate valves installed. The removed valves are then lift tested in preparation for re-installation during the next cutage. Recent lift testing of these valves has been performed by a National Board of Boiler and Pressure Vessel Inspectors (National Board) VR/NR qualified vendor at their off-site facility. The vendor performed examinations of the valve internal surfaces during

no indication of valve degradation.

Based on the results of the vendor examinations and the satisfactory performance of the VT-3 examination on August 19, 1997, there is no indication that degradation of the Pressurizer Safety Valves has occurred which would impact their ability to perform their functions.

Other Previous Occurrences:

There have been no similar events at WCNOC involving a failure to recognize 10-year ISI requirements.