

PHILADELPHIA ELECTRIC COMPANY

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March 24, 1986

Docket No. 50-352

Dr. Thomas E. Murley, Regional Administrator
U. S. Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA 19406

SUBJECT: Special Report - High Pressure Coolant Injection
(HPCI) System Actuations and Injections - Limerick
Generating Station - Unit 1

REFERENCE: Technical Specifications 3.5.1.f and 6.9.2

Dear Dr. Murley:

This Special Report is being submitted to meet the requirements of Limerick Generating Station Technical Specifications 3.5.1.f and 6.9.2. Specification 3.5.1.f states, "In the event an ECCS system is actuated and injects water into the reactor coolant system, a Special Report shall be submitted to the Commission pursuant to Specification 6.9.2 within 90 days describing the circumstances of the actuation and the total accumulated actuation cycles to date. The current value of the usage factor for each affected safety injection nozzle shall be provided in this Special Report whenever its value exceeds 0.70". Technical Specification 6.9.2 states, "Special Reports shall be submitted to the Regional Administrator of the Regional Office of the NRC within the time period specified for each report".

This report concerns the occurrence of one High Pressure Coolant Injection (HPCI) system actuation and injection into the reactor coolant system of Unit No. 1. Below is a description of the HPCI system actuation and injection event.

On January 2, 1986 the High Pressure Coolant Injection (HPCI) system actuated and injected to the reactor pressure vessel. The HPCI was actuated manually to control reactor level following a reactor scram caused by a main turbine trip.

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Reactor parameters prior to the transient were as follows:

Reactor Power -	3259	MWt
Reactor Coolant System Pressure -	1003	PSIG
Moderator Temperature -	542	deg. F
Core Flow -	98.9	Mlbm/Hr.
Feedwater Flow -	14.1	Mlbm/Hr.
Feedwater Temperature -	426	deg. F

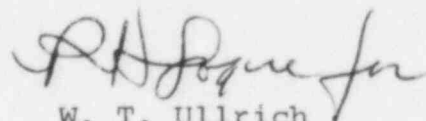
Average HPCI flow was approximately 5,600 gpm which operated continuously for approximately 2 minutes. The injection into the reactor pressure vessel was through the "B" loop Core Spray nozzle only. This constitutes the fourth HPCI actuation and injection to date.

The usage factor for this event for the "B" loop Core Spray nozzle is 9.00N6. Since the effect of this HPCI injection on the "B" loop Core Spray nozzle is negligible, the projected usage factor for this safety injection nozzle, based on the effect of all design transients occurring over the 40 year life of the plant, remains unchanged at 6.07N1. The usage factors for all other safety injection nozzles remain unchanged as a result of this event.

Since HPCI operated as designed to control level, no corrective action is to be taken.

Should you require additional information, please do not hesitate to contact us.

Very truly yours,



W. T. Ullrich
Superintendent
Nuclear Generation Division

cc: E. M. Kelly, Senior Resident Inspector
See Attached Service List