

APR 17 1986

Docket No. 50-289

MEMORANDUM FOR: Harry Kister, Chief  
Projects Branch No. 1

FROM: Allen R. Blough, Chief  
Reactor Projects  
Section 1A

SUBJECT: TMI-1 STATUS REPORT FOR THE PERIOD April 4-11, 1986

Enclosed is the TMI-1 weekly status report from the NRC Resident Office for the subject period. Cold shutdown work activities continued. The NRC staff continued to follow the outage status; including the eddy current testing of steam generator tubes, inspection of the emergency feedwater nozzles; and to conduct random inspection of the test and work activities.

TMI-1 status reports are intended to provide NRC management and the public with highlights from an NRC regulatory perspective of TMI-1 activities for the previous week. Subsequent inspection reports will address many of these topics in more detail.

Original Signed By:

Allen R. Blough, Chief  
Reactor Projects Section 1A  
Branch 1, DRP

Enclosure:  
As stated

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TMI1 WEEKLY STATUS REPORT -

11/29/80 JF31

Harry Kister

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cc w/enclosure:

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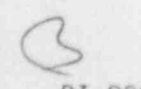
R. Conte, RI (20 cys)

W. Baunack, RI


Region I Docket Room (w/concurrences)

 RI:DRP  
Conte

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TMI1 WEEKLY STATUS REPORT -

11/29/80

## ENCLOSURE

### TMI-1 STATUS REPORT FOR THE PERIOD April 4 - 11, 1986

#### 1. Plant Status

As of 8:00 a.m. on April 11, 1986, TMI-1 was in a cold shutdown condition with reactor vessel water level maintained at the reactor vessel nozzles and the primary side of the steam generators drained for eddy current testing (ECT). The decay heat system was maintaining reactor vessel water temperature at approximately 80-100 F.

#### 2. Facility Operations Summary

Cold shutdown activities continued throughout the period. In addition to eddy current testing, those activities included primarily preventive maintenance, corrective maintenance, and modification work to make progress on those items that must be completed before the restartup after the Cycle 6 refueling outage, which is scheduled to begin in December 1986. Testing continued on safety related motor-operated valves and containment isolation valves.

#### 3. Special Interest Items

##### Eddy Current Testing of Steam Generator Tubes

The licensee made progress during the week in obtaining eddy current data and in conducting analysis of that data. Licensee and contractor personnel collected and computerized data on site and analyzed that data at a location off site. The initial tube sample size was approximately 1218 tubes for the "B" OTSG and 1556 tubes for the "A" OTSG. For both steam generators, the licensee completed collection of data. Thus far, preliminary review of the data indicates defects greater than 40 percent through-wall have been identified in about 30 tubes. Based on current technical specifications and pending final licensee analysis, these tubes will need to be plugged prior to reactor startup.

##### Feedwater Nozzle Inspection

The licensee commenced an inspection of the emergency feedwater spray ring nozzles due to a weld failure of a similar nozzle at Ocone 1. Initial investigation of two nozzles in the 'A' steam generator revealed substantial weld cracks in the welds connecting the nozzle thermal sleeve to the retaining collar. The licensee is removing all twelve nozzles on both steam generators for NDE examination and weld repair, if applicable.

#### 4. NRC Staff Status During the Period

In accordance with previously announced plans, two additional resident inspectors have been officially assigned to full-time duty at TMI-1. One has reported on-site; the other is currently attending an NRC technical

training course off-site. During this report period, the NRC staff on site consisted of the senior resident and two resident inspectors, supplemented by a technical specialist from Region I.

The resident staff continued to follow plant status and outage work and conducted random inspections of these activities. The technical specialist was on site to review eddy current test data and to follow up on the emergency feedwater ring nozzle problem.

NRC inspection report 86-01, detailing inspection activities during January 10 - February 7, was issued April 2, 1986. The inspection identified one violation of regulatory requirements -- failure of fire brigade members to wear required equipment when responding to fire drills.

5. Systematic Assessment of Licensee Performance (SALP)

At 7:30 p.m. on Thursday, April 24, 1986, members of NRC Region I management will be available at the NRC's Middletown office (Downtown Mall, 100 Brown Street, Middletown, Pennsylvania) to answer questions from the public regarding (1) the recently completed NRC Region I Systematic Assessment of Licensee Performance (SALP) at TMI-1, and (2) NRC inspection programs at TMI-1. Because of space limitations at the Middletown Office, it is important for those planning to attend the public meeting to contact Allen R. Blough at (215) 337-5146 or (717) 948-1160. If the number of respondents is large, an alternate location in the Middletown-Harrisburg area will be arranged. Public response to this planned meeting has been minimal thusfar.

6. NRC (TMI-1) Staff Composition During Period

The NRC staff at TMI-1 was comprised of the following personnel during the period:

- R. Conte, Senior Resident Inspector
- D. Johnson, Resident Inspector
- R. McBrearty, Reactor Engineer
- F. Young, Resident Inspector
- C. Hix, Secretary