



Northern States Power Company

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July 15, 1986

Director
Office of Nuclear Reactor Regulation
U S Nuclear Regulatory Commission
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT
Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

License Amendment Request Dated July 15, 1986
Heatup/Cooldown Curves and Miscellaneous Changes

Attached are three originals and 37 conformed copies of a request for a change to the Technical Specifications, Appendix A, of Operating Licenses DPR-42 and DPR-60. This request is submitted in accordance with the provisions of 10 CFR Part 50, Section 50.90. A check is attached in the amount of \$150.00 in accordance with the requirements of 10 CFR Part 170 for the amendment application fee.

The existing Prairie Island reactor coolant system heatup and cooldown curves are only valid for ten effective full power years (EFPY) of plant operation. Prairie Island Unit 1 (lead unit) is expected to reach 10 EFPY by early October, 1986. This license amendment request includes proposed heatup and cooldown curves valid to 15 EFPY and other administrative changes to the Heatup and Cooldown Technical Specifications and associated bases.

This license amendment request also proposes changes in the following areas: 1) Radiation Environmental Monitoring Program; 2) Design Features, Section 5; 3) Security Plan Implementing Procedures Review; and 4) Administrative Controls, Section 6.

Exhibit A describes the proposed changes, reason for the changes and significant hazards considerations evaluation. Exhibit B contains the proposed changes marked up on the existing Technical Specification pages. Exhibit C contains the revised Technical Specification pages.

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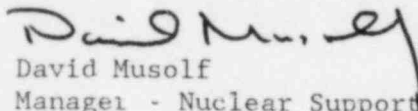
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The Pressurized Thermal Shock (PTS) Rule, 10 CFR Part 50, Section 50.61, requires that the projected assessment of the RT_{PTS} must be updated whenever changes in core loadings, surveillance measurements or other information indicate a significant change in the projected values. In that regard, the NRC Pressurized Thermal Shock Safety Evaluation Reports for Prairie Island Units 1 and 2, dated June 23, 1986, requested that a re-evaluation of the RT_{PTS} and comparison of the predicted value be submitted with any future pressure-temperature submittals which are required by 10 CFR 50, Appendix G. This re-assessment of the RT_{PTS} is attached as Exhibit E.

Please contact us if you have any questions concerning this License Amendment Request or if additional information is required to supplement it. Because the existing heatup and cooldown curves are only valid until early October, NRC Staff review and approval of these changes is requested prior to October 1, 1986.


David Musolf

Manager - Nuclear Support Services

DMM/EFE/efe

c: Regional Administrator-III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
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Attachments

Exhibit A Evaluation of Proposed Changes to the Technical Specifications
Exhibit B Proposed Changes Marked Up on Existing Technical Specification Pages
Exhibit C Revised Technical Specification Pages
Exhibit D Draft Revised Page From Offsite Dose Calculation Manual
Exhibit E Summary of RT_{PTS} Re-evaluation