

OPERATING DATA REPORT

DOCKET NO. 50-321
 DATE 02-10-86
 COMPLETED BY: Lee Kanipe
 TELEPHONE (912) 367-7851 x 2882

OPERATING STATUS

1. Unit Name: E. I. Hatch Nuclear Plant
2. Reporting Period:
3. Licensed Thermal Power (MWt):
4. Nameplate Rating (Gross MWe):
5. Design Electrical Rating (Net MWe):
6. Maximum Dependable Capacity (Gross MWe):
7. Maximum Dependable Capacity (Net MWe):
8. If changes occur in capacity ratings
 (Items Number 3 through 7) since last report, give reasons:
 Item 7 changed from 752.2 to 768.0 because of Peak Period Continuous Rating Data. The change was based on actual performance during the previous year.
9. Power level to which restricted, if any (Net MWe)
10. Reasons for restrictions, if any:

Unit 1
 01-86
 2436
 809.3
 777.3
 801.2
 768.0

No Restrictions
 N/A

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	744	88415
12. Number of Hours Reactor was Critical	0.0	0.0	62126.2
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line	0.0	0.0	58561.1
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	0	0.0	124631490
17. Gross Electrical Energy Generated (MWH)	0	0.0	40242130
18. Net Electrical Energy Generated (MWH)	0	0	38210136
19. Unit Service Factor	0.0	0.0	66.2
20. Unit Availability Factor	0.0	0.0	66.2
21. Unit Capacity Factor (Using MDC Net)	0.0	0.0	56.3
22. Unit Capacity Factor (Using DER Net)	0.0	0.0	55.6
23. Unit Forced Outage Rate	0.0	0.0	17.5
24. Shutdowns scheduled over next 6 months (Type, Date, and Duration of Each): Refueling Outage began November 27, 1985	14 weeks		
25. If shutdown at end of report period, estimated date of startup:	April 1, 1986		
26. Units in Test Status (Prior to Commercial Operation):	N/A		

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 PDR ADOCK 05000321
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AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-321
 DATE: 02-10-86
 COMPLETED BY: Lee Kanipe
 TELEPHONE (912) 367-7851 x 2882

MONTH 01-86

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
-----	--

1	-5
2	-5
3	-5
4	-5
5	-6
6	-6
7	-6
8	-6
9	-5
10	-5
11	-5
12	-6
13	-6
14	-6
15	-6
16	-6

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
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17	-6
18	-5
19	-5
20	-6
21	-6
22	-6
23	-6
24	-5
25	-5
26	-5
27	-5
28	-5
29	-5
30	-4
31	-4

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-321 UNIT NAME HATCH ONE
 COMPLETED BY Lee M. Kanipe Jr. TELEPHONE (912) 367-7851 (2882) DATE FEBRUARY 10, 1986
 REPORT MONTH JANUARY

NO.	DATE	T Y P E	DURATION (HOURS)	R E A S O N	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NUMBER	S Y S T E M	COMPONENT CODE	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
86-001	860101	S	744.0	C	4	N/A	RC	FUELXX	Refueling Outage Continues

F: Forced
 S: Scheduled

Reason:
 A-Equipment Failure (Explain)
 B-Maintenance or Test
 C-Refueling
 D-Regulator Restrictor
 E-Operator Training & License
 Examination
 F-Administrative
 G-Operational Error (Explain)
 H-Other (Explain)

Method:
 1-Manual
 2-Manual Scram
 3-Automatic Scram
 4-Continuations
 5-Load Reduction
 9-Other (Explain)

Events reported involve
 a greater than 20% reduction
 in average daily power level
 for the preceeding 24 hours.

NARRATIVE REPORT
UNIT 2

January 1st	0000	Unit at 705 GMWE and 2136 CMWT. Ramping toward rated Maximum Dependable Capacity (MDC) following scram on December 25, 1985. No major equipment problems.
	0600	Reducing power to approximately 400 GMWE to clean circ water screens.
	1110	Ramping toward rated MDC.
January 2nd	1000	Back at rated MDC.
January 5th	0000	Reducing power for weekly turbine testing.
	0306	Testing complete. Ramping back to rated MDC.
	0627	Back at rated MDC
January 8th	1805	Reducing power to 85%. Management decision to reduce power to help relieve offgas activity in reactor and turbine buildings.
January 18th	1615	Reducing load to take 2A Recirc Pump out of service due to Pump Motor Cooler Housing Leak alarm.
January 19th	0058	Turbine Generator off line.
	0533	Rx. Scram on group one isolation.
January 22nd	0110	Reactor startup in progress. Control rod withdrawal commencing.
	0220	Reactor critical.
January 23rd	2008	Turbine generator on line. Ramping toward 85% power.
January 24th	0430	Scram Time Testing in progress.
	0900	Scram Time Testing complete. Ramping toward 85% power.
January 26th	0800	Back at 85% power.
January 31st	2400	Unit on line at 679 GMWE and 2068 CMWT (84.9% power).

HATCH 2 SAFETY-RELATED MAINTENANCE WORK ORDERS
TO BE REPORTED FOR JANUARY 1986

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
85-5059	01-08-86	Installed new static-O-ring pressure switches for Drywell Pneumatic system (2P70-N003) per WPS-85-180-J002, M002, E002, and E003. Ref. DCR 85-180.
85-4186	01-14-86	Replaced LED's and added jumpers on trip units in control room, Group 100, 400, 500, and 600 of 2A71 (ATTS) system. Ref. DCR 81-139.
84-6393	01-20-86	Installed Non-Class 1E analog cables between panels X75-P601 and 2H11-P645 per WPS-80-157-EC17. Ref. DCR 80-157 Rev 1.
85-5056	01-25-86	Reworked spared cables, tagged unidentified cable, and installed correct tags per WPS-80-157-EM78. Ref. Deficiency Report 2-85-446 and DCR 80-157 Rev. 1.
84-6143	01-25-86	Provide a separate power supply for 2Y33-PC08 per WPS-80-157-EC16 and RC02. Ref. DCR 80-157 Rev. 1.

OPERATING DATA REPORT

DOCKET NO. 50-366
 DATE 02-10-86
 COMPLETED BY: Lee Kanipe
 TELEPHONE (912) 367-7851 x 2882

OPERATING STATUS

- | | |
|--|-----------------|
| 1. Unit Name: E. I. Hatch Nuclear Plant | Unit 2 |
| 2. Reporting Period: | 01-86 |
| 3. Licensed Thermal Power (MWt): | 2436 |
| 4. Nameplate Rating (Gross MWe): | 817.0 |
| 5. Design Electrical Rating (Net MWe): | 784.0 |
| 6. Maximum Dependable Capacity (Gross MWe): | 803.9 |
| 7. Maximum Dependable Capacity (Net MWe): | 777.0 |
| 8. If changes occur in capacity ratings
(Items Number 3 through 7) since last report, give reasons:
Item 7 changed from 747.9 to 777.0 because of Peak Period Continuous Rating Data. The change was based on actual performance during the previous year. | |
| 9. Power level to which restricted, if any (Net MWe): | No Restrictions |
| 10. Reasons for restrictions, if any: | N/A |

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	744	744	56041
12. Number of Hours Reactor was Critical	675.2	675.2	38246.8
13. Reactor Reserve Shutdown Hours	0.0	0.0	0.0
14. Hours Generator On-Line;	628.8	628.8	36497.0
15. Unit Reserve Shutdown Hours	0.0	0.0	0.0
16. Gross Thermal Energy Generated (MWH)	1313008	1313008	79496214
17. Gross Electrical Energy Generated (MWH)	435870	435870	26217370
18. Net Electrical Energy Generated (MWH)	415930	415930	24954584
19. Unit Service Factor	84.5	84.5	65.1
20. Unit Availability Factor	84.5	84.5	65.1
21. Unit Capacity Factor (Using MDC Net)	71.9	71.9	57.3
22. Unit Capacity Factor (Using DER Net)	71.3	71.3	56.8
23. Unit Forced Outage Rate	15.5	15.5	11.0
24. Shutdowns scheduled over next 6 months (Type, Date, and Duration of Each):	N/A		

25. If shutdown at end of report period, estimated date of start-up:
 26. Units in Test Status (Prior to Commercial Operation): N/A

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-366
 DATE: 02-10-86
 COMPLETED BY: Lee Kanipe
 TELEPHONE (912) 367-7851 x 2882

MONTH 01-86

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

1	637
2	772
3	774
4	779
5	772
6	782
7	781
8	766
9	671
10	665
11	662
12	662
13	659
14	659
15	657
16	654

DAY AVERAGE DAILY POWER LEVEL
 (MWe-Net)

17	653
18	515
19	-13
20	-14
21	-13
22	-12
23	6
24	306
25	618
26	654
27	660
28	655
29	652
30	655
31	656

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-366

UNIT NAME HATCH TWO

COMPLETED BY Lee M. Kanipe

TELEPHONE (912) 367-7851 (2882) DATE FEBRUARY 10, 1986

REPORT MONTH JANUARY

NO.	DATE	T Y P E	DURATION (HOURS)	R E S O N	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NUMBER	S Y S T E M	COMPONENT CODE	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
86-001	860101	S	6.0	B	5	N/A	HA	TURBIN	Power ascension follow- ing Scram on 12-25-85.
86-002	860101	S	11.0	B	5	N/A	HF	FILTER	Reducing Power to clean Circ Water Screens.
86-003	860101	S	17.0	B	5	N/A	HA	TURBIN	Scram Recovery Resumes.
86-004	860105	S	6.5	B	5	N/A	HA	TURBIN	Weekly Turbine Testing.
86-005	860108	S	238.7	H	5	N/A	ZZ	ZZZZZZ	Management decision to reduce power to 85%. High offgas activity in Rx. and Turbine Building caused by leaking fuel was slow- ing down Unit 1 Outage work.
86-006	860118	F	8.0	A	5	N/A	CB	HTEXCH	Reducing Load due to "A" Recirc Pump Motor Cooler Alarm.

F: Forced
S: Scheduled

Reason:

A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License
Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

Method:

1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other (Explain)

Events reported involve
a greater than 20% reduction
in average daily power level
for the preceeding 24 hours.

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-366

UNIT NAME HATCH TWO

COMPLETED BY Lee M. Kanipe

TELEPHONE (912) 367-7851 (2882) DATE FEBRUARY 10, 1986

REPORT MONTH JANUARY

NO.	DATE	T Y P E	DURATION (HOURS)	R E A S O N	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NUMBER	S Y S T E M	COMPONENT CODE	CAUSE & CORRECTIVE ACTION TO PREVENT RECURRENCE
86-007	860119	F	115.2	A	3	2/86/3	CB	HTEXCH	Rx. Scram on Group I Isolation. Going to cold shutdown for repair of "A" Recirc Pump Motor Cooler.
86-008	860123	S	59.9	B	5	N/A	CB	HTEXCH	Scram Recovery in progress.
86-009	860126	S	136.0	A	5	N/A	ZZ	ZZZZZZ	Management decision to reduce power to 85%. High offgas activity in Rx. and Turbine Building caused by leaking fuel was slowing down Unit 1 Outage work.

F: Forced

S: Scheduled

Reason:

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

Method:

1-Manual

2-Manual Scram

3-Automatic Scram

4-Continuations

5-Load Reduction

9-Other (Explain)

Events reported involve

a greater than 20% reduction in average daily power level for the preceeding 24 hours.

Georgia Power Company
Post Office Box 439
Baxley, Georgia 31513
Telephone 912 367-7781
912 537-9444



Edwin I. Hatch Nuclear Plant

February 10, 1986

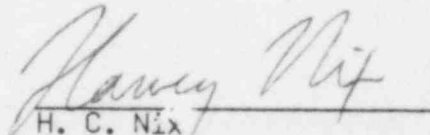
LR-MGR-032-0286

PLANT E. I. HATCH
NRC Monthly Operating Report

Director
Office of Management and Program Analysis
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sir:

Per Tech Specs section 6.9.1.6 please find attached the NRC Monthly Operating Report for Hatch Unit 1, Docket #50-321, and for Hatch Unit 2, Docket #50-366.


H. C. Nix
General Manager

LMK/kel

xc: J. P. O'Reilly
L. T. Gucwa
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NARRATIVE REPORT
UNIT 1

January 1st	0000	Unit in cold shutdown for Refueling Outage.
January 31st	2400	Unit in cold shutdown for Refueling Outage. Expected startup April 01, 1986.

HATCH 1 SAFETY-RELATED MAINTENANCE WORK ORDERS
TO BE REPORTED FOR JANUARY 1986

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
85-7500	01-12-86	Performed retest of relays 1E11A-K54A in panel 1H11-P617 and A71B-K32 in panel 1H11-P623 in accordance with procedure 42SP-DCI-013-1S. Ref. DCR 82-171.
85-6555	01-14-86	Upgraded HFA relays in 1B21C system in panel 1H11-P628 to G. E. Century series relays per WPS-82-171-E028. Ref. DCR 82-171.
85-6552	01-14-86	Upgraded HFA relays in 1A71B system in panels 1H11-P609 to G. E. Century series relays per WPS-82-171-E021. Ref. DCR 82-171.
85-6554	01-14-86	Upgraded HFA relays in 1A71B and 1C61 systems in panel 1H11-P622 to G. E. Century series relays per WPS-82-171-E026. Ref. DCR 82-171.
85-7944	01-12-86	Upgraded HFA relays in the 1R20-B27C system in panel 1H11-P652 to Century series relays per WPS-82-171-E036. Ref. DCR 82-171.
85-5275	01-16-86	Removed cylinder relief valves from Diesel Generator 1A (1R43-S001A) and replaced with plugs per WPS-83-123-M003. Ref. DCR 83-123.

HATCH 1 SAFETY-RELATED MAINTENANCE WORK ORDERS
TO BE REPORTED FOR JANUARY 1986

<u>NUMBER</u>	<u>DATE COMPLETED</u>	<u>DESCRIPTION</u>
85-7485	01-13-86	MWO voided - Work done under MWO 1-85-6850 Sealing TSC Area. Ref. DCR 80-157 Rev. 1.
85-6549	01-19-86	Upgraded HFA relays in the 1E11A system in panel 1H11-P618 to G. E. Century series relays per WPS-82-171-E024. Ref. DCR 82-171.
85-1391	01-30-86	Installed internal jumper between panels X75-P101 and X75-P102 to provide power for X75-P102 lighting, circuits, and receptacles per WPS-80-157-EM79. Ref. DCR 80-157 Rev.1.
85-6550	01-31-86	Upgraded HFA relays in the 1E41A system in panel 1H11-P620 to G. E. Century series relays per WPS-82-171-E025. Ref. DCR 82-171.
86-0156	01-31-86	Replaced 250 volt DC breaker with refurbished breaker equipped with modified trip element. Ref. DCR 85-221 Appendix "R"
85-3238	01-31-86	Changed inscription on annunciator window in panels X75-P101 and P102, and perform wiring in panels X75-P606 and P608 per WPS-80-157-EM67 and EM68. Ref. DCR 80-157 Rev. 1.
85-083	01-31-86	Installed raceways in control building for SPDS (X75) per WPS-80-157-RC05, RC06, and RC07. Ref. DCR 80-157 Rev. 1.
86-0157	01-31-86	Replaced 250 volt DC breaker with refurbished breaker equipped with modified trip elements per WPS-85-221-E002. Ref. DCR 85-221.