APPENDIX D UNIT SHUTDOWNS AND POWER REDUCTIONS

Rev. 1

DOCKET NO. 50-265

.

UNIT NAME TWO

November 7, 1997 REPORT MONTH October 1997

COMPLETED BY <u>Fristal Sirles</u>

DATE

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NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING NOWN REACTOR	LICENSEE EVENT REPORT	SYSTEM CODE	COMPONENT	CORRECTIVE ACTIONS/COMMENTS
97-12	971001	F	72.0	Н	4				Continuation of Safe Shutdown Issues.
97-13	971004	S	552.0	A	2				Commenced Q2P01 to Replaced Leaking Fuel Bundle.
97-14	971027	F	121.0	н	4				Continuation of Safe Shutdown Issues.
			and a second						

Last month 97-13 was reported as 522.0 hours. The correct number is 552.0 hours.

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram timing for the reporting period. All scram timing was performed with reactor pressure greater than 800 PSIG.

RESULTS CF SCRAM TIMING MEASUREMENTS PERFORMED ON UNIT <u>1 & 2</u> CONTROL ROD DRIVES, FROM <u>01/01/97</u> TO <u>11/30/97</u>

		AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN			AT % HDRAWN	MAX. TIME FOR 90% INSERTION	DESCRIPTION		
DATE	NUMBER OF RODS	5 0.375	20	57 2.00	90 3.5	1 7 sec.	Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)		
2/17/97	14	0.314	0.712	1.528	2.671	3.010 (J-14)	STT for Viton Issue U-1 (Core Wide Ave. Times)		
3/29-31/97	21	0.316	0.708	1.513	2.645	3.010 (J-14)	STT for Tech Spec (19) (Core Wide Ave. Times) PMTV (2)		
4/25/97	1	0.29	0.64	1.37	2.4	3.010 (J-14)	Post Maintenance Test for Accumulator Replacmen Unit 1		
5/31/97	14	0.354	0.725	1.497	2.576	2.77 (G-15)	STT for Viton Issue - Ul		
6/23/97	13	0.297	0.681	1.478	2.615	2.95 (M-12)	Post Outage Scram Test Timing and PMTV - U2		
6/29/97	4	0.278	0.630	1.348	2.350	2.46 (P-6)	Post Outage Scram Testing and PMTV - U2		
7/16/97	1	0.280	0.630	1.324	2.306	2.306 (N-6)	PMTV - U2		
7/20/97	32	0.312	0.701	1.497	2.617	2.67 (G-15)	TS & SSPV Testing U-1		
9/18/97	14	0.312	0.702	1.499	2.621	2.77 (G-14)	STT for Viton Issue U-1		
11/01/97	1	0.29	0.62	1.13	2.3	2.850 (D-11)	PMTV U-1		
11/13/97	34	0.351	0.731	1.501	2.593	2.3 (P-5)	TS & SSPV Testing U-1		

VII. REFUELING INFORMATION

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The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

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QUAD CITIES REFUELING INFORMATION REQUEST

1.	Unit:01	Reload:14	Cycle:15
2.	Scheduled date for next	refueling shutdown:	9/05/98
3.	Scheduled date for rest	tart following refueling:	10/15/98

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

Yes

9.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

November, 1997

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

Approx. 216 SPC 9X9IX Fuel Bundlos Q1R15 will be loaded.

7. The number of fuel assemblies.

the present licensed capacity:

a.	Number of	assemblies in	core:	724

b. Number of assemblies in spent fuel pool:

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

a.	License	storage capacity	y for spent fuel:	3557
b.	Planned	increase in licer	nsed storage:	0
The d	projected	date of the last to the spent fuel	refueling that can 1 pool assuming	

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QUAD CITIES REFUELING INFORMATION REQUEST

1.	Unit:Q2	Reload:14	Cycle:15
2.	Scheduled date for new	t refueliny shutdown:	9/25/99
3.	Scheduled date for rea	start following refueling:	10/31/99

Will refueling or resumption of operation thereafter require a Technical 4. Specification change or other license amendment:

Yes

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5. Scheduled date(s) for submitting proposed licensing action and supporting information:

August, 1998

Important licensing considerations associated with refueling, e.g., new 6. or different fuel design or supplier, unreviewed design or performance analysis methods, significant clunges in fuel design, new operating procedures:

N/A

The number - fuel assemblies. 7. 724 Number of assemblies in core: 6. b. Number of assemblies in spent fuel pool: 2943

The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is 8. planned in number of fuel assemblies:

a.	Licensed	storage ca	apacity	for spe	nt fuel	:	3897	
b.	Planned i	ncrease in	licer	sed stor	age:		0	
The p	rojected d	late of the	last fuel	refuelin	g that	can		

9. the present licensed capacity:

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM - Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring ANSI - American National Standards Institute APRM - Average Power Range Monitor ATWS - Anticipated Transient Without Scram BWR - Boiling Water Reactor CRD - Control Rod Drive EHC - Electro-Hydraulic Control System EOF - Emergency Operations Facility GSEP - Generating Stations Emergency Plan. HEPA - High-Efficiency Particulate Filter HPCI - High Pressure Coolant Injection System HRSS - High Radiation Sampling System IPCLRT - Integrated Primary Containment Leak Rate Test IRM - Intermediate Range Monitor ISI - Inservice Inspection LER - Licensee Event Report LLRT - Local Leak Rate Test LPCI - Low Pressure Coolant Injection Mode of RHRs LPRM - Local Power Range Monitor MAPLHGR - Maximum Average Planar Linear Heat Generation Rate MCPR - Minimum Critical Power Ratio MFLCPR - Maximum Fraction Limiting Critical Power Ratio - Maximum Permissible Concentration MPC MSIV - Main Steam Isolation Valve NIOSH - National Institute for Occupational Safety and Health - Primary Containment Isolation PCI PCIOMR - Proconditioning Interim Operating Management Recorgendations RBCCW - Reactor Building Closed Cooling Water System RBM - Rod Block Monitor RCIC - Reactor Core Isolation Cooling System RHRS - Residual Heat Removal System RPS - Reactor Protection Sys em RWM - Rod Worth Minimizer SBGTS - Standby Gas Treatment System SBL - Standby Liquid Control SDC - Shutdown Cooling Mode of RHRS SDV - Scram Discharge Volume SRM - Source Range Monitor TBCCW - Turbine Building Closed Cooling Water System TIP - Traversing Incore Probe TSC - Technical Support Center