QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

NOVEMBER 1997

COMMONWEALTH EDISON COMPANY

AND

MIDAMERICAN ENERGY COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

TABLE OF CONTENTS

- I. Introduction
- II. Summary of Operating Experience
 - A. Unit One
 - B. Unit Two
- III. Plant or Procedure Changes, Tests, Experiments, and Safety Related Maintenance
 - A. Amendments to Facility License or Technical Specifications
 - B. Facility or Procedure Changes Requiring NRC Approval
 - C. Tests and Experiments Requiring NRC Approval
 - IV. Licensee Event Reports
 - V. Data Tabulations
 - A. Operating Data Report
 - B. Average Daily Unit Power Level
 - C. Unit Shutdowns and Power Reductions
 - VI. Unique Reporting Requirements
 - A. Main Steam Relief Valve Operations
 - B. Control Rod Drive Scram Timing Data
- VII. Refueling Information

VIII. Glossary

I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors and Steam Turbine/Generators, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and MidAmerican Energy Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Kristal Sirles and Debra Kelley, telephone number 309-654-2241, extensions 3070 and 2240, respectively.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Quad Cities Unit One was on line the entire month of November 1997. On November 8, 1997 at 2000 hours, a load drop was initiated in preparation for Drywell entry to troubleshoot an increase in Drywell leakage. The inspection indicated that the Core Spray Vent Valve 1-1402-66A had a packing leak. On November 11, 1997 at 2035 hours, repairs were completed and Unit One commenced to increase load. A few other load drops were performed, however the average daily power level remained at 80% or greater.

B. Unit Two

Quad Cities Unit Two started the month of November 1997 in a forced shutdown to address Safe Shutdown issues. On November 10, 1997 at 0000 hours, an extension to Q2P01 planned outage began.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

Technical Specification Amendment No. 178 was issued on October 7, 1997 to Facility Operating License DPR-29 and Amendment No. 176 to Facility Operating License DPR-30 for Quad Cities Nuclear Power Station.

The amendments clarify the load value for the emergency diesel generator to be equal to or greater than the largest single load and revise the frequency and voltage requirements during the performance of the test.

- B. Facility or Procedure Changes Requiring NRC Approval

 There were no Facility or Procedure changes requiring NRC approval for the reporting period.
- C. <u>Tests and Experiments Requiring NRC Approval</u>

 There were no Tests or Experiments requiring NRC approval for the reporting period.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two submitted during the reporting period.

UNIT 1

Licensee Event Report Number	Submission Date	Title of occurrence		
97-022	11/12/97	The Control Room Heating, Ventilation and Air Conditioning System Was Declared INOP Due To Inadequate Cooling Water Flow Which Was Caused By A System Perturbation.		
97-023	11/26/97	A Required Technical Specification (TS) Surveillance Was Not Performed Prior To Reactor Mode Change On Four Occasions Due To Inadequate Procedures Associated With Implementation Of The TS Upgrade program.		

UNIT 2

Licensee Event Report Number	Submission Date	Title of occurrence
97-002 Rev. 1	11/18/97	Unit 2 Was Shutdown, Per The Requirements of Technical Specifications 3.5.A and 3.6.F, Because Four Main Steam Relief Valve Closure Times Did Not Meet Inservice Testing Program Limits. The timing methodology Had Changed; However, The Acceptance Criteria Had Not Been Reevaluated. In Addition, The Unit 2 Shutdown Was Required Because Of A Loss Of Primary Containment Integrity Due To Misinterpretation Of Technical Specifications Resulting In Al. Inadequate Procedure.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

- A. Average Daily Unit Power Level
- B. Operating Data Report
- C. Unit Shutdowns and Power Reductions

APPENDIX B AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO	50-254
UNIT	One
DATE	December 5, 1997
COMPLETED BY	Kristal Sirles
TELEPHONE	(309) 654-2241

MONTH	November 1997				
DAY AVE	RAGE DAILY POWER LEVEL (NWe-Net)	DAY AVERAGE DAILY POWER (MWe-Net)			
1	760	17764			
2	779	18779			
3	778	19			
4	780	20			
5	779	21768			
6	778	22742			
7	778	23778			
8.	739	24			
9	80	25			
10	58	26776			
11	65	27778			
12	515	28776			
13	588	29773			
14	776	30775			
15	757	31			
16.	756				

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B AVERAGE DAILY UNIT POWER LEVEL

MONTH November 1997	DOCKET NO 50-265 UNIT TWO DATE December 5, 1997 COMPLETED BY Kristal Sirles TELEPHONE (309) 654-2241
DAY AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY AVERAGE DAILY POWER LEVEL (MWe-Net)
18	178
28	18
38	19
48	20
58	21
69	22
78	23
88	24
98	25
108	26
11 9	27
128	288
13	298
14	308
15	31.
168	

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT One

DATE December 5,

1997

COMPLETED BY Kristal Sirles

TELEPHONE (309) 654-2241

OPERATING STATUS

0000 110197

- 1. REPORTING PERIOD: 2400 113097 GROSS HOURS IN REPORTING PERIOD: 720
- CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769
 DESIGN ELECTRICAL RATING (MWc-NET): 789
- 3. POWER LEVEL TO WHICH KESTRICTED (IF ANY) (MWe-Net): N/A
- 4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	720.00	7397.10	171906.30
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	3421.90
7. HOURS GENERATOR ON LINE	720.00	7301.20	166830.10
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	909.20
9. GROSS THERMAL ENERGY GENERATED (MWH)	1604413.20	17115424.00	364288306.50
10. GROSS ELECTRICAL ENDINGY GENERATED (MWH)	517715.00	5462120.00	117883550.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	494495.00	5213947.00	111368916.00
12. REACTOR SERVICE FACTOR	100.00	92.28	76.49
13. REACTOR AVAILABILITY FACTOR	100.00	92.28	78.02
14. UNIT SERVICE FACTOR	100.00	91.08	74.23
15. UNIT AVAILABILITY FACTOR	100.00	91.08	74.64
16. UNIT CAPACITY FACTOR (Using MDC)	89.31	84.58	64.44
17 UNIT CAPACITY FACTOR (Using Design MWe)	87.05	82.44	62.81
18. UNIT FORCED OUTAGE RATE	0.00	8.92	7.55

- SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): Maintenance Q1P01 1/4/98 - 1/24/98
- 20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP: N/A
- 21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): N/A

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-2

UNIT Two

DATE December 5,

1997

COMPLETED BY Kristal Sirles

TELEPHONE (309) 654-2241

OPERATING STATUS

0000 110197

- 1. REPORTING PERIOD: 2400 113097 GROSS HOURS IN REPORTING PERIOD: 720
- CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769
 DESIGN ELECTRICAL RATING (MWe-NET): 789
- 3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
- 4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE	
5. NUMBER OF HOURS REACTOR WAS CRITICAL	0.00	3780.50	164357.05	
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	2985.80	
7. HOURS GENERATOR ON LINE	0.00	3720.50	159969.85	
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	702.90	
GROSS THERMAL ENERGY GENERATED (MWH)	0.00	8684167.50	348356256.32	
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	0.00	2745407.00	111762544.00	
11. NET ELECTRICAL ENERGY GENERATED (MWH)	0.00	2627737.00	105947458.00	
12. REACTOR SERVICE FACTOR	0.00	47.16	73.65	
13. REACTOR AVAILABILITY FACTOR	0.00	47.16	74.99	
14. UNIT SERVICE FACTOR	0.00	46.41	71.68	
15. UNIT AVAILABIT FACTOR	0.00	46.41	72.00	
16. UNIT CAPACITY FACTOR (Using MDC)	0.00	42.63	61.73	
17. UNIT CAPACITY FACTOR (Using Design MWe)	0.00	41.55	60.17	
18. UNIT FORCED OUTAGE RATE	0.30	11.90	11.19	

- 19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): N/A
- 20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP: 12/19/97
- 21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): N/A

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX D UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254 COMPLETED BY Kristal Sirles UNIT NAME One December 5, 1997 REPORT MONTH November 1997 TELEPHONE 309-654-2241 DATE METHOD OF SHUTTING DOWN R. ACTOR COMPONENT SYSTEM REASON TYPE S [2, LICENSEE DURATION EVENT CORRECTIVE ACTIONS/COMMENTS (HOURS) DATE REPORT NO. Performed Load Drop to Troubleshoot 97-16 F 0 5 971108 A Increased Drywell Leakage. Found Core Spray Vent Valve had a Packing Leak.

APPENDIX D UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Two COMPLETED BY Kristal Sirles

DATE December 5, 1997 REPORT MONTH November 1997 17LEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT	SYSTEM	COMPONENT	CORRECTIVE ACTIONS/COMMENTS
97-15	971101	F	216.0	Н	4	*****			Continuation of Safe Shutdown Issues.
97-16	971108	S	504.0	Н	4				Continuation of Planned Outage.