LICENSEE:

Duke Energy Corporation

FACILITIES:

McGuire Nuclear Station, Units 1 and 2

SUBJECT:

MEETING SUMMARY - MEETING OF JANUARY 27, 1999, REGARDING

SOLUBLE BORON AND BORAFLEX

On January 27, 1999, the staff met with Duke Energy Corporation (DEC) personnel to discuss information related to a proposed license amendment to take credit for soluble boron and partial credit for boraflex in the spent fuel pools at the McGuire Nuclear Station. The meeting was scheduled at the request of DEC to present its general analytical approach and provide an opportunity for staff's questions.

DEC plans to use two computer codes in its assessments. The BADGER Code would be used to measure boron and the RACKLIFE Code to predict boraflex degradation. Also, DEC plans to test for available boraflex using neutron blackness testing every 3 years. The staff indicated that the review and approval of credit for boraflex is a first-of-a-kind that will require additional time than just the approval for soluble boron.

Enclosure 1 lists the meeting participants. Enclosure 2 is the handout provided by DEC.

Frank Rinaldi, Project Manager Project Directorate II-2 Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

Docket Nos. 50-369 and 50-70

Distribution: See next page

Enclosures:

1. Meeting Attendees

2. DEC Handout

cc w/encls: See next page

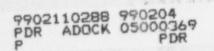
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NAME	FRinaldi:	LBerry A	HBerkow /			
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UNITED STATES NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

February 4, 1999

LICENSEE:

Duke Energy Corporation

FACILITIES:

McGuire Nuclear Station, Units 1 and 2

SUBJECT:

MEETING SUMMARY - MEETING OF JANUARY 27, 1999, REGARDING

SOLUBLE BORON AND BORAFLEX

REFERENCES: Meeting Notice by F. Rinaldi, dated January 11, 1999

On January 27, 1999, the staff met with Duke Energy Corporation (DEC) personnel to discuss information related to a proposed license amendment to take credit for soluble boron and partial credit for boraflex in the spent fuel pools at the McGuire Nuclear Station. The meeting was scheduled at the request of DEC to present its general analytical approach and provide an opportunity for staff's questions.

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Frank Rinaldi, Project Manager

Project Directorate II-2

Division of Reactor Projects - I/II Office of Nuclear Reactor Regulation

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Docket Nos. 50-369 and 50-70

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cc w/encls: See next page

McGuire Nuclear Station

CC:

Ms. Lisa F. Vaughn Legal Department (PBO5E) Duke Energy Corporation 422 South Church Street Charlotte, North Carolina 28201-1006

County Manager of Mecklenburg County 720 East Fourth Street Charlotte, North Carolina 28202

Michael T. Cash
Regulatory Compliance Manager
Duke Energy Corporation
McGuire Nuclear Site
12700 Hagers Ferry Road
Huntersville, North Carolina 28078

J. Michael McGarry, III, Esquire Winston and Strawn 1400 L Street, NW. Washington, DC 20005

Senior Resident Inspector c/o U.S. Nuclear Regulatory Commission 12700 Hagers Ferry Road Huntersville, North Carolina 28078

Dr. John M. Barry
Mecklenberg County
Department of Environmental
Protection
700 N. Tryon Street
Charlotte, North Carolina 28202

Mr. Steven P. Shaver Senior Sales Engineer Westinshouse Electric Company 5929 Carnegie Blvd. Suite 500 Charlotte, North Carolina 28209 Ms. Karen E. Long Assistant Attorney General North Carolina Department of Justice P. O. Box 629 Raleigh, North Carolina 27602

L. A. Keller
Manager - Nuclear Regulatory
Licensing
Duke Energy Corporation
526 South Church Street
Charlotte, North Carolina 28201-1006

Regional Administrator, Region II U.S. Nuclear Regulatory Commission Atlanta Federal Center 61 Forsyth Street, S.W., Suite 23T85 Atlanta, Georgia 30303

Elaine Wathen, Lead REP Planner Division of Emergency Management 116 West Jones Street Raleigh, North Carolina 27603-1335

Mr. Richard M. Fry, Director
Division of Radiation Protection
North Carolina Department of
Environment, Health and Natural
Resources
3825 Barrett Drive
Raleigh, North Carolina 27609-7721

Mr. T. Richard Puryear Owners Group (NCEMC) Duke Energy Corporation 4800 Concord Road York, South Carolina 29745

Mr. H. B. Barron Vice President, McGuire Site Duke Energy Corporation 12700 Hagers Ferry Road Huntersville, North Carolina 28078

Distribution for Meeting Summary dated February 4, 1999

E-MAIL

S. Collins/R. Zimmerman

B. Boger

J. Zwolinski

H. Berkow

I. Berry

L. Kopp

D. Jackson

K. Parczewski

C. Lauron

M. Tschiltz, EDO

L. Plisco, RII

C. Ogle, RII

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F. Rinaldi

OGC

ACRS

LIST OF PARTICIPANTS

MEETING WITH DUKE ENERGY CORPORATION

FEBRUARY 4, 1998

NAME	AFFILIATION
Frank Rinaldi Mike Cash Julius Bryant Keith Waldrop Paul Guill Gary Walden Jimmy Glenn Diana Jackson Kris Parczewski Carolyn Lauron	NRC/NRR/Project Directorate II-2 Duke Energy Corporation, Regulatory Compliance Duke Energy Corporation, Regulatory Compliance Duke Energy Corporation, Fuel Management/Criticality Duke Energy Corporation, Nuclear Engineering Duke Energy Corporation, Nuclear Engineering Duke Energy Corporation, Nuclear Engineering NRC/NRR/Plant Systems Branch NRC/NRR/Materials and Chemical Engineering Branch NRC/NRR/Materials and Chemical Engineering Branch
Larry Kopp	NRC/NRR/Reactor Systems Branch



Management of Boraflex Issues at McGuire Nuclear Station

NRC Meeting

January 27, 1999

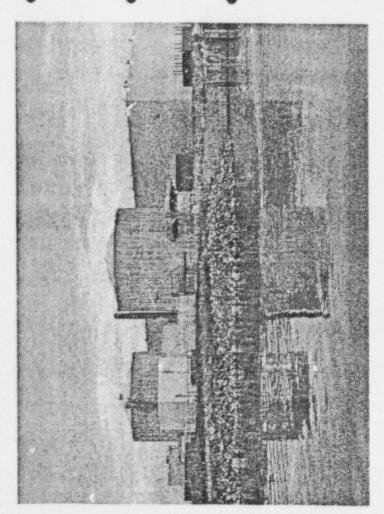


Meeting Purpose

- Review status of Boraflex degradation and subsequent corrective actions;
- Review and discuss proposed Technical
 Specification and regulatory commitments.



McGuire Nuclear Station (MNS)



- Westinghouse Two Unit PWR
- Dedicated Spent Fuel
 Pool for each unit.
- Each Spent Fuel Pool is a two region pool.



MNS Spent Fuel Pool Overview

- High density re-rack with Boraflex design in both McGuire units;
- Duke has monitored pools and racks to detect degradation of Boraflex;
- There are indications of Boraflex degradation in both units' racks.



Industry Experience

- Research started in 1987;
- Industry User's Group formed;
- Enhanced measurement and computer model for monitoring Boraflex health developed in 1996;

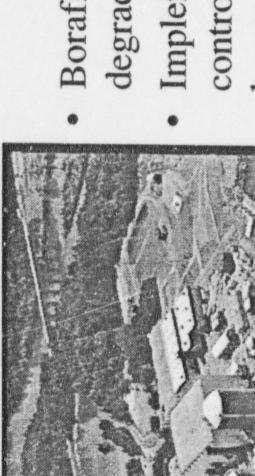


McGuire Efforts

- Neutron "blackness" testing in 1991;
- Duke Power Company has provided responses to NRC Generic Letter 96-04;
- BADGER demonstration in January 1997;
- · Criticality analysis revised;
- A MNS-specific RACKLIFE model was developed to assist in assessment of pool degradation;
- Monitoring pool chemistry.



Current Condition at MNS



- Boraflex panels are degraded;
- Implemented admin controls to ensure design bases are maintained;
- Both pools are "Operable" with admin controls.



MNS Options Considered

- · Poison inserts;
- Restrictive storage patterns;
- Re-rack Region One;
- · Boraflex degradation chemical inhibitors;
- Implement WCAP-14416 methodology.



Option Chosen

MNS has chosen to implement WCAP-14416 methodology for taking credit for soluble boron in place of degrading Boraflex.



Tech Spec Submittal

- Based on WCAP methodology;
- Criticality analysis indicates current fuel inventories cannot be accommodated using WCAP methodology alone;
- WCAP methodology does not consider degrading poison material;
- MNS seeking partial credit for Boraflex.



Proposed TS Submittal

- Defines two sub-regions in Region 1:
 - Region 1A (partial Boraflex)
 - Region 1B (no Boraflex)
- Defines two sub-regions in Region 2:
 - Region 2A (partial Boraflex)
 - Region 2B (no Boraflex)
- 3 storage patterns in each region:
 - Unrestricted
 - Restricted w/ Filler
 - Checkerboard w/ empty cells
- New SLC for poison material limit;
- Propose defining sub-regions in Bases.

DRAFT SLC

Spent Fuel Pool Storage Rack Poison Material

LCO

The storage rack poison material Boron 10 areal density shall be greater than or equal to:

 $0.005~\mathrm{gm~B_{10}/cm^2}$ for Region 1A 0 gm B₁₀/cm² for Region 1B 0.003 gm B₁₀/cm² for Region 2A 0 gm B₁₀/cm² for Region 2B

APPLICABILITY:

When a fuel assembly is stored in a spent fuel rack cell location.

ACTIONS

CONDITION		REQUIRED ACTION	COMPLETION TIME	
	******	LCO 3.0.3 is not applicable.		
Spent fuel pool storage rack poison material not within limit.	A.1	Perform SR 3.7.14.1	1 hour AND Once per 24 hours thereafter until affected fuel assembly moved	
	AND			
	A.2	Verify fuel assembly in affected location meets 3.7.15(b) for Region 1 or 3.7.15(d) for Region 2	1 hour	
Required Action and associated Completion Time of Condition A not met.	B.1	Initiate action to move affected fuel assembly to acceptable location	Immediately	
	Spent fuel pool storage rack poison material not within limit. Required Action and associated Completion Time	Spent fuel pool storage rack poison material not within limit. A.1 AND A.2 Required Action and associated Completion Time	Spent fuel pool storage rack poison material not within limit. AND A.2 Verify fuel assembly in affected location meets 3.7.15(b) for Region 1 or 3.7.15(d) for Region 2 Required Action and associated Completion Time B.1 Initiate action to move affected fuel assembly to acceptable	

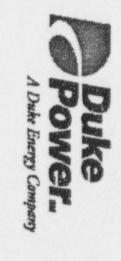
SURVEILLANCE REQUIREMENTS

	SURVEILLANCE	FREQUENCY	
SR	Verify the spent fuel pool storage rack poison material is within limits.	3 years	



Conclusions

- Tech Spec Submittal by 2/28/1999;
- Current Operability expires 12/31/1999;
- NRC approval needed by 12/31/1999;
- Dry Storage to begin in 2000 to provide additional storage capacity;
- Other options are still under consideration for long term resolution.



Wrap-up

Questions