**Proposed Technical Specification Revised Pages** 

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# TABLE OF CONTENTS

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Section		Page
3.16	SHOCK SUPPRESSORS (SNUBBERS)	3-63
3.17	REACTOR BUILDING AIR TEMPERATURE	3-80
3.18	FIRE PROTECTION (DELETED)	3-86
3.19	CONTAINMENT SYSTEMS	3-95
3.20	SPECIAL TEST EXCEPTIONS (DELETED)	3-95a
3.21	RADIOACTIVE EFFLUENT INSTRUMENTATION (DELETED)	3-96
3.21.1	RADIOACTIVE LIQUID EFFLUENT INSTRUMENTATION (DELETED)	3-96
3.21.2	RADIOACTIVE GASEOUS PROCESS AND EFFLUENT	3-96
	MONITORING INSTRUMENTATION (DELETED)	
3.22	RADIOACTIVE EFFLUENTS (DELETED)	3-96
3.22.1	LIQUID EFFLUENTS (DELETED)	3-96
3.22.2	GASEOUS EFFLUENTS (DELETED)	3-96
3.22.3	SOLID RADIOACTIVE WASTE (DELETED)	3-96
3.22.4	TOTAL DOSE (DELETED)	3-96
3.23	RADIOLGGICAL ENVIRONMENTAL MONITORING (DELETED)	3-96
3.23.1	MONITORING PROGRAM (DELETED)	3-96
3.23.2	LAND USE CENSUS (DELETED)	3-96
3.23.3	INTERLABORATORY COMPARISON PROGRAM (DELETED)	3-96
3.24	REACTOR VESSEL WATER LEVEL	3-128
4	SURVEILLANCE STANDARDS	4-1
4.1	OPERATIONAL SAFETY REVIEW	4-1
4.2	REACTOR COOLANT SYSTEM INSERVICE INSPECTION	4-11
4.3	TESTING FOLLOWING OPENING OF SYSTEM (DELETED)	4-13
4.4	REACTOR BUILDING	4-29
4.4.1	CONTAINMENT LEAKAGE TESTS	4-29
4.4.2	STRUCTURAL INTEGRITY	4-35
4.4.3	DELETED	4-37
4.4.4	AYDROGEN RECOMBINER SYSTEM	4-38
4.5	EMERGENCY LOADING SEQUENCE AND POWER TRANSFER	4-39
	EMERGENCY CORE COOLING SYSTEM AND REACTOR	
	BUILDING COOLING SYSTEM PERIODIC TESTING	
4.5.1	EMERGENCY LOADING SEQUENCE	4-39
4.5.2	EMERGENCY CORE COOLING SYSTEM	4-41
4.5.3	REACTOR BUILDING COOLING AND ISOLATION SYSTEM	4-43
4.5.4	ENGINEERED SAFEGUARDS FEATURE (ESF)	
	SYSTEMS LEAKAGE	4-45
4.6	EMERGENCY POWER SYSTEM PERIODIC TESTS	4-46
4.7	REACTOR CONTROL ROD SYSTEM TESTS	4-48
4.7.1	CONTROL ROD DRIVE SYSTEM FUNCTIONAL TESTS	4-48
4.7.2	CONTROL ROD PROGRAM VERIFICATION	4-50

-111-

Amendment 72, 81, 108, 129, 137, 146, 147, 158, 191, 197, 198,

### Bases

The Auxiliary and Fuel Handling Building Air Treatment System (part of the Auxiliary and Fuel Handling Building Ventilation System - References 1 and 2), consists of four banks of exhaust filters (AH-F2A, B, C, and D) and two sets of fans (AH-E-14A and C, and AH-E14B and D) which take the exhaust air from both the Auxiliary Building and the Fuel Handling Building and discharge it to the Auxiliary and Fuel Handling Building exhaust stack. The air normally passes through all four filter banks when either set of fans is in operation.

This system is not nuclear safety related. When available, it can be used to reduce the off-site dose releases; however, no credit was taken for this system in the analyses of the Waste Gas Tank Rupture (WGTR) Accident (Reference 4) or Maximum Hypothetical Accident (Reference 3), or for any other events releasing radioactivity through the Auxiliary Building. The dose consequences resulting from any of these events will be less than the 10 CFR 100 limits with or without system operation.

The in-place testing criteria for the HEPA and carbon adsorber banks, and the laboratory testing for the carbon adsorbers shall be performed in accordance with the test methods of ANSI/ASME N510-1980.

<u>Note</u>: The Fuel Handling Building ESF Air Treatment system controls the release resulting from a postulated spent fuel accident in the Fuel Handling Building per Technical Specification 3.15.4.

### References

- (1) UFSAR Section 9.8.2 "Fuel Handling Building Ventilation System"
- (2) UFSAR Section 9.8.3 "Auxiliary Building Ventilation System"
- (3) UFSAR Section 14.2.2.5 "Maximum Hypothetical Accident"
- (4) UFSAR Section 14.2.2.6 "Waste Gas Tank Rupture"

Amendment No. 55, 122, 157,

## ENGINEERED SAFEGUARDS FEATURE (ESF) SYSTEMS LEAKAGE

#### Applicability

454

Applies to those portions of the Decay Heat, Building Spray, and Make-Up Systems, which are required to contain post accident sump recirculation fluid, when these systems are required to be operable in accordance with Technical Specification 3.3.

#### Objective

To maintain a low leakage rate from the ESF systems in order to prevent significant off-site exposures and dose consequences.

Specification

4.5.4.1

The total maximum allowable leakage into the Auxiliary Building from the applicable portions of the Decay Heat, Building Spray and Make-Up System components as measured during refueling interval tests in Specification 4.5.4.2 shall not exceed 15 gellons per hour.

4.5.4.2

Once each refueling interval the following tests of the applicable portions of the Decay Heat Removal, Building Spray and Make-Up Systems shall be conducted to determine leakage:

- a. The applicable portion of the Decay Heat Removal System that is outside containment shall be leak tested with the Decay Heat pump operating, except as specified in "b".
- b. Piping from the Reactor Building Sump to the Building Spray pump and Decay Heat Removal System pump suction isolation valves shall be pressure tested at no less than 55 psig.
- c. The applicable portion of the Building Spray system that is outside containment shall be leak tested with the Building Spray pumps operating and BS-V-1A/B closed, except as specified in "b" above.
- d. The applicable portion of the Make-Up system on the suction side of the Make-Up pumps shall be leak tested with a Decay Heat pump operating and DH-V-7A/B open.
- e. The applicable portion of the Make-Up system from the Make-Up pumps to the containment boundary valves (MU-V-16A/D, 18, and 20) shall be leak tested with a Make-Up pump operating.
- f. Visual inspection shall be made for leakage from components of these systems. Leakage shall be measured by collection and weighing or by another equivalent method.

#### Bases

The leakage rate limit of 15 gph (measured in standard room temperature gallons) for the accident recirculation portions of the Decay Heat Removal (DHR), Building Spray (BS), and Make-Up (MU) systems is based on ensuring that potential leakage after a loss-of-coolant accident will not result in offsite dose consequences in excess of those calculated to comply with the 10 CFR 100 limits (Reference 1 and 2). The test methods prescribed in 4.5.4.2 above for the applicable portions of the DH, BS and MU systems ensure that the testing results account for the highest pressure within that system during the sump recirculation phase of a design basis accident.

References

- (1) UFSAR, Section 6.4.4 "Design Basis Leakage;" and, Table 6.4-3 "Leakage Quantities to the Auxiliary Building"
- (2) UFSAR, Section 14.2.2.5(d) "Effects of Engineered Safeguards Leakage During Maximum Hypothetical Accident"

Amendment No. 157

**ENCLOSURE 2** 

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Certificate of Service for TMI-1 TSCR No. 274

## UNITED STATES OF AMERICA

## NUCLEAR REGULATORY COMMISSION

## IN THE MATTER OF GPU NUCLEAR INC.

### **DOCKET NO. 50-289** LICENSE NO. DPR-50

## CERTIFICATE OF SERVICE

This is to certify that a copy of Technical Specification Change Request No. 274 Appendix A of the Operating License for Three Mile Island Nuclear Station Unit 1, has, on the date given below, been filed with executives of Londonderry Township, Dauphin County, Pennsylvania; Dauphin County, Pennsylvania; and the Pennsylvania Department of Environmental Resources, Bureau of Radiation Protection, by deposit in the United States mail, addressed as follows:

Mr. Darryl LeHew, Chairman Board of Supervisors of Londonderry Township R. D. #1, Geyers Church Road Middletown, PA 17057

Director, Bureau of Radiation Protection PA Dept. of Environmental Resources Rachael Carson State Office Building P.O. Box 8469 Harrisburg, PA 17105-8469 ATT: Mr. Stan Maingi

Ms. Sally S. Klein, Chairman Board of County Commissioners Dauphin County Courthouse Front and Second Streets Harrisburg, PA 17101

GPU NUCLEAR INC.

Fice President and Director, TMI

DATE: