NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

# LICENSEE EVENT REPORT (LER)

| FACILITY NAME (1) | DOCKET NUMBER (2) | PAGE (3) |
|---|---|---|
| Monticello | 0 5 0 0 0 2 6 3 | 1 OF 0 2 |

TITLE (4)

Isolation of Reactor Building Ventilation and Standby Gas Treatment Initiation

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | |
|---|---|---|---|---|---|---|---|---|---|---|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | DOCKET NUMBER(S) |
| | | | | | | | | | | 0 5 0 0 0 |
| 0 3 | 0 7 | 8 6 | 8 6 — | 0 0 8 — | 0 0 | 0 4 | 0 7 | 8 6 | | 0 5 0 0 0 |

| OPERATING MODE (9) | | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11) | | | |
|---|---|---|---|---|---|
| N | | 20.402(b) | 20.406(c) | X 50.73(a)(2)(iv) | 73.71(b) |
| POWER LEVEL (10) | | 20.405(a)(1)(i) | 50.36(c)(1) | 50.73(a)(2)(v) | 73.71(c) |
| 0 8 0 | | 20.405(a)(1)(ii) | 50.36(c)(2) | 50.73(a)(2)(vii) | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |
| | | 20.405(a)(1)(iii) | 50.73(a)(2)(i) | 50.73(a)(2)(viii)(A) | |
| | | 20.405(a)(1)(iv) | 50.73(a)(2)(ii) | 50.73(a)(2)(viii)(B) | |
| | | 20.405(a)(1)(v) | 50.73(a)(2)(iii) | 50.73(a)(2)(x) | |

LICENSEE CONTACT FOR THIS LER (12)

| NAME | TELEPHONE NUMBER |
|---|---|
| | AREA CODE |
| Robert S. Bayless, Instrument Engineer | 6 1 2 2 9 5 - 5 1 5 1 |

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | |
|---|---|---|---|---|---|---|---|---|---|---|---|
| | | | | | | | | | | | |
| | | | | | | | | | | | |

SUPPLEMENTAL REPORT EXPECTED (14)

| | | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
|---|---|---|---|---|---|
| YES (If yes, complete EXPECTED SUBMISSION DATE) | X NO | | | | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)


Isolations of the Reactor Building Ventilation and initiations of the Standby Gas Treatment System occurred during routine power operation. The isolations were caused by a spurious HIGH HIGH trip from Channel B Reactor Building Vent Wide Range Gas Monitor during performance of calibration 0359 and by a personnel error while performing the Reactor Building Exhaust Plenum Monitor Calibration. In both cases the isolations were reset immediately. Test switches will be installed to prevent spurious trips from occurring on monitors under test. Procedure 1268 will be revised to enhance the identification of the unit being calibrated.




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| FACILITY NAME (1) | DOCKET NUMBER (2) | LER NUMBER (6) | | | PAGE (3) |
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| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | |
| Monticello | 0 5 0 0 0 2 6 3 | 8 6 | — 0 0 8 | — 0 0 | 0 2 OF 0 2 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

During power operation on March 7, 1986, at 19:23, a Reactor Building Ventilation (VA ) isolation and Standby Gas Treatment System (BH) initiation occurred. This was caused by a spurious HIGH HIGH trip from Channel B Reactor Building Vent (RBV) Wide Range Gas Monitor (WRGM) (RIS) General Atomic part number 0366-0000-06.

At the time of the event, Procedure 0359, Reactor Building Vent Wide Range Gas Monitor Source Check, was being performed on Channel B WRGM. On performance of the source check test of the mid-range detector, a HIGH HIGH trip occurred. Investigation to determine the cause did not reveal any reason for the event other than a spurious trip.

During power operation on March 11, 1986, at 14:19, a Reactor Building Ventilation isolation and Standby Gas Treatment System initiation occurred. This was caused by a personnel error while performing Calibration 1268, Reactor Building Exhaust Plenum Monitor Calibration and Functional Check, due to an upscale trip from the Spent Fuel Pool Monitor (RA) (General Electric part number 129B2802G12).

During the performance of this procedure, the Instrument and Controls Specialist (I&CS) accidently pressed the trip check switch on the Channel A Spent Fuel Pool Monitor instead of the Channel A Plenum Monitor. This caused a HIGH trip which resulted in the isolation. The Channel A Fuel Pool Monitor is next to and identical to the Channel A Plenum Monitor which makes it possible to push the wrong button. In addition the Plenum Monitor is bypassed for this procedure while the Fuel Pool Monitor is not, thus pressing the trip check switch on the Fuel Pool Monitor will give an isolation. This was not a cognitive error, the I&CS made an error performing Procedure 1268.

These events had no effect on the public health or safety and were not the result of a defect or failure to comply. The safety function of the Wide Range Gas Monitors and Fuel Pool Monitor were initiated.

Test switches will be installed on the systems which will prevent HIGH trips from occurring on monitors under test. This will prevent an event like the first described here from occurring in the future. From discussions with the I&CS personnel, Procedure 12[illegible] will be revised to add steps to enchance the identification of the [illegible] being calibrated.

There have been no previous similar reportable events.

Northern States Power Company

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April 7, 1986

U S Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Isolation of Reactor Building Ventilation
and Standby Gas Treatment Initiation

The License Event Report for this occurrence is attached.

This event was reported via Emergency Notification System per 10 CFR Part 72 on March 7, 1986 and March 11, 1986.

Monica Vick
for David Musolf
Manager - Nuclear Support Services

DMM/MMV/dab

c: Regional Administrator-III, NRC
NRR Project Manager, NRC
Resident Inspector, NRC
MPCA
Attn: J W Ferman

Attachment

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