

Mr. J. E. Cross  
President-Generation Group  
Duquesne Light Company  
Post Office Box 4  
Shippingport, PA 15077

February 4, 1999

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION (RAI) REGARDING REVIEW OF  
BEAVER VALLEY POWER STATION, UNIT NO. 1 (BVPS-1) LICENSE  
AMENDMENT TO ALLOW ONE-TIME EXTENSION OF STEAM GENERATOR  
INSPECTION INTERVAL (TAC NO. MA4149)

Dear Mr. Cross:

By letter dated November 11, 1998, Duquesne Light Company (DLC) submitted for staff review an amendment request with a proposed license condition which would allow a one-time extension of the steam generator (SG) inspection interval of technical specification (TS) 4.4.5.3.b to coincide with the fuel cycle. The TS requirements of 4.4.5.3.b. are intended to ensure the SG tubes maintain margins consistent with Regulatory Guide 1.121. The Nuclear Regulatory Commission (NRC) staff has been reviewing the requested amendment, however, we have determined that additional information is required to complete our review. The requested information outlined in the enclosure is needed to ensure that there is adequate assurance that the necessary margins will continue to be maintained throughout the period of the extended operation. This was discussed during a January 21, 1999, conference call between NRC technical staff and Messrs. Greg Kammerdeiner, Gary Alberti, Lyle Berry, et al. of your staff. Subsequently, in discussions with Lyle Berry on January 27, 1999, a mutually agreeable target date for your response of February 26, 1999, was established. This schedule will support timely completion of the NRC's reviews of this amendment request. If circumstances result in the need to revise the target date, please call me at the earliest possible opportunity.

If you have any questions regarding this request, please contact me at (301) 415-1427.

Sincerely,

Original signed by:

Daniel S. Collins, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

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PDR ADOCK 05000334  
Q PDR

Docket No. 50-334

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Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

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UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Duquesne Light Company  
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If you have any questions regarding this request, please contact me at (301) 415-1427.

Sincerely,

A handwritten signature in cursive script, reading "Daniel S. Collins", is written over a horizontal line.

Daniel S. Collins, Project Manager  
Project Directorate I-1  
Division of Reactor Projects - I/II  
Office of Nuclear Reactor Regulation

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J. E. Cross  
Duquesne Light Company

Beaver Valley Power Station, Units 1 & 2

cc:

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West Virginia Department of Labor  
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Public Utilities Commission  
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Duquesne Light Company  
Beaver Valley Power Station  
PO Box 4  
Shippingport, PA 15077  
ATTN: Kevin L. Ostrowski, Division Vice  
President, Nuclear Operations Group  
and Plant Manager (BV-SOSB-7)

Bureau of Radiation Protection  
Pennsylvania Department of  
Environmental Resources  
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Mayor of the Borough of  
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Shippingport, PA 15077

Regional Administrator, Region I  
U.S. Nuclear Regulatory Commission  
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King of Prussia, PA 19406

Resident Inspector  
U.S. Nuclear Regulatory Commission  
Post Office Box 298  
Shippingport, PA 15077

Duquesne Light Company  
Beaver Valley Power Station  
PO Box 4  
Shippingport, PA 15077  
ATTN: S. C. Jain, Senior Vice President  
Nuclear Services (BV-A)

Mr. J. A. Hultz, Manager  
Projects & Support Services  
First Energy  
76 South Main Street  
Akron, OH 44308

REQUEST FOR ADDITIONAL INFORMATION  
REGARDING REVIEW OF  
LICENSE AMENDMENT TO ALLOW ONE-TIME EXTENSION OF STEAM GENERATOR  
INSPECTION INTERVAL  
BEAVER VALLEY POWER STATION, UNIT NO. 1  
DOCKET NO. 50-334

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- 1) For each degradation mechanism, provide a general description of the methodology for performing the operational assessment for End Of Cycle 13 and a description of the flaw growth rate and eddy current measurement error allowances used in the assessments and how they were determined. For Primary Water Stress Corrosion Cracking (PWSCC), the description should include an explanation of the model for assessing accident leak rate and the technical basis for the model.
- 2) What degradation mechanisms were in-situ tested? How were samples selected? How were conclusions derived? For the case of PWSCC at tight radius U-bends, what were the leak rates when tested at the postulated Main Steam Line Break accident pressure?