

LICENSEE EVENT REPORT (LER)

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|--|--------|-----------|--|---------------------|-----------------|-------|------------------|-----------|----------------|---|----------------------|-------------------------------|------------------|----------------------|--|------|
| FACILITY NAME (1) Indian Point Station Unit 2 | | | | | | | | | | DOCKET NUMBER (2) 0 5 0 0 0 2 1 4 7 | | | | PAGE (3) 1 OF 0 2 | | |
| TITLE (4) Environmental Qualification of Main Steam Flow Transmitters | | | | | | | | | | | | | | | | |
| EVENT DATE (5) | | | LER NUMBER (6) | | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | | | | | | |
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | | | | DOCKET NUMBER(S) | | | |
| 03 | 07 | 86 | 86 | 007 | 00 | 04 | 04 | 86 | | | | | 0 5 0 0 0 | | | |
| OPERATING MODE (9) | | | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 8: (Check one or more of the following) (11) | | | | | | | | | | | | | |
| POWER LEVEL (10) 0 0 0 | | | 20.402(b) | | | | 20.406(c) | | | | 80.73(a)(2)(iv) | | | | 73.71(b) | |
| | | | 20.406(a)(1)(i) | | | | 80.38(c)(1) | | | | 80.73(a)(2)(v) | | | | 73.71(c) | |
| | | | 20.406(a)(1)(ii) | | | | 80.38(c)(2) | | | | 80.73(a)(2)(vii) | | | | OTHER (Specify in Abstract below and in Text, NRC Form 366A) | |
| | | | 20.406(a)(1)(iii) | | | | 80.73(a)(2)(i) | | | | 80.73(a)(2)(viii)(A) | | | | | |
| | | | 20.406(a)(1)(iv) | | | | 80.73(a)(2)(ii) | | | | 80.73(a)(2)(viii)(B) | | | | | |
| | | | 20.406(a)(1)(v) | | | | 80.73(a)(2)(iii) | | | | 80.73(a)(2)(x) | | | | | |
| LICENSEE CONTACT FOR THIS LER (12) | | | | | | | | | | | | | | | | |
| NAME John Ellwanger | | | | | | | | | | TELEPHONE NUMBER 9 1 1 4 5 2 6 1 5 1 8 2 | | | | | | |
| COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13) | | | | | | | | | | | | | | | | |
| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | | | | | | |
| E | S | R | F | T | 1 | R | 1 | 3 | 6 | 1 | 9 | N | | | | |
| | | | | | | | | | | | | | | | | |
| SUPPLEMENTAL REPORT EXPECTED (14) | | | | | | | | | | | | EXPECTED SUBMISSION DATE (15) | | MONTH | DAY | YEAR |
| YES (If yes, complete EXPECTED SUBMISSION DATE) | | | | | | | | | | | | NO | | | | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On March 14, 1986 it was concluded that provisions of the Technical Specifications precluded heating Unit 2 above cold shutdown, when the main steam flow transmitters on two of four main steam lines were not resealed. Seal restoration with RTV is necessary for the transmitters to remain environmentally qualified. The previous RTV seal had been disrupted for maintenance purposes. It was not previously recognized that restoration of the environmental qualification status of these transmitters should have been completed before heating above 200°F.

The public health and safety were not affected.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

| FACILITY NAME (1) | DOCKET NUMBER (2) | LER NUMBER (6) | | | PAGE (3) | | |
|-----------------------------|-------------------|----------------|-------------------|-----------------|----------|----|----|
| | | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | | | |
| Indian Point Station Unit 2 | 05000247 | 86 | 007 | 00 | 02 | OF | 02 |

TEXT (If more space is required, use additional NRC Form 366A's) (17)

Plant and System Identification

Westinghouse 4-Loop Pressurized Water Reactor 900 MWe

Identification of Occurrence:

Transmitters 439A, 439B, 449A and 449B were not resealed with RTV at the time the plant was heated above cold shutdown (above 200°F).

Event Date: March 7, 1986

Discovery Date: March 14, 1986

Report Due Date: April 7, 1986

Past Similar Occurrences: None

Description of Occurrence:

During a planned refueling outage, routine surveillance was to be performed on the main steam flow transmitters. The procedures for this surveillance included environmental qualification requirements i.e., restoration of RTV sealant. On March 7, 1986 the plant was heated above 200°F, although the RTV had not been reapplied because it was believed that further maintenance would be conducted. The transmitters were operable and based on subsequent review we believe all intended functions under accident conditions would have been successfully fulfilled.

On March 11, 1986, during a walkdown inside containment, it was discovered that four transmitters had not been resealed. Immediate steps were taken to reseat these transmitters and the sealant was applied and properly cured and the status of the transmitter surveillance after the plant heatup was reviewed. Based on this review, which was completed on March 14, we concluded Unit 2 should not have been brought above 200°F without resealing the transmitters and the NRC was notified pursuant to 10CFR 50.72

Cause of Occurrence:

The cause of the occurrence involves not recognizing that restoration of the environmental qualification status of the main steam flow transmitters should have been completed before heating above 200°F.

Analysis of Occurrence:

Based on subsequent review, the transmitters were operable and we believe all intended functions under accident conditions would have been successfully fulfilled. Therefore, the public health and safety were not affected.

Corrective Actions:

Station procedures regarding startup checklist-type requirements are undergoing review to assure they contain sufficient details on environmental qualification. Training is in progress to increase awareness of Station personnel to environmental qualification requirements.

John D. O'Toole
Vice President

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April 7, 1986

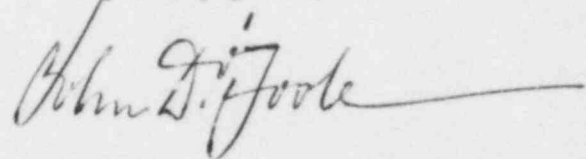
Re: Indian Point Unit No. 2
Docket No. 50-247
LER-86-007-00

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, D. C. 20555

Dear Sirs:

The attached Licensee Event Report LER-86-007-00 is hereby submitted in accordance with the requirements of 10 CFR Part 50.73.

Very truly yours,



attach.

cc: Dr. Thomas E. Murley,
Regional Administrator - Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pa. 19406

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