ALABAMA POWER COMPANY
JOSEPH M. FARLEY NUCLEAR PLANT
UNIT 1 - ANNUAL REPORT
REQUIRED BY 10CFR50.59 (1985)

Section 50.59 of Part 50, Licensing of Production and Utilization Facilities, of the regulations of the United States Nuclear Regulatory Commission, states that the holder of a license authorizing operation of a production or utilization facility may (1) make changes in the facility as described in the safety analysis report, (2) make changes in the procedures as described in the safety analysis report, and (3) conduct tests or experiments not described in the safety analysis report, without prior commission approval, unless the proposed change, test or experiment involves a change in the technical specifications incorporated in the license or an unreviewed safety question (as defined in 10CFR50.59).

The licensee is required to maintain records of such changes, tests or experiments, and these records are required to include written safety evaluations which provide the basis for the determination that the change, test or experiment does not involve an unreviewed safety question.

Brief descriptions and a summary of the safety evaluations of the changes, tests or experiments as described above which were completed in 1985 for the Joseph M. Farley Nuclear Plant Unit 1, are provided in the following pages.

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Subject: Cycle 7 Reload Safety Evaluation

Description: The Unit 1 Cycle 7 fuel reload consisted of 2 Region 4A, 2 Region 7, 77 Region 8, and 76 fresh Region 9 fuel assemblies. The 2 Region 4A bundles are two Optimized Fuel Demonstration Assemblies (OFDAs) which are being reinserted in Cycle 7 from the spent fuel pool for further irradiation. A total of 672 fresh standard burnable poison rodlets were utilized in clusters of 4, 12, and 16 rodlets. All of the assemblies used in Cycle 7 have antisnag grid designs except for the two OFDAs.

Following return to power, it was discovered that a misload had occurred involving one of the OFDAs and another spent fuel assembly.

Safety Evaluation: A Reload Safety Evaluation was conducted by Westinghouse. The reload design was found to be in compliance with current Technical Specifications and to involve no unreviewed safety questions per 10CFR50.59. This safety evaluation was later determined to be inadequate (See ler 86-003-00 Docket No. 50-348).

Following determination that a misload had occurred, this condition was evaluated and determined to have no safety significance.

PORC Review: PORC Meeting 1368, 3/21/85
PORC Meeting 1419, 8/8/85
PORC Meeting 1431, 9/10/85

Subject: FNP-1-AOP-2.0, TCN 5A

Description: The final Safety Analysis Report indicates that the steam generator blowdown isolation valves close automatically on a high radiation signal from radiation monitor R-19 in addition to signals from R-23A and R-23B. This is contrary to design drawings and actual tests which indicate that these valves do not close on a signal from R-19. This procedure was changed to reflect the actual design of the plant.

Safety Evaluation: This change was made to reflect the existing design of the plant. The existing design was determined to be the proper design.

PORC Review: PORC Meeting 1450, 11/7/85

Subject:	FNP-0-AP-2, Revision 9 "Plant Operations Review Committee"
Description:	Revised procedure to show new membership of the Plant Operations Review Committee (PORC) due to plant management changes and made minor changes in the review requirements of the PORC.
Safety Evaluation:	These changes increase the effectiveness of the PORC and plant management.
PORC Review:	PORC Meeting 1370, 3/28/85
Subject:	FNP-0-AP-3, Revision 4 "Plant Organization"
Description:	Revised procedure to show the new plant management and changed the educational requirements of the Systems Performance Supervisor.
Safety Evaluation:	These changes increase the effectiveness of the plant management.
PORC Review:	PORC Meeting 1370, 3/28/85
Subject:	FNP-0-AP-45, Revision 5 "Farley Nuclear Plant Training Plan"
Description:	Revised procedure to not allow exempting operators from annual retraining based on the previous year's examination performance.
Safety Evaluation:	This change increases the effectiveness of the training program.
PORC Review:	PORC Meeting 1371, 3/29/85

Subject:	FNP-O-FHP-1.0, Revision 10
Description:	This procedure provides the method used to remove the reactor vessel head during refueling. This method involves removing the reactor vessel head and placing it on the storage stand prior to flooding the refueling cavity. This is different from the procedure described in the Final Safety Analysis Report in which the head is raised while simultaneously flooding the cavity.
Safety Evaluation:	This method reduces the radiation exposure involved by shortening the duration of the job. The extremely difficult coordination effort of raising the head and cavity level simultaneously is eliminated. The time the head is suspended from the polar crane is reduced.
PORC Review:	PORC Meeting 1400, 6/18/85
Subject:	FNP-O-FHP-3.1, Revision 9, TCN 0B
Description:	The Final Safety Analysis Report states that new fuel assemblies are moved in the new fuel area by use of the new fuel assembly handling fixture suspended from the new fuel hoist. This procedure change allowed for the use of slings to lift a dropped fuel assembly from the horizontal position since the handling fixture could not be used.
Safety Evaluation:	The handling of a single damaged unburned assembly is safe from a criticality standpoint. The undamaged assembly is the most reactive geometry, i.e., the assembly will not become significantly more reactive if it is further deformed when it is lifted and placed in the shipping container.
PORC Review:	PORC Meeting 1458, 11/26/85

Subject:	FNP-1-SOP-50.0, Revision 18
Description:	During a review of this procedure, it was found that the Waste Evaporator Feed Pump and Floor Drain Tank recirc valves were listed as "locked throttled" which was contrary to system drawings and the Final Safety Analysis Report (FSAR). Upon investigation it was found that "locked throttled" was the desired condition and that the system drawings and FSAR should be changed. Further, this revision changed the normal required positions for the following valves from "locked open" to "locked throttled": Waste Evaporator Concentrate Tank pump recirc valve Chemical Drain Tank pump recirc valve Laundry and Hot Shower Tank pump recirc valve
Safety Evaluation:	The purpose of the recirc path is to provide minimum flow protection for the pumps and recirc mixing. According to startup test procedures, the recirc valves for all waste processing system pumps should have been throttled to obtain a nominal recirc flow sufficient for pump protection and recirc mixing.
PORC Review:	PORC Meeting 1350 2/8/85

Subject:	FNP-1-SOP-50.0 Revision 18, TCN 18C
Description:	Changed the normal required positions for the Reactor Coolant Drain Tank (RCDT) pump discharge manual isolation valves and the RCDT pump recirc orifice outlet isolation valve from "locked open" to "locked throttled". Having these valves in the locked open position has resulted in the RCDT pumps exceeding design flow. Also, having the recirc valve wide open was causing insufficient head for transfering tank contents through the discharge path.
Safety Evaluation:	The valves have been throttled to conform with pump design and pre-operational testing results.
PORC Review:	PORC Meeting 1389, 5/14/85
Subject:	PCR/PCN 78-209 (S78-1-209)
Description:	Installed two additional pressure gauges to measure Boron Thermal Regeneration System (BTRS) chiller pump suction pressure.
Safety Evaluation:	The pressure range of the existing gauges was too high to accurately determine pump suction pressures.
PORC Review:	PORC Meeting 1387, 5/7/85
Subject:	PCR/PCN 79-449 (B79-449)
Description:	Made several changes associated with the concentrated waste storage tank to reduce personnel exposure.
Safety Evaluation:	These changes in no way adversely impact the safety of the plant since the system is not a safety system, nor does it impact a safety system.
PORC Review:	PORC Meeting 826, 6/02/81 PORC Meeting 995, 6/15/82 PORC Meeting 1010, 7/27/82

Subject:	PCR/PCN 79-566 (B79-566 & SE79-566)
Description:	Added P.A. speakers, amplifiers and new flashing beacon lamps in the Auxiliary Building and the Containment Building.
Safety Evaluation:	These additions improve plant emergency evacuation alarm coverage and satisfy a commitment made to the NRC in IE Bulletin 79-18.
PORC Review:	PORC Meeting 590, 8/21/80 PORC Meeting 611, 9/23/80 PORC Meeting 629, 10/29/80 PORC Meeting 824, 5/28/81 PORC Meeting 852, 7/15/81 PORC Meeting 987, 5/18/82
Subject:	PCR/PCN 81-975 (S81-975)
Description:	Added a service water standpipe with a surge tank to provide backpressure on the service water system and removed existing backpressure control valves Q1P16V559A and B.
Safety Evaluation:	The standpipe and surge tank were seismically qualified to protect adjacent class 1E electrical equipment; however, the standpipe and surge tank are in a non-seismic portion of the service water system. Operability of the service water system is not contingent upon the standpipe and surge tank since, by transferring to pond recirc mode and bypassing the non-seismic portion of the system, adequate system backpressure is maintained.
PORC Review:	PORC Meeting 1200, 1/10/84 PORC Meeting 1204, 11/24/84
* Subject:	PCR/PCN 81-0-1037-1A-132 (SM81-0-1037-1A-132)
Description:	Installed pre-action sprinkler system 1A-132 in auxiliary building room 107 to facilitate use of this room as an occasional combustible materials storage area. Also the room boundary was upgraded to a 3-hour fire rating.
Safety Evaluation:	This design was evaluated for its effect on other systems and was found not to interfere with or degrade other installed systems.
PORC Review:	PORC Meeting 1443, 10/10/85

* Subject:	PCR/PCN 81-0-1037-1A-134 (SM81-0-1037-1A-134)
Description:	Installed pre-action sprinkler system 1A-134 to cover auxiliary building rooms 601, 602, 603, 2601, 2602, and 2603 to facilitate use of these rooms for occasional combustible materials storage.
Safety Evaluation:	This design has been evaluated for its effect on other systems and has been found not to interfere with or degrade other installed systems.
PORC Review:	PORC Meeting 1443, 10/10/85
* Subject:	PCR/PCN 81-0-1037-603 (B81-0-1037-603)
Description:	Upgraded rooms 601, 602, 603, 2601, 2602, and 2603 to be a common three hour rated fire area for use as a combustible materials storage area.
Safety Evaluation:	This design has been evaluated for its effect on other systems and has been found not to interfere with or degrade other installed systems.
PORC Review:	PORC Meeting 1415, 8/1/85
* Subject:	PCR/PCN 81-1167 (S81-0-1167)
Description:	Piping and associated valves were added in order to connect the Unit 2 Service Water standpipe inlet to the fire pump suction header.
Safety Evaluation:	This addition provides a supply of water to the fire pumps in case the fire protection storage tanks are depleted for any reason and has no adverse effect on service water system operation.
PORC Review:	PORC Meeting 1297, 10/9/84
Subject:	PCR/PCN 82-1228 (B82-1228)
Description:	Modified annunciation circuit for loss of voltage tripping busses 1F, 1G, 1H, 1J, and degraded grid voltage tripping busses 1F and 1G to provide immediate targeted information to aid in troubleshooting a tripped bus.
Safety Evaluation:	This modification in no way adversely affects the safe operation of the plant.
PORC Review:	PORC Meeting 1299, 10/12/84

Subject:	PCR/PCN 82-1-1250.1 (S82-1-1250.1)
Description:	Replaced the 3A, 3B, 4A, and 4B feedwater heaters.
Safety Evaluation:	Copper in the original feedwater heaters is suspected to be a major contributor to steam generator tube denting. The new heaters utilize stainless steel tubes. The new heaters are designed to meet or exceed the old heaters' performance with the operation being similar. Replacement of the feedwater heaters improves the system's performance without degrading system safety. These heater replacements are expected to assist in prolonging steam generator longevity.
PORC Review:	PORC Meeting 1193 12/13/83

Subject:	PCR/PCN 82-1-1250.2 (S82-1-1250.2)
Description:	Replaced the 1A, 1B, 2A, 2B, 5A, 5B, 6A, and 6B feedwater heaters.
Safety Evaluation:	Copper in the original feedwater heaters is suspected to be a major contributor to steam generator tube denting. The new heaters are designed to meet or exceed the old heater's performance with operation being similar. Replacement of the feedwater heaters improves the system's performance without degrading system safety. These heater replacements are expected to assist in prolonging steam generator longivity.
PORC Review:	PORC Meeting 1353, 2/14/85

Subject:	PCR/PCN 82-1328
Description:	Replaced sample pumps for the containment air sample monitor. Also, heat tracing was added to the incoming sample lines from containment and a valve modification was made to improve blower performance.
Safety Evaluation:	The new pumps have all metal internal parts which should reduce maintenance requirements. In addition, system function will be improved due to reduced condensation in the radiation monitor pumps and reduced back pressure on the sample pump.
PORC Review:	PORC Meeting 1126, 6/28/83

Subject: PCR/PCN 83-1419 (S83-1-1419)

Description: Replaced existing carbon steel service water supply and drain piping valves for Diesel Generators 1-2A, 1B, 1C and 2C air start system aftercoolers with stainless steel.

Safety Evaluation: This replacement will allow proper cooling water flow to the aftercoolers. This will reduce maintenance and operational problems associated with corrosion of existing carbon steel piping. This modification will not reduce the integrity of the system and does not constitute an unreviewed safety question.

PORC Review: PORC Meeting, 1402 6/25/85
PORC Meeting, 1418 8/8/85

Subject: PCR/PCN 83-1465 (B83-1465)

Description: Provided additional control capability, from the hot shutdown panel, for the following components, previously controlled from the main control board only.

Main Steam Isolation Valves	SV-3369A, B, and C
CCW Heat Exchanger Discharge Valve	MOV-3047-B
RWST to Charging Pump Suction Valves	LCV-0115B and D
Pressurizer Power Operated Relief Valves (PORVs)	PCV-0444B, and 0445A
Pressurizer PORV Block Valves	MOV-8000A, and B
Reactor Vessel Head Vent Valves	SV-2213A, B, and SV-2214A, B
Seal Water Injection Valve	HCV-0186

Also, this design change provided an RCS Tcold indication and a condensate storage tank level indication on the hot shutdown panel.

Safety Evaluation: These modifications were in response to commitments made to the NRC. They comply with the requirements of 10CFR50, Appendix R, and increase the safe operation of the plant.

PORC Review: PORC Meeting, 1346 1/29/85

Subject:	PCR/PCN 83-2489 (S83-1-2489)
Description:	Added vent valves to the turbine steam bypass line, downstream of the main steam bypass valves.
Safety Evaluation:	This modification provides the capability to test the bypass valves individually for leakage. This modification already exists on Unit 2.
PORC Review:	PORC Meeting 1307, 11/2/84 PORC Meeting 1348, 2/5/85
Subject:	PCR/PCN 84-2518 (B84-1-2518)
Description:	Add temperature monitoring system to detect check valve leakage in the auxiliary feedwater system.
Safety Evaluation:	This change addresses the issue of auxiliary feedwater pump steam binding due to leakage of system check valves. This change has been evaluated for its effect on other installed systems.
PORC Review:	PORC Meeting 1269, 8/7/84
Subject:	PCR/PCN 84-2644 (B84-1-2644)
Description:	Replaced existing flow sensing instrumentation in the battery room exhaust fan ducts.
Safety Evaluation:	The previously existing instrumentation was in constant alarm. The ventilation system was determined to have no design deficiency. The purpose of the flow sensing instrumentation is to measure pressure drop across the system fans to ensure fan operation, but the actual pressure drop was too low to be sensed by the previously existing instrumentation.
PORC Review:	PORC Meeting 1356, 2/21/85
Subject:	PCR/PCN 84-2678 (S84-0-2678)
Description:	Assigned TPNS numbers to eight thermostats which control the operation of the control room ventilation duct heating systems covering both units.
Safety Evaluation:	This change assigns TPNS numbers to existing thermostats and does not degrade the safe shutdown capabilities of the plant.
PORC Review:	PORC Meeting 1387, 5/7/85

Subject:	PCR/PCN 84-2721 (B84-0-2721)
Description:	Modified the existing HVAC system for the computer room to adequately cool the room upon installation of Safety Parameter Display System (SPDS) equipment.
Safety Evaluation:	This design supports the implementation of the SPDS which is required by NUREG-0737, Supplement 1.
PORC Review:	PORC Meeting 1311, 11/13/84 PORC Meeting 1387, 5/7/85

Subject:	PCR/PCN 84-2726 (B84-1-2726)
Description:	Installed a normally locked open valve downstream of normal charging containment penetration #24 test connection valves QV407A and QV407B. This is to facilitate the performance of the Local Leak Rate Test (LLRT) of this penetration without the need to use four separate boundary valves which are not part of the acceptance criteria for the LLRT, but experience has shown will leak--delaying the LLRT.
Safety Evaluation:	Installation of this valve enhances the ability to perform a meaningful LLRT. Since the valve will be locked open at all times, except during the LLRT, there will be no adverse effect on plant operation or plant safety.
PORC Review:	PORC Meeting 1346, 1/29/85

Subject:	PCR/PCN 84-1-2742 (B84-1-2742)
Description:	Modified the extended range containment pressure instruments PT0950Y and Z to provide adequate train separation.
Safety Evaluation:	This modification complies with regulatory guide 1.97.
PORC Review:	PORC Meeting 1394, 5/28/85

Subject:	PCR/PCN 84-2773 (S84-1-2773)
Description:	Installed differential pressure gauges across each charging pump oil cooler to supplement existing pressure gauges in detecting developing oil cooler service water side plugging problems.
Safety Evaluation:	The additional gauges will inform plant operators of developing oil cooler problems without degrading plant safety.
PORC Review:	PORC Meeting 1297, 10/9/84

Subject:	PCR/PCN 84-2821 (B84-1-2821)
Description:	Routed the fuel transfer tube Local Leak Rate Test (LLRT) test connections outside of the concrete shielding. This provided a means for performing the LLRT of the bellows from the auxiliary building hallway.
Safety Evaluation:	Piping the test connection to the hallway is necessary to allow access to the test point without the major effort required to remove and replace shielding. The piping design complies with the applicable codes and standards.
PORC Review:	PORC Meeting 1306, 10/31/84

Subject:	PCR/PCN 84-2824 (S84-0-2824)
Description:	Installed a vertical scale indicator to provide local level indication for the demineralized water storage tank.
Safety Evaluation:	A local level indicator was installed in the tank level transmitter circuit of the demineralized water storage tank due to the existing mechanical level indicator being broken and beyond repair. Since the demineralized water system is not required for any safety related functions, this change will not degrade the safe shutdown capabilities of the plant.
PORC Review:	PORC Meeting 1387, 5/7/85 PORC Meeting 1452, 11/12/85

Subject:	PCR/PCN 84-2883 (S84-1-2883)
Description:	Removed pressure controller Q1P16PC667, high select relay N1P16V730, and the associated pressure control valve and tubing from the control logic for valve Q1P16V560.
Safety Evaluation:	Prior to installation of the service water surge tank, Q1P16PC667 existed to detect high service water system pressure and relieve it by opening Q1P16V560. With the installation of the service water surge tank, this function is no longer required.
PORC Review:	PORC Meeting 1409, 7/23/85
Subject:	PCR/PCN 84-1-2960 (S84-1-2960)
Description:	Replaced the existing carbon steel moisture separator reheater (MSR) first stage extraction steam piping with stainless steel piping.
Safety Evaluation:	The carbon steel piping has been experiencing wall thinning due to erosion/corrosion by wet steam. This change affects only the MSR first stage extraction steam piping which has no safety function and is not safety-related.
PORC Review:	PORC Meeting 1348, 2/5/85
Subject:	PCR/PCN 84-1-2961 (S84-1-2961)
Description:	Replaced existing carbon steel extraction steam piping to 6A and 6B feedwater heaters with stainless steel piping.
Safety Evaluation:	The carbon steel piping has been experiencing wall thinning due to erosion/corrosion by wet steam. This change affects only the extraction steam piping to feedwater heaters 6A and 6B. This system has no safety function and is not safety-related.
PORC Review:	PORC Meeting 1348, 2/5/85

Subject:	PCR/PCN 84-1-3003 (B84-1-3003)
Description:	To allow location of the Unit 1 Safety Parameter Display System (SPDS) computer hardware in the existing Unit 1 computer room, the Nuclear Data Link functions of the P2550 Data Link System were consolidated into the Unit 2 SPDS computer.
Safety Evaluation:	This change has no impact on safety systems. Use of this system was deemed acceptable with no impact on plant safety as long as the appropriate procedures were updated.
PORC Review:	PORC Meeting 1350, 2/8/85

Subject:	PCR/PCN 84-3020 (B84-0-3020)
Description:	Removed the interlock mechanism on Motor Control Center (MCC) "F" cubicle door which tripped the MCC if the door was opened.
Safety Evaluation:	This design change was accomplished within the applicable electrical codes and standards. This change prevents the loss of safety-related loads if the cubicle door is inadvertently opened.
PORC Review:	PORC Meeting 1408, 7/19/85

Subject:	PCR/PCN 84-3021 (B84-0-3021)
Description:	Removed the interlock mechanism on Motor Control Center (MCC) "G" cubicle door which tripped the MCC if the door was opened.
Safety Evaluation:	This design change was accomplished within the applicable electrical codes and standards. This change prevents the loss of safety-related loads if the cubicle door is inadvertently opened.
PORC Review:	PORC Meeting 1408, 7/19/85 PORC Meeting 1434, 9/19/85

Subject:	PCR/PCN 84-3044 (S84-1-3044)
Description:	The pipe downstream of component cooling water system drain valve Q1P17V113A was cut, threaded, and a pipe cap was installed.
Safety Evaluation:	The pipe downstream of this drain valve was previously shown on the Final Safety Analysis Report figure as being open ended. This change in no way adversely affects safe system operation.
PORC Review:	PORC Meeting 1395, 5/30/85

Subject:	PCR/PCN 85-3072 (S85-1-3072)
Description:	Changed component cooling water system heat exchanger 1B pressure sensing line from 3/4" carbon steel piping and valves to 3/4" stainless steel piping and valves.
Safety Evaluation:	The carbon steel pressure sensing lines and valves were becoming clogged. This change in no way adversely affects safe system operation.
PORC Review:	PORC Meeting 1387, 5/7/85

Subject:	PCR/PCN 85-3073 (B85-1-3073)
Description:	Replaced two elbows in pipe HCD-412 between the plant vent stack sample nozzle and the Sping-4 inlet with pipe bends having a minimum radius of 5 times the pipe diameter.
Safety Evaluation:	This modification is in accordance with ANSI N13.1-1969 and, based on engineering judgement, this modification will not have adverse effects on existing design.
PORC Review:	PORC Meeting 1343, (1/22/85)

Subject:	PCR/PCN 85-3075 (B85-1-3075)
Description:	Replaced the existing overcurrent trip unit on breaker LB02 with a unit having a lower trip setting.
Safety Evaluation:	This modification protects power cable from overheating inside Kaowool wrap. The new trip setting is still far in excess of normal full load current.
PORC Review:	PORC Meeting 1351, 2/12/85

Subject:	PCR/PCN 85-3094 (S85-1-3094)
Description:	Updated drawing D-175042, sheets 1, 2, 3, and 4 for processing waste water via the Disposable Demineralizer System as the primary method of waste treatment.
Safety Evaluation:	This change in no way adversely affects safe plant operation.
PORC Review:	PORC Meeting 1400, 6/18/85

Subject:	PCR/PCN 85-3154 (S85-1-3154)
Description:	Revised drawings to document the removal of the control and all but the final driver cards in the instrument loops for valves Q1P16FV3009A, B, C.
Safety Evaluation:	The removal of the control cards from cabinets Q1H11NGB2504J and 2504K (Process Control Cabinets) will have no effect on the operation of the other devices located in these cabinets. Since the plant always operates with these service water valves in the manual control mode, there will be no impact on safe plant operation.
PORC Review:	PORC Meeting 1405, 7/9/85

Subject:	PCR/PCN 85-3161 (S85-1-3161)
Description:	Realigned containment venting system manual isolation valve from "normally closed" to "normally open".
Safety Evaluation:	This change does not impact the design of the system with respect to any of the system design bases including containment isolation. This change eliminates the need to make a "reentry" during postulated accident conditions to manually open this valve to allow containment venting for hydrogen concentration control.
PORC Review:	PORC Meeting 1402, 6/25/85

Subject:	PCR/PCN 85-3175 (S85-1-3175)
Description:	Changed the drawing designation from "locked open" to "locked throttled" for the recirculation valves for the waste evaporator feed pump, the waste evaporator concentrate pump, the chemical drain tank pump, the laundry and hot shower tank pump and the floor drain tank pump.
Safety Evaluation:	The purpose of the recirculation lines is to provide minimum flow protection for the pumps and to allow for recirculation mixing of the tank contents. It is not desirable from a pump design standpoint to have the recirculation valves wide open. With the valve throttled, there is still sufficient flow for tank mixing.
PORC Review:	PORC Meeting 1400, 6/18/85

Subject:	PCR/PCN 85-3233 (B85-1-3233)
Description:	Re-routed the residual heat removal system heat exchanger room sump cover vent to the waste holdup tank room as on Unit 2.
Safety Evaluation:	The vent previously extended into the component cooling water system heat exchanger room creating a pathway for radioactive gases to enter the non-rad side of the Unit 1 auxiliary building.
PORC Review:	PORC Meeting 1403, 6/27/85

Subject:	PCR/PCN 85-3234 (B85-1-3234)
Description:	Plugged the Unit 1 non-rad side elevator shaft drain.
Safety Evaluation:	The existing drain was tied into the rad-side waste gas processing sump. This created a pathway for radioactive gases to enter the non-rad side of the auxiliary building.
PORC Review:	PORC Meeting 1408, 7/19/85

Subject:	PCR/PCN 85-3238 (B85-1-3238)
Description:	Revised drawing D-175002 sheet 2 to show the actual, correct, arrangement of component cooling water system low air pressure switches Q1P17PSL3184D and Q1P17PSL3045, for solenoid valves Q1P175SV3184 and Q1P175SV3045.
Safety Evaluation:	This change corrected a drawing error only and in no way adversely affects safe system operations.
PORC Review:	PORC Meeting 1447, 10/24/85

Subject:	PCR/PCN 85-3265 (S85-1-3265) PCR/PCN 85-3266 (S85-1-3266)
Description:	Replaced the small diameter (less than two inch) carbon steel service water piping and valves in the auxiliary and turbine buildings with stainless steel piping and valves.
Safety Evaluation:	The carbon steel piping has been a continually worsening maintenance problem. Corrosion of the piping and the accumulation of crud in the piping have caused reduced flows and, in some cases, flow blockages. This change is expected to eliminate the corrosion problem and minimize the crud accumulation problem.
PORC Review:	PORC Meeting 1444, 10/15/85

Subject:	GO-NG-1, Rev. 6 Procedure Development
Description:	This procedure provides guidance for writing a general office procedure.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG, Rev. 4 Organization and Responsibilities
Description:	This procedure decribes the general organization and responsibilities of the offsite support organization.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG, Rev. 2 Master Bid List Input
Description:	This procedure provides guidance for adding vendors to the Master Bid List.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG-4, Rev. 3 Specification Review
Description:	This procedure provides guidance for reviewing equipment specifications.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A

Subject:	GO-NG-5, Rev. 2 Environmental Monitoring
Description:	This procedure provides guidance for conducting an environmental monitoring program.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG-9, Rev. 3 Plant Services Approved Suppliers List
Description:	This procedure provides guidance for developing and maintaining a Plant Services Approved Suppliers List.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG-12, Rev. 9 Procurement Document Control
Description:	This procedure provides guidance for maintaining control of equipment procurement documentation.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG-22, Rev. 3 Maintenance Support Conduct of Operation
Description:	This procedure describes the responsibilities of the maintenance support organization.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A

Subject:	GO-NG-33, Rev. 5 Operational Procurement Interfaces
Description:	This procedure describes the interfaces between Nuclear Support, Safety Audit and Engineering Review and the Purchasing Department required for procurement related activities.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG-44, Rev. 2 Inservice Inspection
Description:	This procedure provides guidance for conducting an inservice inspection program.
Safety Evaluation:	This revision to the procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A
Subject:	GO-NG-45, Rev. 0 Technical Support Conduct of Operations
Description:	This procedure describes the responsibilities of the technical support organization.
Safety Evaluation:	This procedure reflects management changes made to the offsite support organization.
PORC Review:	N/A

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the southern electric system

March 31, 1986

Docket No. 50-348

Dr. J. N. Grace, Administrator
U. S. Nuclear Regulatory Commission
Region II, Suite 3100
101 Marietta Street, N.W.
Atlanta, Georgia 30303

Dear Dr. Grace:

Attached for your review is the annual report required by 10CFR50.59 for 1985. This report summarizes changes to the plant performed in accordance with the provisions of 10CFR50.59 for Joseph M. Farley Nuclear Plant Unit 1. Also, in accordance with 10CFR50.59(b), 39 additional copies are provided for your use.

If you have any questions, please advise.

Respectfully submitted,

R. P. McDonald

RPM/JLO:ddb-D6

Attachment

cc: Mr. L. B. Long
Mr. E. A. Reeves
Mr. W. H. Bradford
Director, Office of Inspection and Enforcement, Washington, D.C. 20555
Document Control Desk
Washington, D.C. 20555

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