OPERATING DATA REPORT

Docket No. 50-311
Date June 10,1986
Telephone 935-6000
Extension 4451

Completed by Pell White

Operating Status

		Salem No. 2	Notes
1.	Unit Name		
2.	Reporting Period	May 1986	
3.	Licensed Thermal Power (MWt)	3411	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1115	
6.	Maximum Dependable Capacity (Gross MWe)	1149	
7.	Maximum Dependable Capacity (Net MWe)	1106	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

	This Month	Year to Date	Cumulative
11. Hours in Reporting Period	744	3623	40608
12. No. of Hrs. Reactor was Critical	713.4	3128.0	23453.8
13. Reactor Reserve Shutdown Hrs.	0	0	3533.6
14. Hours Generator On-Line	700.0	3087.7	22624.5
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWH)	2324078	10233464	69979162
17. Gross Elec. Energy Generated (MWH)	750850	3363200	22939680
18. Net Elec. Energy Generated (MWH)	718914	3219044	21753888
19. Unit Service Factor	94.1	85.2	55.7
20. Unit Availability Factor	94.1	85.2	55.7
21. Unit Capacity Factor (using MDC Net)	87.4	80.3	48.4
22. Unit Capacity Factor (using DER Net)	86.7	79.7	48.0
23. Unit Forced Outage Rate	5.9	12.3	34.7

24. Shutdowns scheduled over next 6 months (type, date and duration of each)
REFUELING 9-20-86

25. If shutdown at end of Report Period, Estimated Date of Startup:
N/A

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	6/30/80	8/2/80
Initial Electricity	9/1/80	6/3/81
Commercial Operation	9/24/80	10/13/81

8-1-7.R2

IE24
111

AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-311
Unit Name Salem # 2
Date June 10,1986
Telephone 609-935-6000
Extension 4451

Completed by Pell White

Month May 1986

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	930	17	1057
2	0	18	1064
3	0	19	1060
4	771	20	1065
5	1059	21	1054
6	1067	22	1050
7	1065	23	899
8	1081	24	1026
9	1075	25	1049
10	1058	26	1048
11	999	27	1032
12	1068	28	1052
13	1057	29	1052
14	1065	30	1047
15	1065	31	1014
16	1071		

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH MAY 1986

Completed by Pell White

Docket No. 50-311
Unit Name Salem No.2
Date June 10,1986
Telephone 609-935-6000
Extension 4451

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence
268	0501	F	5.2	A	5	-	SF	VESSEL	Other Nuclear Reactor Problems Misc.
270	0502	F	44.0	A	1	-	SF	VESSEL	Other Nuclear Reactor Problems Misc.
320	0523	F	12.4	A	5	-	HC	HTEXCH	Loss of Vacuum/High Back Pressure
348	0531	F	2.1	A	5	-	CH	PUMPXX	Feedwater Pump Drive Lube Oil System

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4 Exhibit G Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1 Salem as Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH MAY 1986

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: June 10, 1986
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPAL SYSTEM	SUBJECT
2EC-1501	Diesel Generators	Install control circuit transfer switches on the engine and generator control panels of each diesel generator to enable an operator to start the diesel generators under postulated fire and blackout conditions.
2EC-1566	Chilled Water System	Install a 10 PSIG relief valve at the chilled water expansion tank.
2EC-1785	Safe Guards Emergency Controls	Install new Lamda Power Supplies new Potter Brumfield relays in 2B SEC, 2A SEC and 2C SEC.
2EC-1970	Containment Personnel Hatch	Install caps and valving at personnel hatch - El. 100 & 130.
2EC-2000	Main Reheat & Turbine Bypass	Replace turbine first stage impulse pressure pressure transmitters PA-0195 and PA-0216 and mount so as to reduce vibration.
2SC-1168	Reactor Coolant System	Change reactor coolant flow transmitter from Fisher Porter model # 10B2496PB to new model and type specified by Engineering.

* Design Change Request

MAJOR PLANT MODIFICATIONS
REPORT MONTH - MAY 1986

DOCKET NO: 50-311
UNIT NAME: SALEM 2
DATE: MAY 10, 1986
COMPLETED BY: J. RONAFALVY
TELEPHONE: (609)339-4455

*DCR	SAFETY EVALUATION 10 CFR 50.59
2EC-1501	This design change improves the plant safety by making vital power available much sooner during certain postulated fire plus blackout conditions. Failure of the control transfer switch may cause the associated diesel to fail to start during a safeguards actuation. This failure is considered to be within the realm of the single failure criteria. This modification does not alter any plant process or discharge. Therefore, no unreviewed safety or environmental questions are involved.
2EC-1566	This design change does not jeopardize the pressure retaining boundary nor impact the operation or basic design criteria of the Chilled Water System. This modification does not alter any plant process or discharge. Therefore, no unreviewed safety or environmental questions are involved.
2EC-1785	This design change replaced existing components with ones manufactured by a different vendor. The replacement components have been seismically qualified to IEEE 344 which is to the same or higher stress levels as the original equipment. This modification does not alter any plant process or discharge. Therefore, no unreviewed safety or environmental questions are involved.
2EC-1970	All realistic failure modes have been considered but are not applicable. This design change meets GDC 56 requirements and complies with Safety Guide #11. The additional equipment meets the design criteria of seismic I and nuclear class II. This modification does not alter any plant process or discharge. Therefore, no unreviewed safety or environmental questions are involved.

* Design Change Request

UNIT 2 MAJOR PLANT MODIFICATIONS (contd)
REPORT MONTH - MAY 1986

*DCR	SAFETY EVALUATION 10 CFR 50.59
2EC-2000	The existing transmitters malfunction. The replacement ones are of better design. There is no functional change in the system or component. This modification does not alter any plant process or discharge. Therefore, no unreviewed safety or environmental questions are involved.
2SC-1168	All potential, realistic failure modes have been considered but are not applicable. Regulatory requirements do not apply. The replacement transmitters do not modify the function of the system. The original transmitters are considered obsolete by the manufacturer. This modification does not alter any plant process or discharge. Therefore, no unreviewed safety or environmental questions are involved.

* Design Change Request

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 2

WO NO	UNIT	EQUIPMENT IDENTIFICATION	
8604251090	2	21 CHILLER CONDENSER	
		FAILURE DESCRIPTION:	21 CHILLER CONDENSER HAS A TUBE LEAK.
		CORRECTIVE ACTION:	REPLACED DEFECTIVE CONDENSER TUBE, PRESSURE TESTED SYSTEM. REPLACED FILTER, DRYERS AND OIL.
8605020580	2	RTD ORIFICE	
		FAILURE DESCRIPTION:	22 RCP COLD LEG BYPASS RTD ORIFICE IS BLOWING AND LEAKING ONTO FLOOR.
		CORRECTIVE ACTION:	REPLACED GASKET IN FLANGES.
8605121173	2	21 SW PUMP AUTO CONTROL	
		FAILURE DESCRIPTION:	21 SERVICE WATER PUMP WILL NOT STAY IN THE AUTO CONTROL MODE.
		CORRECTIVE ACTION:	INSTALLED NEW OPERATE RESET RELAY IN BAILEY CABINET RC-116.

WO NO	UNIT	EQUIPMENT IDENTIFICATION	
8605121513	2	26 SW PUMP	
		FAILURE DESCRIPTION:	PUMP HAS EXCESSIVE PACKING LEAKAGE.
		CORRECTIVE ACTION:	REPACKED PUMP.
8605140500	2	21 SW STRAINER	
		FAILURE DESCRIPTION:	STRAINER HAS A SEVERE PACKING LEAK.
		CORRECTIVE ACTION:	REPLACED PACKING.
8605150882	2	REACTOR VESSEL FLANGE LEAKOFF INDICATOR	
		FAILURE DESCRIPTION:	BEZEL INDICATION HAS FAILED HIGH, VERIFIED VESSEL LEAKOFF WITH COMPUTER POINT.
		CORRECTIVE ACTION:	FOUND LOWER LEVEL AMP 2TM-401 FAILED HIGH. REPLACED S/N E092 WITH S/N E040. CALIBRATED TO AS LEFT VALUES OF TABLE 20.0 PAGE 5 OF 2IC-2.10.047. RETURNED CHANNEL TO SERVICE. TEMPERATURE INDICATION WAS 87 DEGREES F.

WO NO	UNIT	EQUIPMENT IDENTIFICATION	
8605190019	2	ACCUMULATOR LEVEL CHANNEL	
		FAILURE DESCRIPTION:	#24 ACCUMULATOR CHANNEL 2 LEVEL HI-LO ALARM IS ILLUMINATED. NO APPARENT LEVEL PROBLEM.
		CORRECTIVE ACTION:	REPLACED PC BOARD IN RELAY RACK #112 AND TESTED OKAY.
8605200073	2	CHARGING FLOW ORIFICE	
		FAILURE DESCRIPTION:	NO CLOSED INDICATION.
		CORRECTIVE ACTION:	REALIGNED LIMIT SWITCH ACTUATING ARM AND TIGHTENED. TIGHTENED LIMIT SWITCH BRACKET. CONTROL STROKED AND VERIFIED LIMITS.

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT NO. 2
MAY 1986

SALEM UNIT NO. 2

The Unit began the period operating at full power. On May 2, 1986 at 1717 hours, a controlled shutdown was initiated to perform environmental equipment qualification inspections in the Containment. As the Unit was taken off the line, on May 3, 1986 at 0332 hours, a Reactor Trip with Safety Injection occurred due to "High Steam Line Flow With Low Tave" which was apparently caused by a steam flow spike when the Generator output breakers were opened. On May 3, 1986 at 2231 hours, the Turbine was latched and power ascension to full power operations was initiated. Unit power was reduced to 65% on May 11, 1986 at 0220 hours due to limited system load demand. The Unit was returned to full power the same day at 0642 hours. During the period from May 23, 1986 to May 25, 1986 minor load reductions were required due to high river grasses and debris causing Circulating Water System problems. On May 31, 1986 at 2200 hours a load reduction to 61% was commenced in order to remove #21 SGFP from service for replacement of the governor. On June 1, 1986 at 1515 hours, work on #21 SGFP was completed and the Unit was returned to 100% power where it remained for the rest of the period.

REFUELING INFORMATION

COMPLETED BY: J. Ronafalvy

DOCKET NO.: 50-311
UNIT NAME: Salem 2
DATE: June 10, 1986
TELEPHONE: 609/935-6000
EXTENSION: 4455

Month May 1986

1. Refueling information has changed from last month:
   YES ____ NO X

2. Scheduled date for next refueling: September 20, 1986

3. Scheduled date for restart following refueling: November 19, 1986

4. A) Will Technical Specification changes or other license amendments be required?
   YES ____ NO ____

   Not determined to date ____

   B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
   YES ____ NO X
   If no, when is it scheduled? August 1986

5. Scheduled date(s) for submitting proposed licensing action:
   August 1986 if required

6. Important licensing considerations associated with refueling:
   NONE

7. Number of Fuel Assemblies:
   A) Incore 193
   B) In Spent Fuel Storage 140

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: March 2003

8-1-7.R4

PSE&G

Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

June 10, 1986

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 2
DOCKET NO. 50-311

In compliance with Section 6.9, Reporting Requirements for the Salem Technical Specification, 10 copies of the following monthly operating reports for the month of May 1986 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

JR:sl
cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Enclosures
8-1-7.R4

IE24
1/1