

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) <b>SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2</b>	DOCKET NUMBER (2) <b>0 5 0 0 0 3 6 1</b>	PAGE (3) <b>1 OF 0 1</b>
---	---	-----------------------------

TITLE (4)  
**PLANT PROTECTION SYSTEM ACTUATION ON LOW REACTOR COOLANT FLOW**

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQ. NUMBER	REV. NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		
06	03	86	86	012	00	06	27	86			
									DOCKET NUMBER(S) 0 5 0 0 0		

OPERATING MODE (9) **3**

POWER LEVEL (10) **0 0 0**

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

20.402(b)	20.405(c)	<input checked="" type="checkbox"/>	50.73(a)(2)(iv)	73.71(b)
20.405(a)(1)(i)	50.36(c)(1)	<input type="checkbox"/>	50.73(a)(2)(v)	73.71(c)
20.405(a)(1)(ii)	50.36(c)(2)	<input type="checkbox"/>	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(iii)	50.73(a)(2)(i)	<input type="checkbox"/>	50.73(a)(2)(viii)(A)	
20.405(a)(1)(iv)	50.73(a)(2)(ii)	<input type="checkbox"/>	50.73(a)(2)(viii)(B)	
20.405(a)(1)(v)	50.73(a)(2)(iii)	<input type="checkbox"/>	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME <b>H. E. MORGAN, STATION MANAGER</b>	TELEPHONE NUMBER AREA CODE: <b>7 1 4</b> NUMBER: <b>3 6 8 - 6 2 4 1</b>
--	--

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NFRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NFRDS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE)  NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

Abstract (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On 06/03/86, at 1708, with Unit 2 in hot standby, during a Quality Control inspection inside containment, smoke was observed in the area of Reactor Coolant Pump (RCP) P004 (EIIS System Code AB) (EIIS Component Code P). The control room was immediately notified and RCP P004 was promptly secured. A Plant Protection System (PPS) (EIIS System Code JC) actuation was generated from all four "Reactor Coolant Flow-Low" channels associated with Steam Generator (S/G) E-088 (EIIS Component Code HX).

An examination of RCP P004 revealed a smoldering cloth rag wedged between the pump casing and insulation. No evidence of flames or pump damage was observed. RCP P004 was restarted on 06/04/86, at 0310, and monitored for several days with no unusual indications.

An investigation was unable to determine when and why the cloth rag was placed at that location. Maintenance practices were reviewed and determined to adequately address clean-up activities. This is considered an isolated occurrence and no further action is planned.

Since all PPS actuated components functioned as designed, there are no safety consequences associated with this event.

8607110083 860627  
PDR ADOCK 05000361  
S PDR

IE22  
1/1



*Southern California Edison Company*

SAN ONOFRE NUCLEAR GENERATING STATION

P. O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

H. E. MORGAN  
STATION MANAGER

TELEPHONE  
(714) 368-6241

June 27, 1986

U. S. Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

Subject: Docket No. 50-361  
30-Day Report  
Licensee Event Report No. 86-012  
San Onofre Nuclear Generating Station, Unit 2

Pursuant to 10 CFR 50.73(a)(2)(iv), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving the Reactor Protection System. Neither the health and safety of plant personnel nor the health and safety of the public was affected by this event.

If you require any additional information, please so advise.

Sincerely,

Enclosure: LER No. 86-012

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)  
J. B. Martin (Regional Administrator, USNRC Region V)  
Institute of Nuclear Power Operations (INPO)

IE22  
11